



**GPU Nuclear**  
P.O. Box 388  
Forked River, New Jersey 08731  
609-693-6000  
Writer's Direct Dial Number:

March 15, 1983

Mr. Ronald C. Haynes, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

Dear Mr. Haynes:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Licensee Event Report  
Reportable Occurrence No. 50-219/83-09/01T

This letter forwards three copies of a Licensee Event Report (LER) to report Reportable Occurrence No. 50-219/83-09/01T in compliance with paragraph 6.9.2.a.2 of the Technical Specifications.

Very truly yours,

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:jal  
Enclosures

cc: Director (40 copies)  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director (3)  
Office of Management Information and  
Program Control  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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OYSTER CREEK NUCLEAR GENERATING STATION  
Forked River, New Jersey 08731

Licensee Event Report  
Reportable Occurrence No. 50-219/83-09/01T

Report Date

March 15, 1983

Occurrence Date

February 22, 1983

Identification of Occurrence

The results of local leak rate testing has identified that Main Steam Isolation Valves (MSIV) NS04A and NS04B failed to meet acceptance criteria. This constitutes operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications, paragraph 4.5.F.d.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.a.2.

Conditions Prior to Occurrence

The reactor was in the cold shutdown condition at the time the occurrence was identified. The reactor was in various operating modes prior to the occurrence.

Description of Occurrence

On February 16, 1983, while conducting Local Leak Rate testing, analysis identified the following valves with leakage in excess of the acceptance criteria of 11.9SCFH @ 20 psig. The results of the Leak Rate Test Program for those valves are as follows:

<u>Description</u>	<u>Valve</u>	<u>Date of Test</u>	<u>Leakage @ 20 psig.</u>
Main Steam Isolation	NS04A	2-14-83	15.03 SCFH
Main Steam Isolation	NS04B	2-14-83	15.95 SCFH

Apparent Cause of Occurrence

The cause of this occurrence is believed to be leakage through the valve packing.

Analysis of Occurrence

The analysis of this occurrence is in progress. An update of this LER will be submitted upon the completion of the leak rate test program.

Corrective Action

Valve packing replacement is planned for the MSIVs during the current refueling/maintenance outage.

NOTE: Any other valve(s) or penetration(s) identified in the Local Leak Rate Test Program that do not meet their acceptance criteria will be included in the update of this LER.