

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

FLORIDA POWER.CORPORATION CITY OF ALACHUA CITY OF BUSHNELL CITY OF GAINESVILLE CITY OF GAINESVILLE CITY OF KISSIMMEE CITY OF LEESBURG CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO SEBRING UTILITIES COMMISSION SEBRING UTILITIES COMMISSION SEBRING UTILITIES COMMISSION SEBRING UTILITIES COMMISSION CITY OF TALLAHASSEE

## DOCKET NO. 50-302

## CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57 License No. DPR-72

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated December 3, 1980, as revised January 19, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - C. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8209280449 820913 PDR ADDCK 05000302 P PDR  Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

 This license amendment becomes effective 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief Operating Reactors Branch #4 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: September 13, 1982

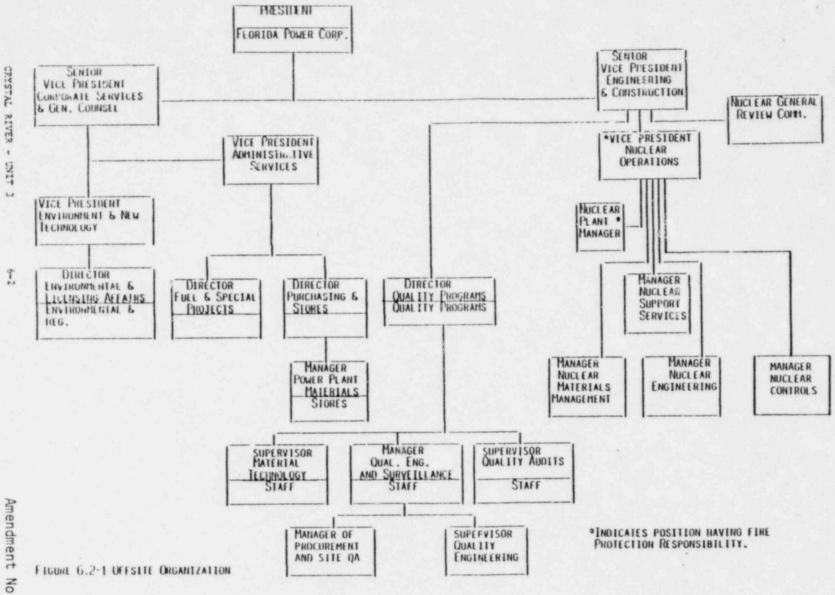
## ATTACHMENT TO LICENSE AMENDMENT NO. 57

# FACILITY OPERATING LICENSE NO. DPR-72

## DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

| 6-2    |      | 8 ( B |
|--------|------|-------|
| 6-3    |      |       |
| 6-5    |      |       |
| 6-6    |      |       |
| 6-7    |      |       |
| - 6-11 |      |       |
| 6-12   |      |       |
| 6-12a  | (new | page) |
| 6-13   |      |       |
| 6-14   |      |       |



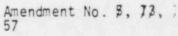
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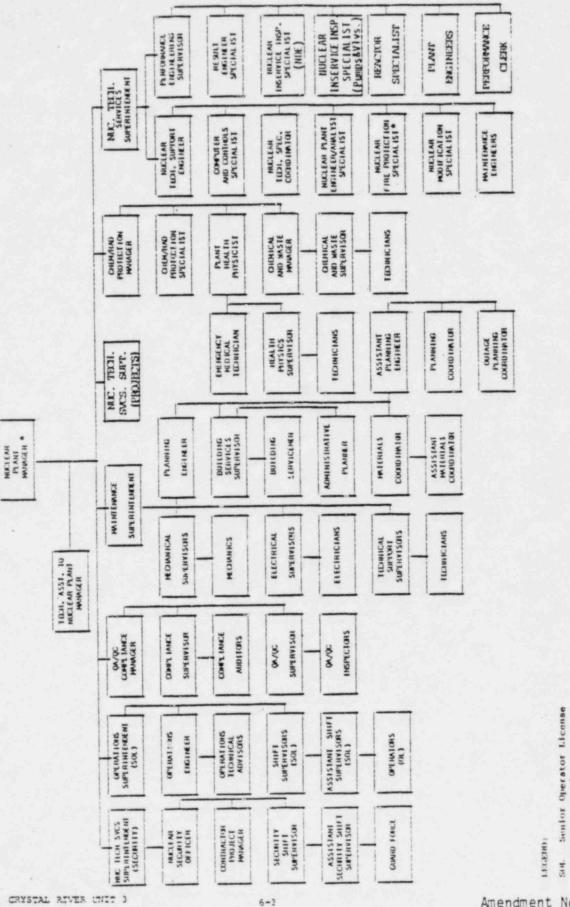




Indicates position having fire protection responsibility

Operator idcense

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# TABLE 6.2-1

# MINIMUM SHIFT CREW COMPOSITIONS

| LICENSE CATEGORY         | APPLICABLE MODES |       |
|--------------------------|------------------|-------|
|                          | 1, 2, 3, 8 4     | 5 & 6 |
| SOL                      | 2                | 1*    |
| OL                       | 2                | 1     |
| Non-Licensed             | 3                | 1     |
| Operations Tech. Advisor | 1                | 0     |

\*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling Individual supervising CORE ALTERATIONS after the initial fuel loading.

Shift crew composition may be less than the minumum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

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## 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Manager who shall meet or enceed the qualifications of Regulatory Guide 1.8, September 1975, and the Operations Technical Advisor, who shall have a Bachelor's degree, or the equivalent, in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

## 6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Plant Training Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except for Fire Brigade training sessions which shall be held at least quarterly.

## 6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

## FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Nuclear Plant Manager on all matters related to nuclear safety.

#### COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

| Chairman:<br>Member: | Technical Services Superintendent<br>Operations Superintendent |
|----------------------|--|
| Member:              | Maintenance Superintendent                                     |
|                      | Nuclear Technical Services Superintendent (Security)           |
| Member:              |  |
| Member:              | QA/QC Compliance Manager                                       |
| Member:              | Chem/Rad Protection Manager                                    |
| Member:              | Technical Support Engineer                                     |
| Member:              | Performance Engineering Supervisor                             |
| Member:              | At Large (Designated by Chairman)                              |
| Member :             | At Large (Designated by Chairman)                              |

## ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

## MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

## QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

## RESPONSIBILITIES

- 6.5.1.6 The Plant Review Committee shall be responsible for:
  - a. Review of 1) all procedures and changes thereto as required by Specification 6.812, 2) any other proposed procedures or changes [ thereto as determined by the Nuclear Plant Manager to affect nuclear safety.
  - Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to the Appendix "A" Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice-President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
  - Review of events requiring 24-hour written notification to the Commission.
  - g. Review of facility operations to detect potential nuclear safety hazards.
  - h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
  - i. Review of the Plant Security Plan and implementing procedures.
  - j. Review of the Emergency Plan and implementing procedures.

## AUTHORITY

- 6.5.1.7 The Plant Review Committee shall:
  - Recommend to the Nuclear Plant Manager written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.
  - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
  - c. Provide written notification within 24 hours to the Vice President, Nuclear Operations and the Nuclear General Review Committee of disagreement between the PRC and the Nuclear Plant Manager; however, the Nuclear Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## RECORDS

6.5.1.8 The Plant Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Nuclear Operations and Chairman of the Nuclear General Review Committee.

6.5.2 NUCLEAR GENERAL REVIEW COMMITTEE (NGRC)

#### FUNCTION

- 6.5.2.1 The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of:
  - a. Nuclear power plant operations
  - b. Nuclear engineering
  - c. Chemistry and radiochemistry
  - d. Metallurgy
  - e. Instrumentation and control
  - f. Radiological safety
  - g. Mechanical and electrical engineering
  - h. Quality assurance practices

## COMPOSITION

6.5.2.2 The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Nondestructive testing
- f. Instrumentation and control
- q. Radiological safety
- h. Mechanical and electrical engineering
- i. Administrative controls
- j. Environmental
- k. Quality assurance practices

## QUALIFICATIONS

6.5.2.3 The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit program:

- a. Chairman and Vice-Chairman-Bachelor of Science in engineering or related field and ten years related experience including five years involvement with operation and/or design of nuclear power plants.
- b. Member-Bachelor of Science in engineering or related field and five years related experience including three years involvement with operation and/or design of nuclear power plants.

#### ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however no more than two alternates shall participate as voting members in NGRC activities at any one time. CRYSTAL RIVER - UNIT 3 6-8 Amendment No. 5

#### RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRGC meeting shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Senior Vice President, Engineering and Construction, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

## 6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for REPURTABLE OCCURRENCES:
  - a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
  - b. Each REPORTABLE OCCURRENCE requiring 24-hour notification to the Commission shall be reviewed by the PRC and submitted to the NGRC and the Vice President, Nuclear Operations.

## 6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
  - a. The facility shall be placed in at least HOT STANDBY Within one hour.
  - b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations and to the NGRC within 24 hours.
  - c. A Safety Limit Viblation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures and (3) corrective action taken to prevent recurrence.
  - d. The Safety Limit Violation Report shall be submitted to the Commission, the NGRC and the Vice President, Nuclear Operations within 14 days of the violation.

## 6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:
  - a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
  - b. Refueling operations.
  - c. Surveillance and test activities of safety related equipment.
  - d. Security Plan implementation.
  - e. Emergency Plan implementation.
  - f. Fire Protection Program implementation.
  - g. Systems Integrity Program implementation.
  - h. Iodine Monitoring Program implementation.
- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:
  - a. The Emergency Plan, Security Plan, Fire Protection Plan and implementing procedures, Administrative Instructions and those test procedures associated with plant modifications shall be reviewed and approved by the PRC and the Nuclear Plant Manager prior to implementation.

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- b. For all other procedures, the review cycle shall consist of: an intradepartmental review by a Qualified Reviewer, an interdisciplinary review by Qualified Reviewer(s) in interfacing departments, as specified in Administrative Procedures, and approval by the responsible Superintendent or Manager, as specified by Administrative Procedures. The PRC shall then review the 10CFR 50.59 evaluation within 14 days of approval.
- c. The training and qualification of Qualified Reviewers shall be governed by Administrative Procedures, with final certification by the Nuclear Plant Manager. Recertification will be required on a periodic basis and upon transfer between departments. As a minimum, all Qualified Reviewers shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.3, 4.4, or 4.6, or the equivalent.
- d. Each procedure and administrative policy of 6.8.1 shall be reviewed on a periodic basis as set forth in Administrative -Procedures.

- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
  - a. The intent of the original procedure is not altered.
  - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License.
  - c. The change is documented and subsequently reviewed and approved within 14 days of implementation, in accordance with the requirements of Specification 6.8.2.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

## STARTUP REPORTS

- 6.9.1.1 A summary report of plant startup and power escalation testing will be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.
- 6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details requested in license conditions based on other commitments shall be included in this report.
- 6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and the resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

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### ANNUAL REPORTS

- 6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.
- 6.9.1.5 Reports required on an annual basis shall include:
  - a. A tabulation of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, 1 e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignments to various duty functions may be estimated based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80 percent of the total whole body dose received from external sources should be assigned to specific major work functions.
  - b. A list of the reactor vessel material surveillance capsules installed in the reactor at the end of the report period and a summary of any withdrawals or insertions of capsules during the report period. In supplying this information, the ownership of each capsule shall be indicated and the irradiation location in the vessel of each capsule which was inserted during the report period shall be identified.

This tabulation supplements the requirements of 20.407 of 10CFR Part 20.

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