UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) telecopied February 8, 1982, as confirmed April 5, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment was effective February 8, 1982 and expired February 10, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 27, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
3.7-9	3.7-9
3.7-10	3.7-10

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A.5 Oxygen Concentration	4.7.A.5 Oxygen Concentration
a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as stated in Specification 3.7.A.5.b.	The primary containment oxygen concentration shall be measured and recorded daily in the main control room.
b. Within the 24-hour* period subsequent to placing the reactor in the Run Mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. Deinerting may commence 24 hours prior to a shutdown.	*A 72-hour period is allowed for the startup in progress on February 7, 1982.
6. Containment Atmosphere Dilution (CAD)	6. Containment Atmosphere Dilution (CAD)
a. Operability Requirements After completion of the startup test program and demonstration of plant electrical output and thereafter whenever the reactor is in power operation, the post-LOCA containment Atmosphere Dilution (CAD) System must be operable and capable of supplying nitrogen to the primary containment for dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within seven days or the reactor must be taken out of power operation.	a. Functional Test The post-LOCA Containment Atmosphere Dilution (CAD) System shall be functionally tested once per operating cycle.
b. Seven-Day Nitrogen Supply After completion of the startup test program and demonstration of plant electric output and thereafter whenever the reactor is in power operation, the CAD System shall contain a minimum of 2000 gallons of liquid nitrogen. If this specification cannot be met, the minimum volume will be restored within seven days or the reactor must be taken out of power operation.	Seven-Day Nitrogen Supply The level in the liquid nitrogen storage tanks shall be recorded twice weekly.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A.6.c. H_2 and O_2 Analyzer	4.7.A.6.c. H_2 and O_2 Analyzer
Whenever the reactor is in power operation, there shall be at least one CAD System H_2 and O_2 analyzer serving the primary containment. If one H_2 and O_2 analyzer is inoperable, the reactor may remain in operation for a period not to exceed seven days.	Instrumentation surveillance is listed in Table 4.2-11.
d. Post-LOCA Repressurization Limit	
The maximum post-LOCA primary containment repressurization limit allowable using the CAD System shall be 30 psig. Venting via the SGTS to the main stack must be initiated at 30 psig following the initial post-LOCA pressure peak.	
7. Drywell-Suppression Chamber Differential Pressure	7. Drywell-Suppression Chamber Differential Pressure
Differential pressure between the drywell and suppression chamber shall be maintained equal to or greater than 1.5 psid except as specified in (1) and (2) below: If this specification cannot be met, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.	The pressure differential between the drywell and suppression chamber shall be recorded once each shift.
1) This differential pressure shall be established within 24 hours* after having placed the Mode Switch in the RUN mode. The differential pressure may be removed within 24 hours prior to achieving a shutdown.	*A 72-hour period is allowed for the start-up in progress on February 7, 1982.
2) This differential pressure may be decreased to less than 1.5 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, and the drywell-pressure suppression chamber vacuum breakers.	