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DONALD B. MILLER, Jr. SENIOR VICE PRESIDENT - MILLSTONE

Re: 10CFR50.73(a)(2)(i)

May 27, 1994 MP-94-366

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Reference: Facility Operating License No. NPF-49 Docket No. 50-423 Licensee Event Report 94-003-01

Gentlemen:

This letter forwards Licensee Event Report 94-003-01 which is being submitted to report on root cause and corrective action. This supplements Licensee Event Report 94-003-00 which was submitted pursuant to 10CFR50.73(a)(2)(i).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Donald B. Miller, Jr/ Senior Vice President - Millstone Station

DBM/RM:ljs

Attachment: LER 94-003-01

- cc: T. T. Martin, Region I Administrator
  - P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2 and 3
  - V. L. Rooney, NRC Project Manager, Millstone Unit No. 3

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		U.S. NU		(LEF	R)	MMISS		ESTIMATED B COLLECTION BURDEN ESTI BRANCH (M WASHINGTON	URDEN PER REQUEST IMATE TO NBB 7714). I DC 2055 1150-0104)	EXPI R RESPON 50.0 HR THE INFO U.S N 15-0001	OMB NO. 3 RES: 5/31/4 ISE TO COMPI S FORWARD REMATION AND RUCLEAR RE AND TO THE E OF MANU	95 LY WITH TH COMMEN D RECORD GULATORY PAPERWO	IS INFOR ITS REG S MANAG COMM ORK RED	ARDING SEMENT ISSION UCTION
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TITLE (4)	Millsto	one Nuclear F	Power St	ation l	Unit 3					05	000423		1 OF	4
	d Man	ual Containm	ent Isola	tion V	alve									
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MODE (9) 1		402(b)	T	20.405			1	50.73(8)				73.71(b)		
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NAME		LIC	CENSEE C	ONTAC	T FOR 1	HIS LI	ER (12	)		1 -	ELEPHONE N	IMBER (Inc	unia Aras	Codel
	n J. Ter	mple, Site Lic	ensing							T		437-59		
CO	MPLETE	ONE LINE FOR	REACH CO	OMPON	NENT FA	ILURE	DESC	RIBED IN T	HIS REP	PORT (1	3)			
CAUSE SYSTEM COM	AEPORTABLE					NENT	ENT MANUFACTURER		REPORTABLE TO NPRDS					
		1						1	L					
1		SUPPLEMENT	AL REPO	RT EXP	ECTED	(14)					ECTED	MONTH	DAY	YEAR
VES (If yes, complete EXP	ECTED SU	BMISSION DATE)			XNO						TE (15)			
ABSTRACT (Limit to 1 On January 25 valve for the In control room w was closed bet between the ur This condition be demonstrat automatic isola positions. The its intended iso locked. The co OPERABLE ke throughout this The root cause expectations. a	, 1994, tegrate vas imm fore tag hlockin is repo ed by v tion va outboa olation f ndition y locke s condifi was a As actio	at 0130 hour d Leak Rate 1 hediately info gging it close g and relocki rted as a viol rerifying that i lives, are close ard isolation y function. How did not have d closed auti tion. program faili on to prevent	rs, with the Test (ILR rmed, ar d to supp ng was 4 ation of T all penet sed by var valve war wever, whe safety s omatic in ure; attrik	ne plai T) pre bort a t hour fechni alves, l s alwa hile it ignific iboard	nt in Me ssuriza valve v work o s 10 mi cal Spe s not ca blind fla uys mai was tag ance. d isolati e to a p	ode 1 tion/d vas re rder fi nutes ecifica apable anges ntaine gged o The c on va	lepres locke or dov s. ation 4 e of bi ad clo closed ontair lve, w	4.6.1.1 wh eing close eactivate sed and v d for down ment pe hich was	n line w alve ha mainte nich req ed by C d autor was the nstrean netratio also al v and a	vas fou d beer enance DPERA natic v erefore n main on was ways r manag	and unlocke a. The tol containme BLE cont alves sec capable itenance, isolated maintaine gement d	ked. T d to ve tal time ent inte tainmen cured in of perfo it was by the ed close	he rify tha of their orming not ed cy on	at it 0

NRC F (5-92	U.S. NUCLEAR REGULA LICENSEE EVENT REPORT (I TEXT CONTINUATION		APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATI COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDI BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEME BRANCH (MINBS 7714). U.S. NUCLEAR REGULATORY COMMISSI WASHINGTON DC 20555-0001 AND TO THE PAPERWORK REDUCTI PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDG WASHINGTON DC 20503								
FACILITY	NAME (1)	DOCKET NUMBER (2)			LER NUM	states and contact of		PA	GE (3)		
	Milistone Nuclear Power Station Unit 3	05000423		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	02 01		04	
TEXT (	ff more space is required, use additional copies of NRC Form 366A) (17)	1				****	L				
1.	Description of Event										
	On January 25, 1994, at 0130 hours, with isolation valve for the Integrated Leak Ra red-tagged (danger-tagged) closed bu immediately informed and the valve was been unlocked to verify that it was close with downstream maintenance. It was re The total time between the unlocking an	tte Test (ILRT) Pres ut unlocked when then locked close d before red-tage elocked when disc	ssurizat perform d. An i ging it c overed	ion/ D ning a nvest losed durin	epressu surveilla gation r to supp g the va	rizati ance. eveal ort a lve po	on line wa The con ed that th work orde	as fou trol ro e valv er assi	nd om w e had ociate	1	
П.	Cause of Event										
	The root cause was a program failure; at on expectations. Operating procedures guidance on locked valves when hanging provided on locked valves when hanging	that covered valve g a tag clearance.	e lineup	s and	verificat	ion d	id not pro	vide		су	
HI.	Analysis of Event										
	This condition is reported under 10CFR5 plant's Technical Specifications. Technic demonstrated by verifying that all penetr automatic isolation valves, are closed by their positions. The outboard isolation values performing its intended isolation function maintenance, it was not locked (secured penetration was isolated by the OPERAE was also always maintained closed throu	al Specification 4 ations not capable valves, blind flam alve (3HVU*V005) h. However, while b) for a period of 4 BLE key locked clo	6.1.1 re e of bein ges, or remain it was r hours a bsed au	equire ng clo deact ned clo ed - ta and 10 tomat	s contail sed by ( ivated al osed, an agged se minute ic inboal	DPER utoma d wa ecure s. Th rd isc	it integrity IABLE con atic valves s therefor d for dow le contain lation val	to be ntainn s secu e cap nstrea ment	nent ired in able c am		
	The outboard valve (3HVU*V005) is a mi isolation for the ILRT pressurization/depr for the pressurization and depressurizati the emergency depressurization of the c automatic isolation valve (3HVU*CTV33/ locked switch.	essurization line. on of containment ontainment. This	The pu during line is it	the IL solate	of this I RT. It al d within	ine is so se conta	to provid erves as a ainment b	e a fic flow p y an	ow pat bath f		
	The condition did not have safety signific automatic isolation valve was always loc question the closed outboard manual iso while maintenance was being performed principal safety barriers, nor significantly outside the design basis of the plant.	ked, although not plation valve was u i. This condition o	deactiv under th lid not i	vated. ne adr nvolve	For the ninistrati	limite ve co us de	ed period entrol of a gradatior	of tim red- n of th	e in tag, e		
fV.	Corrective Action										
	The control room was immediately inform position surveillance. A night order was The total time between the unlocking an	promptly issued v	vith add	litiona	l restrict	ions				ve	
	As action to prevent recurrence, procedu are being provided to operations person that are locked.										

## U.S. NUCLEAR REGULATORY COMMISSION

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

## APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95

EARTIMES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB8 7714). U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20556-0001, AND TO THE PAPERWORK REDUCTION PROJECT (5150-0104), OFFICE OF MANAGEMENT AND BUDGET. WASHINGTON, DC 20505.

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Millstone Nuclear Power Station Unit 3	05000423	94	-	003	_	01	03	OF	04

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Management expectations on tagging locked valves have been documented viz memo and provided to operations personnel. The operations manager will review these expectations with all operating shifts. In addition, there is an ongoing effort to develop criteria to define which valves should be locked. Applying these criteria to the plant will help determine if the number of locked valves should be changed. The criteria will also reduce the potential for recurrence of errors involving locking valves.

## V. Additional Information

NRC Form 366A

(5 - 92)

Four cases of unlocked containment isolation valves have been reported since initial plant operation in 1986:

LER 93-022 reported an unlocked manual containment isolation valve due to equipment interface. It was reported that the outboard isolation valve had an unlocked padlock for up to 28 days following the opening of the valve to perform an operational surveillance test of the hydrogen recombiner. During the surveillance the inboard containment isolation valve was also opened to establish the necessary flowpath. Subsequent surveillances were performed to verify that the outboard valve was locked, and those surveillances were inadequate because they missed the fact that it had been unlocked from the time it was last operated. The root cause was a design deficiency with the locking cap. As corrective action, the padlocks were replaced with a different design.

LER 92-001 reported six valves which were missing locking devices. The valves were found closed but not locked as required. These valves are leakage monitoring connection vent valves which are used to test the containment isolation valves associated with Residual Heat Removal cold leg injection lines. The root cause was program failure – procedure deficiency, administrative error. The system lineups did not include the subject valves among those required to be locked closed. As corrective action the valves were locked closed and the penetration surveillances were updated to include these valves.

LER 91-021 reported that a containment isolation valve required to be locked closed was found to be open. The valve is a two inch, manually operated valve that provides inside containment isolation of a demineralized water line. The root cause was improper task verification. After being verified closed the subject valve was opened to support work evolutions in containment. While the need to close this valve was identified on the Shift Turnover Report, the Shift Supervisor removed the item from the turnover report without verifying that the valve had been closed and locked. The outboard containment isolation valve was verified closed throughout the event. As corrective action, the event and its associated root cause were discussed with Operations department personnel and were incorporated into Operations department training in order to emphasize the importance of containment isolation valves.

LER 90-021 reported that the Integrated Leak Rate Test (ILRT) Supply and Exhaust valve (3HVU\*V005) was unlocked and open. As part of a containment entry the Shift Supervisor (SS) initiated steps to facilitate a rapid pressurization of the sub-atmospheric containment if necessary. One of these steps was unlocking and opening 3HVU\*V005. The evolution was not covered under a plant procedure. The SS subsequently did not log the applicable Technical Specification Limiting Condition of Operation. The root cause was the failure to use the applicable procedure for a non-routine evolution. In addition there was a cognitive failure to recognize the Technical Specification implications of 3 HVU\*V005. As corrective action, the valve was closed and locked upon discovery; the SS was counseled on procedural usage and communications; and procedure changes were made to heighten personnel awareness of the contingent actions for rapid Containment pressurization.

By comparison, the current case involves a manual outboard isolation valve (3HVU\*V005) that had always been closed, and it was red-tagged closed without locking for 4 hours and 10 minutes, to support a work order associated with downstream components. During this time the inboard containment isolation valve was always closed and it was locked closed. The penetration had been secured as described above, throughout the limited condition.

U.S. NUCLEAR REGULATORY COMMISSION (5-92) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION			APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATIO COLLECTION REQUEST 50D HRS. FORWARD COMMENTS REGARDIN BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMEN BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20505-0001, AND TO THE PAPERWORK REDUCTIO PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGE WASHINGTON, DC 20503.								
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Millstone Nuclear Power Station Unit 3		05000423		94	-	003	-	01	04	OF	04
TEXT (If more space is required, us EIIS Codes	e additional copies of NRC Form 386A) (17	n			d						
System	Compo	pnent									
None	ISV (Va	live, Isolation)									