

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of
OMAHA PUBLIC POWER DISTRICT
(Fort Calhoun Station
Unit No. 1)

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Docket No. 50-285

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

Omaha Public Power District (the licensee) is the holder of Facility Operating License No. DPR-40 which authorizes the operation of the Fort Calhoun Station, Unit 1 (the facility) at a steady-state power level not in excess of 1500 megawatts thermal. The facility consists of a pressurized water reactor (PWR) located at the licensee's site in Washington County, Nebraska.

II.

Following the accident at Three Mile Island No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial

additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently, on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff had proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

III.

Omaha Public Power District responded to Generic Letter 82-05 by letters dated April 30, June 1, June 8, June 30, July 1, September 30, October 1, November 1, November 17, December 1 and December 17, 1982. Omaha Public Power District responded to Generic Letter 82-10 by letters dated June 4, July 1, October 27, and December 30, 1982. In these submittals, Omaha Public Power District confirmed that some of the items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitments or status were developed by the staff from the Generic Letters and the licensee-provided information.

Generic Letters 82-05 and 82-10 addressed thirteen and sixteen items, respectively. Of the ten items listed in Generic Letter 82-10 requiring a response, six items are not included in this Order. Item I.A.1.3.2 is part of a separate rulemaking; Items I.C.1, III.A.1.2 (2 items), and III.A.2.2 will be handled separately following Commission actions that would proceed as a result of its consideration of Commission Paper SECY 82-111, as amended; for Items II.K.3.30 and II.K.3.31 (one item), the staff review of the generic models under II.K.3.30 has not been completed, and II.K.3.31 is not required until one year after staff approval of the generic models.

Fourteen of the items addressed in this Order are considered by the licensee to be completed or to require no modifications. The staff's evaluation of the licensee's delays for the remaining three items is provided herein:

II.B.3 - Post Accident Sampling System

Installation of the post accident sampling system is complete. However, during final system check out, a number of problems arose. A minor software problem was identified in the system microprocessors. Calibration of the on-line pH meter is proving to be difficult. Operational problems have been found with the ionchromatograph. Lastly, the complete testing of the system cannot be completed until the reactor is at power; it is currently shut down for refueling. The licensee is working diligently in correcting these problems and plans to have the system completely operational by the end of June 1983. Interim post-accident sampling procedures are in place.

II.F.1.6 - Containment Hydrogen Monitors

This item will be delayed until restart from the present refueling outage. In late 1981, the hydrogen monitors were damaged during calibration; they were subsequently repaired. However, during testing being performed during final system checkout, two valves associated with one of the two monitors failed. Because of the valve failures, the suction and discharge line associated with one of the two monitors were capped. The valves, which are inside containment, will be repaired during the present refueling outage. The licensee presently has the capability to monitor the containment for hydrogen.

III.D.3.4 - Control Room Habitability

NUREG-0737 permitted the completion date to be determined by the licensee. The licensee states that all modifications will be completed by January 1984. Three items remain to be completed. Two of the items will be completed by June 30, 1983. These are "Instrumentation for Detection of Airborne Iodine Radiation in the Control Room" and "Electrical and Mechanical Modifications to the HVAC System." The third item will be completed by January 1, 1984. This is "Instrumentation for Monitoring of Toxic Chemical Gases." These items require long lead times because of the engineering work involved, equipment purchases and delivery cycles, and installation and testing for operability.

We find, based on the above evaluation, that 1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; 2) there is good cause for the several delays; and 3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these modifications and actions are required in the interest of public health and safety and should, therefore, be confirmed by Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT THE LICENSEE SHALL:

Implement and maintain the specific items described as complete in the attachments to this Order. Incomplete items shall be completed by no later than the dates shown in the attachments (as described in the licensee's submittals noted in Section III herein) and maintained thereafter.

V.

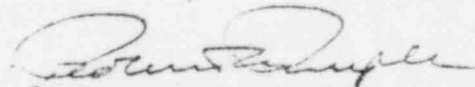
The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Purple, Deputy Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland
this 14th day of March,
1983.

Attachments:

1. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-05
2. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-10

LICENSEE COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS
FROM GENERIC LETTER 82-05

| ITEM | TITLE | NUREG-0737 SCHEDULE | REQUIREMENT | LICENSEE'S COMPLETION SCHEDULE (OR STATUS)* |
|----------|-------------------------------------|------------------------|---|--|
| I.A.3.1 | Simulator Exams | 10/1/81 | Include simulator exams in licensing examinations. | Complete |
| II.B.2 | Plant Shielding | 1/1/82 | Modify facility to provide access to vital areas under accident conditions. | Complete |
| II.B.3 | Post-accident sampling | 1/1/82 | Install upgrade post-accident sampling capability. | 6/83 |
| II.B.4 | Training for Mitigating Core Damage | 10/1/81 | Complete training program. | Complete |
| II.E.1.2 | Aux FW Indication & Flow Indicator | 7/1/81 | Modify instrumentation to level of safety grade. | Complete |
| II.E.4.2 | Containment Isolation Dependability | 7/1/81 | Part 5 - lower containment pressure setpoint to level compatible with normal operation. | Complete |
| II.E.4.2 | Containment Isolation Dependability | 7/1/81 | Part 7 - isolate purge and vent valves on radiation signal. | Complete |

*Where complete date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.

LICENSEE COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS
FROM GENERIC LETTER 82-05

| ITEM | TITLE | NUREG-0737 SCHEDULE | REQUIREMENT | LICENSEE'S COMPLETION SCHEDULE (OR STATUS)* |
|--------|---------------------|------------------------|---|--|
| II.F.1 | Accident Monitoring | 1/1/82 | (1) Install noble gas effluent monitors. | Complete |
| | | 1/1/82 | (2) Provide capability for effluent monitoring of iodine. | Complete |
| | | 1/1/82 | (3) Install in-containment radiation-level monitor. | Complete |
| | | 1/1/82 | (4) Provide continuous indication of containment pressure. | Complete |
| | | 1/1/82 | (5) Provide continuous indication of containment water level. | Complete |
| | | 1/1/82 | (6) Provide continuous indication of hydrogen concentration in containment. | 1982/1983 Refueling Outage (12/82) |

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.

| ITEM | TITLE | NUREG-0737 SCHEDULE | REQUIREMENT | LICENSEE'S COMPLETION SCHEDULE (OR STATUS) * |
|----------------|---|--|---|---|
| I.A.1.3.1 | Limit Overtime | 10/1/82 per Gen. Ltr. 82-12 dtd. 6/15/82 | Revise administrative procedures to limit overtime in accordance w/NRC Policy Statement issued by Gen. Ltr. No. 82-12, dtd. June 15, 1982. | Complete |
| I.A.1.3.2 | **Minimum Shift Crew | To be superseded by Proposed Rule. | To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units. | To be addressed when Final Rule is issued. |
| I.C.1 | **Revise Emergency Procedures | Superseded by SECY 82-111 | Reference SECY 82-111, Requirements for Emergency Response Capability. | To be determined |
| II.D.1.2 | RV and SV Test Programs | 7/1/82 | Submit plant specific reports on relief and safety valve program | Complete |
| II.D.1.3 | Block Valve Test Program | 7/1/82 | Submit report of results of test program | Complete |
| II.K.3.30 & 31 | **SBLOCA Analysis | 1 yr. after staff approval of model | Submit plant specific analyses. | To be determined following staff approval of model |
| III.A.1.2 | **Staffing Levels for Emergency Situations | Superseded by SECY 82-111 | Reference SECY 82-111, Requirements for Emergency Response Capability | To be determined |
| III.A.1.2 | **Upgrade Emergency Support Facilities | "" | "" | "" |
| III.A.2.2 | **Meteorological Data | "" | "" | "" |
| III.D.3.4 | Control Room Habitability | To be determined by licensee | Modify facility as identified by licensee study | 1/84 |

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.

**Not Part of Confirmatory Order