

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
)
CONSUMERS POWER COMPANY) Docket No. 50-155
)
(Big Rock Point Plant))

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

Consumers Power Company (the licensee) is the holder of Facility Operating License No. DPR-6 which authorizes the operation of the Big Rock Point Plant (the facility) at steady-state power level not in excess of 240 megawatts thermal. The facility is a boiling water reactor (BWR) located at the licensee's site in Charlevoix County, Michigan.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification

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of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982¹. Subsequently, on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff had proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

¹The letter sent to Consumers Power Company was lost in the mail and this was not determined until they were questioned about their response on April 30, 1982. The letter was then telecopied to them and they responded 30 days after receipt.

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III.

Consumers Power Company (CPC) responded to the Generic Letter 82-05 by letter dated May 14, 1982; CPC responded to the Generic Letter 82-10 by letter dated May 25, 1982. In these submittals, CPC confirmed that some of the items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitments or status were developed by the staff from the Generic Letters and licensee-provided information.

There are several items in GL 82-10 that as noted in Attachment 2 have licensee schedules which are yet to be determined. These items are, therefore, not included in this Order. The licensee considers some of the items addressed in this Order to be completed or to require no modifications. These items are so noted in Attachments 1 and 2. The staff's evaluation of the licensee's delays for implementation of the remaining items is provided herein.

II.B.2 Plant Shielding

II.B.3 Post Accident Sampling

II.F.1.6 Containment Hydrogen Monitor

III.D.3.4 Control Room Habitability

By letter dated March 31, 1981, CPC submitted a Probabilistic Risk Assessment (PRA) for the Big Rock Point Plant to the NRC. CPC also requested that implementation of several NUREG-0737 items be deferred for Big Rock Point pending completion of the NRC's review of PRA.

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The basis for this request was that the PRA would show that these items are not necessary for Big Rock Point. By letter dated August 12, 1981, NRC informed CPC that implementation of these four items, among others, would be deferred for Big Rock Point pending completion of the NRC's review of the PRA. The NRC's review of the PRA is presently scheduled to be completed in early 1983. After completion of the NRC review, the NRC will decide if these four items should be implemented at Big Rock Point. Therefore, items II.B.2, II.B.3, II.F.1.6 and IV.D.3.4 are not included in this Order.

II.F.1.1 Noble Gas Monitor

II.F.1.2 Iodine/Particulate Monitor

By letter dated February 16, 1983, CPC committed to have the stack monitoring system which fulfills both TMI Items II.F.1.1 and II.F.1.2 fully operational by December 31, 1983. Installation was delayed from previous commitments by failure of the instrument vendor to deliver the proper equipment in a timely fashion. The equipment is now on-site at Big Rock Point and installation is essentially complete. However, the licensee has been unable to obtain spare parts from the vendor to ensure that the system can be restored to operation in a timely manner if the equipment breaks down and needs repair. The current monitoring system used to quantify high level releases from the stack is still operational and will remain operational as an interim compensatory measure until the new monitoring system is ready to be declared fully operational.

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We find, based on the above evaluation, that: 1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; 2) there is good cause for the delays (delay of vendor delivery of equipment); and 3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these commitments are required in the interest of public health and safety and should, therefore, the licensee's commitment should be confirmed by the Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall:

Implement and maintain the specific items described in the Attachments to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachments.

The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

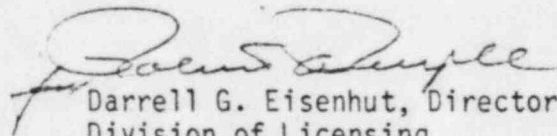
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If a hearing is requested by the licensee, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,
this 14 day of March,
1983.

Attachments:

1. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-05
2. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-10

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.3.1	Simulator Exams	10/01/81	Include simulator exams in licensing examinations	Complete
II.B.2	**Plant Shielding	01/01/82	Modify facility to provide access to vital areas under accident conditions	Deferred until NRC completes review of PRA
II.B.3	**Post-Accident Sampling	01/01/82	Install upgrade post-accident sampling capability	Deferred until NRC completes review of PRA
II.B.4	Training for Mitigating Core Damage	10/01/81	Complete training program	Complete
II.E.4.2	Containment Isolation Dependability	07/01/81	Part 5-lower containment pressure setpoint to level compatible w/normal operation.	Complete
		07/01/81	Part 7-isolate purge & vent valves on radiation signal	Complete
II.F.1	Accident Monitoring	01/01/82	(1) Install noble gas effluent monitors.	12/83
		01/01/82	(2) Provide capability for effluent monitoring of iodine	12/83
		01/01/82	(3) Install incontainment radiation-level monitors	Complete
		01/01/82	(4) Provide continuous indication of containment pressure	Complete
		01/01/82	(5) Provide continuous indication of containment water level.	Complete
		01/01/82	** (6) Provide continuous indication of hydrogen concentration in containment.	Deferred until NRC completes review of PRA

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility
 **Not part of Confirmatory Order.

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
11.K.3.19	Interlock on Re-circulation Pump	07/01/81	Install interlocks on re-circulation pump loops	Complete, NRC concluded that modification was not necessary for Big Rock Point
11.K.3.27	Common Reference Level	07/01/81	Provide common reference level for vessel level instrumentation	Complete

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-10

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)
I.A.1.3.1	Limit Overtime	10/01/82 per Gen Ltr 82-12 dtd. 6/15/82	Revise administrative proce- dures to limit overtime in accordance w/NRC Policy Statement issued by Generic Ltr. No. 82-12, dtd 6/15/82	Complete
I.A.1.3.2	* Minimum Shift Crew	To be super- seded by Proposed Rule.	To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units	To be addressed when Final Rule is issued.
I.C.1	* Revise Emergency Procedures	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be determined
II.D.1.2	RV and SV Test Programs	07/01/82	Submit plant specific reports on relief valve and safety valve tests.	Complete
II.K.3.18	ADS Actuation	09/30/82	Submit revised position on need for modifications.	Complete, NRC concluded that mo- dification was not necessary.
II.K.3.30 & 31	* SBLOCA Analysis	1 year after staff approval of model	Submit plant specific analyses	To be determined following staff approval of model
III.A.1.2	* Staffing Levels for Emergency Situations	Superseded by 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be Determined
III.A.1.2	* Upgrade Emergency Support Facilities	" " "	" " " "	" " "
III.A.2.2	* Meteorological Data	" " "	" " " "	" " "
III.D.3.4	* Control Room Habitability	To be deter- mined by licensee	Modify facility as identified by licensee's study.	Deferred until NRC completes review of PRA

* Not Part of Confirmatory Order.