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QUANTITATIVE SOFTWARE RELIABILITY ANALYSIS OF COMPUTER CODES RELEVANT TO NUCLEAR SAFETY

ARGONNE NATIONAL LABORATORY

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Abstract

This report presents the results of the first year of an ongoing research program to determine the probability of failure characteristics of computer codes relevant to nuclear safety. An introduction to both qualitative and quantitative aspects of nuclear software is given. A mathematical framework is presented which will enable the a priori prediction of the probability of failure characteristics of a code given the proper specification of its properties. The framework consists of four parts: 1) a classification system for software errors and code failures; 2) probabilistic modeling for selected reliability characteristics; 3) multivariate regression analyses to establish predictive relationships among reliability characteristics and generic code property and development parameters; and 4) the associated information base. Preliminary data of the type needed to support the modeling and the predictions of this program are described. Illustrations of the use of the modeling are given but the results so obtained, as well as all "results" of code failure probabilities presented herein, are based on data which at this point are preliminary, incomplete, and possibly non-representative of codes relevant to nuclear safety.

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A2226

Software Reliability

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Executive Summary

This report presents the results of the first year of an ongoing research program to determine the probability of failure characteristics of computer codes relevant to nuclear safety. An introduction to both qualitative and quantitative aspects of nuclear software is given. A mathematical framework is presented which will enable the a priori prediction of the probability of failure characteristics of a code given the proper specification of its properties. Preliminary data of the type needed to support the modeling and the predictions of this program are described. Illustrations of the use of the modeling are given but the results so obtained, as well as all "results" of code failure probabilities presented herein, are based on data which at this point are preliminary, incomplete, and possibly non-representative of codes relevant to nuclear safety.

1.0 Introduction and Summary

This report describes the results of the initial phase of a research program sponsored by the Nuclear Regulatory Commission (NRC) and designed to establish quantitative predictions of reliability for the computer software used in the nuclear industry. The overall objective of this program is to develop generically applicable analytic tools and an associated information base that can be used to estimate the probability of success and other reliability characteristics of any specific computer code relevant to nuclear reactor safety, given the characteristics of the specified code. Relevant applications in nuclear safety include both analysis computer codes used in design and safety assessment and data acquisition codes used in real and simulated on-line and emergency situations.

Illustrative examples of the type of question that this research program is designed to answer or to help to answer are:

- 1. What is the probability that a large safety code yields the correct (or at least a conservative estimate of the) containment pressure for a hypothetical loss of coolant accident? Although the phenomenological modeling uncertainties are evident and may dominate the "answer" a contributor to this probability is the uncertainty in whether the associated software is input and exercised properly. This latter contribution is to be assessed in this program.
- 2. What is the probability that a computer program monitoring and analyzing real time nuclear reactor sensor data correctly interprets, analyzes, and presents this data to a reactor operator in an emergency situation?

To not only answer questions such as these for existing specific computer programs, but to provide generically applicable answers for use in enhancing nuclear safety, the designated specific goals for the analytic tools and information developed as part of this program are to:

- 1. Allow NRC to make an a priori estimate of a computer code's probability of successful operation given the code's general properties and its intended application. These properties include not only physical characteristics such as size, complexity and programming language but development and quality assurance parameters such as testing history and operational history after testing. It is to be emphasized that the usage or application of a code can be a strong determinant of its probability of successful operation.
- Provide information for NRC to determine quantitative criteria for the acceptability of software relevant to nuclear safety for eventual use in the licensing arena.

The motivation for this program is provided by the important role played by computers in reactor design and safety analysis and by the growing need for improved on-line information processing. Propagation of software errors in reactor design codes can lead to analytical predictions, resulting among other things, in unsafe plant hardware design, unsafe operational and safety parameters such as trip settings, and erroneous predictions of plant lifetime related parameters. Software errors in reactor safety assessment codes may result in either false "security" in which needed safety measures go unrecognized or in undue alarm leading to the incorporation of unnecessary, costly, and even potentially unsafe mitigation devices. As pointed out by Fabic [1.1], the accuracy of a safety code in predicting the response of a plant to a nuclear accident is a function of programming errors as well as a host of plant and modeling uncertainties. All these factors make it clear that software reliability must be integrated into quantitative assessments of nuclear safety.

The growing need for data acquisition and information processing codes in online situations has been highlighted by the President's Commission on the Three Mile Island Accident [1.2]. Long et al. [1.3] and Ramamoorthy et al. [1.4] have emphasized the necessity for a methodology to assess the development and validation of critical software for nuclear power plants. On-line safety protection system software such as the Core Protection Calculator (CPC) implemented in the Arkansas Nuclear One, Unit 2 [1.5] and a proposed disturbance analysis system (DAS) [1.6] are examples of critical software. The CPC is a software system which processes sensor information and if necessary initiates a High Local Power Density or Low DNBR trip. DAS is a computer network of hardware and software which allows plant personnel to access and act upon realtime information relating to possible causes and consequences of disturbances and corrective actions. Propagation of software errors through these systems can lead to delayed or erroneous control and operator actions, and unnecessary forced outages. For example, in a review of 14 events [1.7] in which reactor control was either lost or impaired at one Canadian power station, nearly half of the events were traced to deficiencies in control computer software. These deficiencies were about evenly split between changes introduced since software implementation and problems with code design.

Given the goals of this research, the software reliability program in FY 81 was broken into interrelated tasks:

- Review the literature for and discuss with experts in the field known mathematical approaches and associated data bases for quantifying software reliability. Investigate this information as to its applicability for this program.
- Develop a mathematical approach that will meet the specific goals outlined above. As will be described in Section 3, existing models have limited usefulness here. Further, to support the approach of this program, the existing data base will have to be greatly expanded. This is discussed in Section 4.

The work that was done to satisfy the first task, which included information exchanges and discussion with personnel in aerospace, defense, communications, and computer technologies, clearly pointed out the need for the quantitative analytic capability and associated information base that are the end products of this program.

The remainder of this report discusses the progress that has been made on the program to date. Section 2 of this report briefly discusses some of the design and development considerations that enter into the production of reliable software. Organizations instrumental in establishing software standards in the nuclear industry are cited. Finally, problems in the validation and verification of nuclear-safety-related software are discussed. It is intended to give the reader insight into qualitative reliability characteristics of computer software.

Section 3 discusses the functional requirements and desired results of this program in mathematical forms. The mathematical reliability models that have been published are described. Obtaining and analyzing failure data with these models to understand their underlying bases was a significant part of the FY81 research. The role of these models in assessing nuclear software is described in light of our studies with them. A mathematical approach that will meet the quantitative requirements of this program is outlined with descriptions of the necessary supporting mathematical and statistical models and information. The implementation of this approach is next described.

The most significant problem in developing mathematical models to predict software reliability characteristics is the lack of failure data upon which to base and subsequently test these models. As will be seen in Section 3, published models do not explicitly treat code characteristics such as complexity, testing history, and application, which are critical to a code's probability of successful operation. Further, published models were developed to evaluate the reliability of continuously running codes such as those used in continuous data acquisition and generally predict failure rates vs. time, a parameter which has little significance in evaluating the failure probability of an analysis code in a specific execution. The accumulation of non-nuclear and nuclear software failure data, both published and unpublished, as well as the initiation of documentation of related failure data within ANL is being pursued with the objective to categorize code failure data according to the general code properties. This categorization is necessary for research goal number 1 to be achieved. Section 4 reports progress to date and future efforts in accumulating and generating these data.

Section 5 discusses initial results of analyzing published data to obtain preliminary but illustrative reliability estimates. Far more extensive studies will be performed but the flavor of the studies is shown in Section 5. This section also describes some very brief results of preliminary surveys to obtain subjective estimates of software reliability among computer code developers and users. Because the data to date are preliminary, unverified, and possibly non-representative of related nuclear software, these results are presented only for illustration.

Section 6 outlines the directions that should be followed to best achieve the goals of this research program. Specific research paths and tasks are suggested with the end product of each described.

References

- S. Fabic, "Code Assessment for Nuclear Reactor Accident Analysis Programs," Trans. Am. Nucl. Soc., 35, 254 (1980).
- 1.2 J. G. Kemeny (Chairman), "Report of the President's Commission on the Accident at Three Mile Island," U.S. Government Printino Office, Washington, D.C., October (1979).
- 1.3 A. B. Long, et al., "A Methodology for the Development and Validation of Critical Software for Nuclear Power Plants," 1st International Computer Software and Applications Conference, IEEE COMSAC 77, 620 (1977).
- 1.4 C. V. Ramamoorthy, et al., "A Systematic Approach to the Development and Validation of Critical Software for Nuclear Power Plants," Proceedings 4th International Conference on Software Engineering, 231 (1979).
- 1.5 Combustion Engineering, Inc., "Assessment of the Accuracy of PWR Safety System Actuation as Performed by the Core Protection Calculators," CENPD-170, July (1975).
- 1.6 A. B. Long, "Technical Assessment of Disturbance Analysis Systems," Nuclear Safety, Vol. 21-1, 38 (1980).
- L. Magagna, "Control Computer Systems in Generating Stations in Ontario Hydro Design Approach and Experience," 1976 EEI Engineering Computer Forum, 155 (1976).

2.0 Considerations in the Development of Reliable Software

This section briefly discusses some of the design and development considerations that enter into the development of reliable software. Organizations instrumental in establishing software standards, especially in the nuclear industry, are cited. Finally, comments regarding the validation and verification of codes are offered. It is seen that safety codes in the nuclear industry pose special problems.

The initial phase of software development, as depicted in Figure 1, emphasizes the structure of software, with the assumption that well structured or modular software results in improved reliability. By simplifying logic and program control, program functions become easier to understand and consequently, testing of such software is facilitated [2.1,2.2,2.3].

In the fast reactor physics community, there is an ongoing, major effort to implement standardization procedures and guidelines to maximize the exchangeability of software, as proposed by the Committee on Computer Code Coordination (CCCC) [2.4] established by DOE. Recommendations of the CCCC also include standardization of utility subroutines, data storage, and input/output (I/O) files. This reduces the introduction of new errors (bugs) and the dependence of coding upon a specified computer, thereby facilitating exportability of software, exchangeability of information, and availability of documentation. The CCCC is strongly supported by the national laboratories and to a lesser extent by the nuclear industry.

Another approach to the development of reliable software is redundancy. This is a familiar concept in the hardware design of nuclear power plants. Application of a similar concept to fault-tolerant software development has been suggested by Randall [2.5]. Redundancy may be accomplished through code design rather than by simple replication of programs. For example, in the operation of nuclear power plants, redundancy in the on-line software can be provided by comparing computed signals with signals from the sensors located at the various parts of the plants. This approach provides independent validation of both the sensors and the software.

In the military, aerospace, and communication industries, where development of highly reliable software is critical, emphasis [2.6,2.7,2.8] has been focused on guidelines and standard procedures to achieve highly reliable software. In the nuclear field, efforts in software reliability have been also directed to developing or assuring reliable software as cited earlier.

In the nuclear energy field, standards committees and organizations have developed standards applicable to nuclear software as follows:

1. American National Standard Institute (ANSI)

ANSI X3.9-1966	American National Standard FORTRAN
ANSI/IEEE Std 730	IEEL Trial-Use Standard for Software
	Quality Assurance Plan

2. American Nuclear Society (ANS)

ANS-STD.3-1971	Recommended Programming Practices to Facilitate the Interchange of Digital Computer Programs					
ANS-10.3/N413-1974	Guidelines for the Documentation of Digital Computer Programs (also approved by ANSI)					

3. Nuclear Standards Management Center

Coordinates development of DOE nuclear program standards

4. Committee on Computer Code Coordination

LA-6941-MS Standard Interface Files and Procedures for Reactor Physics, Version IV

 Reactor Development Technology (RDT), now designated as Nuclear Energy (NE) Standards

RDT Std F1-4

Computer Coding, Documentation, and Distribution (Draft, 1975)

6. National Energy Software Center

7. Nuclear Energy Programs/Organizations

LA-7812-MS	Quality Assurance for TRAC Development
R0010-1001-SA-00	Argonne National Laboratory Quality Assurance Policy and Procedures Manual

Uniform application of these standards throughout the industry will help ensure reliable software. For the majority of nuclear codes specific appreaches to software validation and verification (V&V) are adequate. Code validation and verification is normally accomplished by testing each of the several modules in a code and the code as a whole. Several possible modes of V&V are as follows:

- Comparison of the results generated by the module to known analytic solutions to the equation sets solved in the module or appropriate simplification of these sets;
- Comparison of the results generated by the module to the results of other computer codes that model the same phenomena;
- Comparison of the results generated by the module to experimental results; and
- Comparison of the results generated by the module to the "expected behavior" of the models it contains.

Although such comparisons may validate portions of a code they rarely test the whole system. It is important to note that testing can guarantee the presence of, but not the absence of, bugs.

There are codes, however, especially in the risk assessment area for which this procedure is not totally satisfactory because:

- Either analytical solutions do not exist or an inability to place bounds on uncertainty associated with data makes the solutions inconclusive.
- 2. No other codes of this type exist.
- 3. Experimental results do not exist or are unavailable.
- 4. The driving phenomena are not understood well enough to compare the results with "expected behavior".

Clearly there is a need to develop additional procedures and methods to assess reliability of codes which fall in the category of the unverifiable.



Figure 2.1 A Structure for Developing Reliable Software

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SOFTWARE RELIABILITY

References

- 2.1 E. C. Nelson, "Software Reliability," TRW-SS-75-05, (1975).
- 2.2 B. W. Boehm, "Software and Its Impact: A Quantitative Assessment," Datamation, May (1973).
- 2.3 H. D. Mills, "Software Development," IEEE Transactions on Software Engineering, Vol. SE-2, No. 4, (1976).
- 2.4 R. D. O'Dell, "Standard Interface Files and Procedures for Reactor Physics, Version IV," LA-6941-MS, (1977).
- 2.5 B. Randall, "System Structure for Software Fault Tolerance," Proceedings of International Conference on Reliable Software (1975).
- 2.6 "Final Report of the Joint Logistics Commanders Software Workshop," Vol. 1, Army Material Development and Readiness Command, Naval Material Command, Air Force Logistics Command and Air Force Systems Command, October (1979).
- 2.7 A. D. Schuman, "Proposed Multi-Service Documentation Standards and Requirement," Panel Presentation at the IEEE Reliability and Maintainability Symposium, (1981).
- 2.8 "Software Quality Assurance Program Requirements," MIL-S-52779.
- 2.9 Record of the IEEE-NRC Working Conference on Advanced Electrotechnology Applications to Nuclear Power Plants, (1980).
- 2.10 J. S. Moore and L. Lamport, "Program Verification: An Approach to Reliable Hardware and Software," Trans. Am. Nuc. Soc., 35 (1980).
- 2.11 H. R. Downs, "Automated Tools for the Verification of Computer Programs," Trans. Am. Nucl. Soc., 35 (1980).
- 2.12 T. E. Dunn, "Development of a Standard for Computer Code Control and Verification," Trans. Am. Nuc. Soc., 35 (1980).

3.0 Quantitative Software Reliability Analysis

This section describes an analytic framework through which the goals of this research program, as described in Section 1, can be met. The mathematical approach that is formulated here is designed to be able to yield predictions of software probability or reliability characteristics for all types of computer codes relevant to nuclear safety. Codes of interest in nuclear safety applications fall into two broad categories: What might be called discrete task codes, exemplified by analysis codes such as RELAP or TRAC, are run to accomplish a specified task and then stopped for examination of computational results. The second category consists of continuous task codes exemplified by data acquisition codes such as might be used to provide a reactor operator with a continuous display of data from an operating reactor. Because these two types of codes have conceptually different measures of reliability, it is important to formulate an analytic framework which encompasses both.

Section 3.1 discusses the functional requirements of this analytic framework and the form of the desired results. It begins with some basic definitions and then describes mathematical parameters that can be used to measure the probabilities of successful or unsuccessful operation of computer codes in nuclear applications. These probabilities are affected by the way failures are categorized and counted, by the specific properties and usage of the computer code, and by the techniques used in developing and testing the code. The concept of classifying the failure, computer code property, and code development parameters that affect these probabilities is described and examples are shown. The potential use of regression analyses to assess the functional dependencies between the calculated software probability characteristics and these parameters is cited.

Section 3.2 describes mathematical modeling that can be used to fill out the analytical framework. The section begins with a brief review of failure rate models which have been used extensively for more than ten years in communications, aerospace, and defense applications. The applicability that these models have in the mathematical framework of this program is discussed in view of the special needs of the nuclear industry. It will be seen that the approach used in the models published to date have rather limited usefulness in meeting the goals of this program because they have been primarily designed for and applied to calculating the reliability characteristics of the so called continuous task codes. Modeling that comprises the analytic framework which is proposed here to encompass both the discrete and continuous task codes is then described. Possible model extensions that may prove useful in the longer term are proposed and the concepts and models needed to support or augment this framework are discussed.

Finally, Section 3.3 summarizes data requirements needed to implement the models described in Section 3.2. Some alternative approaches which may be attempted in the absence of the desired data are also outlined, and the use of standard statistical analysis packages to obtain correlations between probabilistic code parameters and other code characteristics is briefly discussed.

3.1 Functional Requirements and Desired Results

Before describing the functional requirements and desired results of the analytical framework in detail, a few comments will be made regarding definitions. In the discussions that follow, reference will be made to the detection of code errors. A code error will be regarded as synonymous with a software error and is here defined as any defect in a line of code or in the input data which can cause the computer code to fail. Code failure includes abnormal termination, normal termination with erroneous results, or any other unacceptable departure of program operation from the required operation.

It may sometimes be convenient to refer to code faults rather than errors. For example, a computer code may compute an incorrect temperature. It may be necessary to correct several code errors to correct the temperature calculation, but the incorrect temperature could be regarded as the result of a single fault. As envisioned here, a code failure may be the result of one or more faults and a fault may involve one or more code errors. It should be possible to apply the framework described below for code errors to code faults; however, the numerical values of pertinent parameters would change.

The first step in formulating the mathematical approach for this research program is to clearly delineate the form of the desired results. Probabilistic or reliability characteristics that can be used to describe the successful or unsuccessful operation of a code are described in Table 3.1. These characteristics depend on the generic type of code. Codes are categorized according to whether they have discrete tasks or continuous tasks. As seen in the table, the operational mode distinction means that for analysis codes the desired code probability characteristics are "per run or execution" oriented, i.e., an estimate of the probability of failure is desired for a given run or set of runs as would occur in a safety assessment; while for data acquisition codes, the desired probability characteristics are "execution time oriented", i.e., an estimate of the probability of failure is desired for a given length of time for a specified demand.

Perhaps the single, most important code property is its reliability, i.e., the probability that the code executes without failure on a given run (discrete task code) or over a specified time period (continuous task code). There does not appear to be a uniquely useful means to estimate the reliability of a discrete task code based on the reliability of a continuous task code or vice versa. The importance of these differences in code reliability characteristics takes two forms:

- The mathematical models used in estimating the desired characteristics and/or the techniques used in their implementation can be expected to be somewhat different depending on the type of code.
- (2) The required (as well as available) failure data needed to verify these approaches are different.

Thus both the requirements, namely the mathematical framework and the supporting failure data, as well as the form of desired results, namely the predicted software failure probability characteristics, depend upon the type of computer code. It will be seen that all the published work to date on providing probability or reliability models has focused on one type of code, the continuous-running type.

A second quantity of considerable interest and one that is independent, from a mathematical standpoint, of the type of code is the expected number of code errors per line of code or more generally, per machine language instruction. While this quantity does not provide a direct indication of code reliability, it is a useful figure of merit by which to gauge the developmental progress of a computer code. In addition, it appears that estimates of this quantity would be similar for continuous and discrete task codes at similar stages of development.

As an aid to developing the correlations between probability-of-failure characteristics and code properties, it is desirable to classify errors according to error type using one or more classification systems. Table 3.2 shows one method of categorizing errors, failures, and code properties. In this table code execution failures are categorized according to their consequences and their software error causal mechanisms; the controlling code properties and software development characteristics that are assumed to affect the failure frequency are categorized as shown. The detail in the table is presented for completeness--it will be shown in this report that actual failure data are not detailed enough to support estimates of failure probabilities due to a specific faulty code unit, for example. However, by coarsely segregating the failure data according to potential consequence and probable cause, probability-of-failure characteristics may be categorized. For example, assuming sufficient failure data can be obtained, the probability of either a random (e.g., keypunch) or logical (e.g., incorrect implementation of an equation) error causing an execution failure of a specified type may be obtained. Although the current paucity of data precludes anything but gross estimates of such probabilities, even these can be used to make bounding estimates relevant to nuclear safety.

In general, two or three classification systems should probably be used simultaneously. In setting up error classification systems, the following general rules should be followed. First, error types should be mutually exclusive within a single classification system. Second, the number of errors of one type should be independent of the number of errors of other types withir the same classification system. Third, ground rules should be clearly stated regarding how to count errors. Finally, when more than one classification system is in use, each system should result in the same total number of errors being counted.

Although, in general, it may be desirable to set up classifications that are not programming language specific, the following example illustrates how a single classification system for a program written in FORTRAN might be set up. Error types are defined according to whether they occur in

- 1. Input Data,
- 2. Arithmetic or Logical Assignment Statements,
- 3. Input or Output Statements,

- 4. Data Initialization Statements,
- 5. Specification Statements,
- 6. Subprogram Statements,
- 7. Control Statements.

A specific FORTRAN manual would be identified to resolve questions as to what FORTRAN statements fall into each of these types. In addition, one might stipulate that statements that are out of order or misnumbered would be counted as control statement errors and that more than one error in a single statement would be counted as a single error. The question as to whether the number of errors of one type is independent of the number of errors of other types with this classification system is not obvious. One may simply have to assume independence with the understanding that whenever possible, statistical tests will be performed to test the assumption.

Tabulating the controlling variables associated with each code and its failure history must be done to obtain the dependencies among the probability-offailure characteristics and the code properties. A statistically meaningful number of codes must be investigated so that these characteristics can be correlated. Obtaining these failure data is considered the most difficult part of this program and is discussed in Section 4. Given that the appropriate failure data can be obtained and probability characteristics calculated, then regression analyses can be performed to identify the most important properties and controlling variables in software development. This knowledge could be used as a basis for defining development or quality assurance criteria for codes used or proposed for the licensing arena.

To summarize the above, the desired results include:

- Quantitative estimates of the key probability or reliability characteristics describing the successful or unsuccessful operation of a code. Table 3.1 illustrates some of these characteristics.
- (2) Quantitative estimates of the dependence of these characteristics on the type of failure, on the type and generic properties of the code, and on the controlling variables in the development of the code so that probabilistic predictions in terms of these parameters may be made. Table 3.2 illustrates several classifications of failure, code, and development parameters that may be used to facilitate correlation with the probability characteristics to seek the functional dependencies.

The quantitative requirements include not only the mathematical models but the associated data base with which to accomplish these. The modeling is described in detail in Section 3.2. Failure data are discussed in Section 4.

3.2 Mathematical Modeling

The modeling to be described here is designed to fill out the analytic framework discussed in the previous section. This framework is designed to encompass predictions for the software probability or reliability characteristics for both discrete task codes and continuous task codes. The probability characteristics of interest in discrete task codes are the probability of failure parameters shown in Table 3.1 for a given run or set of runs; the analogous parameters of interest for continuous task codes are for a given duration of time for a specific application. An example of a parameter of interest in the former case is the probability that a run or set of runs using the RELAP code in a reactor safety analysis successfully (within the constraints of the modeling) predicts peak fuel temperatures for a postulated loss-of-coolant scenario. An example of a parameter of interest in the latter case is the probability that an on-line code successfully interprets sensor data and outputs proper analytically-derived temperatures, pressures, or other state parameters to a reactor operator during an emergency situation. In this latter case, clearly the time that the emergency situation required that the on-line software operate successfully would affect the probability of successful operation. As the TMI situation graphically illustrated, this time scale could be for many hours.

These examples imply that the required mathematical models and the associated failure data depend somewhat upon the type of code. However, once the appropriate probability characteristics have been obtained for each generic type of code, the analyses to be used for obtaining the dependencies on non-probabilistic code parameters such as those illustrated in Table 3.2 are the same.

The methematical models currently used in software reliability analysis are based on failure rate models such as those used in hardware reliability applications. Given that an error of type k in classification j has not occurred pricr to time t, the probability that such an error occurs between t and t + dt is expressed as

$$\lambda_{ik}(t)dt$$

where λ_{jk} is the detection rate for errors of type k in classification j and is, in general, a function of time. The probability that an error occurs between time t and t + dt, given no error prior to time t is expressed as

(1)

(2)

 $\lambda(t)dt$

where

$$\lambda(t) = \sum_{k} \lambda_{jk}(t).$$
(3)

Here, $\lambda(t)$ is the detection rate for errors of all types. (The subscript j is not required after summation over all error types k since all classification systems are postulated to count the same total number of errors.) From the definition of $\lambda(t)$ a simple derivation shown in Appendix A yields the reliability

$$\kappa(t) = \exp\left\{-\int_0^t dt'\lambda(t')\right\}.$$

Work to date has focused almost entirely on estimating $\lambda(t)$ for continuous task codes. Attempts to break $\lambda(t)$ into components of various types as in (3) are not apparent. Several models have been suggested to express λ as a function of parameters such as the number of errors present in the code, the number of errors discovered, and the probability that an error is detected given that it is present. These models are described in Appendix A. In most of the models, the detection rate is not regarded as a function of time; however, in the Littlewood-Verrall model, the parameter in the exponential distribution is regarded as a random variable having a gamma distribution. This leads to a time-dependent detection rate in (4).

A major fraction of the computer codes of interest in this research program are discrete task codes. So long as errors occur infrequently, it should be possible to apply failure rate models to these codes provided the time is the accumulated CPU time for all runs, and provided the codes are run on identical computers with identical compilers. However, detection rates obtained for a code operation on one computer are not necessarily the same as detection rates for operation on another computer. This is especially important in the case of nuclear safety analysis codes which are often developed on a single computing system but are then exported to different computing systems throughout the country. In addition, if errors are detected too frequently, as might be the case for a code undergoing a new phase of testing, the run time assigned to an individual task might impose a structure on the error detection time which has little to do with the actual error occurrence rate. For these reasons, it is necessary to develop more comprehensive modeling which is more naturally applied to discrete task codes and yet can also be applied to continuous task codes.

The modeling which will now be described can be used to meet these dual applications. Let X_{jk} be a random variable defined as the number of errors of type k in classification system j detected as the result of a given run. The expected value of X_{jk} will be denoted by Λ_{jk} . The probability distribution for X_{jk} is assumed to be Poisson, i.e.

$$P(X_{jk} = x) = \frac{\Lambda_{jk}}{x!} e^{-\Lambda_{jk}}$$

where x is a non-negative integer. If the values for X_{jk} are independent for each type k and if the error types are mutually exclusive, then it is easy to show that the probability distribution for the total number of errors X, defined as

$$X = \sum_{k} X_{jk},$$
(6)

is also Poisson, i.e.

16

(4)

(5)

$$P(X = x) = \frac{\Lambda^{x}}{x!} e^{-\Lambda} ,$$

where

$$= \sum_{k}^{N} jk$$

With this model, the reliability is

$$R = P(X = 0) = e^{-\Lambda} .$$
 (9)

The use of the Poisson distribution has certain mathematical advantages. First, the Poisson distribution is often used to approximate the binomial distribution. There may be instances where it is useful to model the detection of errors of type k in terms of M_{jk} , the number of code units (lines of code, instructions, or some other unit) capable of producing type-k errors, and the probability q_{jk} that an error of type k is detected in a given unit during a given run. If q_{jk} is the same for each unit and if error detection in each unit is independent of error detection in other units, then the probability distribution for X_{jk} will be binomial with parameters M_{jk} and q_{jk} . In general, M_{jk} is likely to be large and q_{jk} small. The binomial distribution can then be approximated by the Poisson distribution in (5) with

 $\Lambda_{ik} = M_{ik} q_{ik}$ (10)

In (10) failure data may be used to estimate Λ_{jk} or q_{jk} directly; some other source such as expert opinion or the human error literature, using analogous error probabilities to estimate q_{jk} , may also be used. Of course, the value of M_{jk} is provided from the code properties or specifications.

A second advantage arises for codes that are conveniently described in terms of error detection rates $\lambda_{jk}(t)$ as in (1). For runs of duration t, the probability distribution for X_{jk} is easily seen to be given by the Poisson distribution in (5) with

$$\Lambda_{jk} = \int_0^t dt' \lambda_{jk}(t')$$

The reliability estimates (4) and (9) become identical in this case.

Finally, for a given error type and classification system, only one parameter needs to be estimated for the Poisson distribution in (5). The accumulation of appropriate failure data is n eded to verify whether values of Λ_{jk} obtained for a code operating on one computing system can be applied to the same code when it operates on a different computing system.

This discussion of reliability modeling will conclude with a dicussion of a few potentially useful extensions to the 'oisson model described above. Reference will be made only to the total number of errors; however, there does not appear

(11)

(7)

(8)

to be any a priori reason why the same developments could not be applied to individual error types.

Table 3.1 suggests that probability distributions for important parameters describing probabilistic code characteristics should be estimated if possible. In the case of the Poisson model, the important characteristic is the expected number of errors detected in a given run or equivalently the probability q_{jk} of error detection. Let $f(\Lambda)$ represent the probability density of the expected number of errors. The probability distribution of X, the number of errors occurring on a given run, taking into account the uncertainty in the value of Λ expressed by $f(\Lambda)$ can be computed as

$$P(X = x) = \int_{0}^{\infty} d\Lambda \frac{\Lambda^{X} e^{-\Lambda}}{x!} f(\Lambda)$$
(12)

where (12) becomes the replacement for (7). Generally $f(\Lambda)$ would be chosen from a parametric family of functions and (12) should be regarded as the conditional probability that X = x given the appropriate parameter values. The dependence of P(X = x) on such parameters should be understood but will not be explicitly noted.

It may or may not be possible to evaluate the integral in (12) analytically depending on the precise form of $f(\Lambda)$. In any event the integral can be evaluated numerically, perhaps using Monte Carlo techniques. Further, the use of Monte Carlo would allow the sampling of Λ from actual data without the need for determining an analytic form for $f(\Lambda)$ (or equivalently f(q)).

If an analytic form is desired, a particularly convenient choice for the function $f(\Lambda)$ would be to select it from the family of gamma distributions, i.e.

$$f(\Lambda) = \frac{\alpha^{\beta}}{\Gamma(\beta)} \Lambda^{\beta-1} e^{-\alpha\Lambda} .$$
 (13)

where $\Gamma(\beta)$ is the gamma function. With this choice, evaluation of the integral in (12) yields

$$P(X = x) = \frac{\alpha^{\beta}}{(\alpha + 1)^{\beta + x}} \frac{\Gamma(x + \beta)}{x! \Gamma(\beta)} , \qquad (14)$$

and the reliability becomes

$$R = \left(\frac{\alpha}{\alpha + 1}\right)^{\beta}.$$
 (15)

The expected number of errors detected in a given run is found to be

$$E(X) = \frac{\beta}{\alpha}$$
(16)

and the variance of the number of errors detected is

$$Var(X) = \frac{\beta}{\alpha} \left(1 + \frac{1}{\alpha}\right) . \tag{17}$$

Section 1 states that one of the objectives of this program is to permit a priori estimates of a computer code's reliability given the code's general properties and its intended application. This implies that it will be possible to extrapolate data for codes of various types and for a variety of applications to a specific code and code application for which error detection data are not available. As one begins to apply the code, one may modify the reliability estimates based on experience in using the code. If the required data extrapolation is done in a way that provides estimates of the parameters α and β in (13), then the gamma distribution in (18) can be regarded as a prior distribution for a Bayesian analysis. If after running the code r times, one observes s errors, then a strightforward application of Bayes theorem leads to the following posterior distribution for Λ :

$$f(\Lambda|s,r) = \frac{(\alpha + r)^{\beta+s}}{\Gamma(\beta + s)} \quad \Lambda^{\beta+s-1} = e^{-(\alpha+r)\Lambda} \quad . \tag{18}$$

It then follows that updated estimates for the probability distribution of X, for the reliability, for the expected number of errors detected, and for the variance of the number of errors detected are given respectively by (14), (15), (16), and (17) with α replaced by α +r and β replaced by β +s.

The discussion up to this point has been concerned with methods to estimate the reliability of a computer code. However, as noted in Section 3.1, the expected number of code errors per instruction or per line of code is also a useful figure of merit for which estimates are desirable. Many of the failure rate models described in Appendix A provide estimates of this quantity as a result of attempting to model the way in which error detection and correction modifies the error detection rate. It is possible to devise analogous models to describe the influence of error detection and correction on the expected number of errors. A by-product of such models might be estimates of the expected number of code errors per instruction. Since such models have not been investigated, the following approach was developed:

An examination of failure data such as that of Musa as listed in Appendix B suggests a general picture in which, over a specified testing phase, the error discovery rate accelerates early in the testing phase, increases at an approximately linear rate during the central part of the testing phase, and then decelerates during the later stages. Qualitatively, such behavior might be anticipated. Early in the testing phase, the more obvious errors may result in relatively short code runs and exposure of relatively small portions of the code. As these errors are eliminated, the runs are likely to increase in length and a larger fraction of the code will exercised. Late in the testing phase, only the more subtle errors remain and the rate of error detection decreases. A convenient measure of progress through the testing period appears to be the time in working days. To model this behavior, a relatively simple process may be envisioned. Let N be the total number of errors present in the code and n(t) be the expected number of errors detected in t working days. (Introduction of new errors during error correction is assumed to be negligible.) If one defines

$$x = \frac{n - \frac{N}{2}}{\frac{N}{2}}, \qquad (19)$$

then during the early part of the testing phase, x is negative; during the central part, x is nearly zero; and during the later part x approaches unity. Now let t_{o} be the time when x = 0 and define

 $y = k(t - t_0) \tag{20}$

where k is a constant to be specified later. The error detection rate will be proportional to dx/dy. A relatively straightfoward way to model the qualitative behavior of the error detection rate, as described in the preceding paragraph, is to let

$$\frac{dx}{dy} = 1 - x^2$$
 (21)

This equation clearly predicts an accelerating error detection rate early in the testing period and a decelerating rate late in the period.

If (21) is solved subject to the initial condition x = 0 when y = 0, one finds

x = tanh y.

Substituting from (19) and (20), the following expression for n(t) results.

$$n(t) = \frac{N}{2} [1 + tanh k(t - t_0)], \qquad (22)$$

A least squares fit of (22) to failure data then provides estimates of the parameters N, k, and t_0 .

Equation (21) assumes that the error detection rate is an even function of t - t. This assumption can be relaxed, e.g. by subtracting a term proportional to x on the right side of (21). The solution of the resulting equation leads to a result similar to (22), namely

 $n(t) = a + b \tanh k(t - t_0),$

but there are now four constants to be determined rather than three. Further analysis is required to determine whether the additional flexibility provided by a fourth constant justifies the additional computational effort required to determine four constants instead of three. The estimate of N, the total number of errors present in the code, as obtained through the use of (22) provides a means to estimate the number of errors in the code per line of instruction at any stage during the test phase. If this quantity is designated as n, one finds

$$r_1 = \frac{N - n}{I}$$
(23)

where I is the number of lines of instructions.

In summary, modeling has been developed to calculate reliabilities and expected numbers of errors detected for both continuous task and discrete task codes. In addition, a means of estimating the expected number of errors per line of instruction has been developed. The implementation of these models to produce probability of code failure estimates useful in nuclear safety is illustrated in the next section.

3.3 Implementation

The implementation of the Poisson model described in Section 3.2 requires a procedure for estimating the expected number of errors per run* (discrete task codes) or during a specified period of operating time (continuous task codes). If one is interested only in the reliability, classification of errors detected is not essential; however, classifying errors promotes precision in the counting of errors. Further, the consequences of code failure as categorized, for example, in Table 3.2, obviously depend on the type of causal software error mechanism.

Data requirements are apparent from the discussion of the model. Ideally, one would like to have records of appropriately classified errors detected as a function of the number of the run (or time period in the case of continuous task codes) in which they were found. Data in this form would permit the direct investigation of the dependence of the expected number of errors on run number and error detection and correction. Such data should be recorded for codes following their release for general use; however, they will probably be more readily obtainable during the testing period just prior to release. It should be possible to make reasonable estimates of code reliability in the post-release period using data collected just prior to release, but, whenever possible, these estimates should be checked against post release data. Obtaining generic correlations between pre- and post-release data has been identified in this report as a means of expanding the usefulness of both data bases.

Data in the form just described have not been found for discrete task codes. Generally records of error corrections are recorded as a function of the date when the corrections were made. Sources of these data are discussed in Section 4. These data can be used to estimate the expected number of errors per run provided reasonable estimates can be made for the total number of runs involved in producing the data; however, meaningful analysis of the changes in the expected number of errors as errors are detected and corrected will be more difficult.

Another possible approach to the estimation of the expected number of errors per run is based on the observation that code errors are human errors. To use this approach, classification of errors would be essential. An estimate of the number of code units M_{jk} (see Eq. (10) in Section 3.2) capable of producing errors of a given type would be made. Then the human steps required to produce these units would be analyzed and human factor data would be used to estimate the probability that these steps were carried out incorrectly. Finally, the code structure would be examined to estimate the probability that one of the probability that the code unit is produce of this probability and the probability that the code unit is produced incorrectly would then provide an estimate of the probability q_{ik} in Eq. (10) in Section 3.2.

*Since the expected number of errors per run is given by $\Lambda = qM$, where the notation in the previous section is used and M is specified, knowledge of Λ implies knowledge of q and vice-versa. Thus, the discussion relating estimation of Λ applies equally well to estimation of q.

Still another means of estimating the expected number of errors per run is through consultation with experts. For example, assume that for a certain set of conditions the probability that a code would incorrectly compute an important safety parameter is estimated to lie between 1/20 and 1/100 with 90% certainty. Further assume that this probability is estimated to be equally likely to be greater than 1/20 or less than 1/100. One way to use this information is to fix the parameters in a probability distribution for A, the expected number of errors per run. If the gamma distribution in Eq. (13) of Section 3.2 is used for convenience of illustration, $\alpha = 165$ and $\beta = 4.48$ and using Eq. (14), the probability of failure is estimated to be

P(X > 1) = 0.03 + 0.01,

where 0.01 represents one standard deviation. Although some type of combined subjective/statistical procedure may be necessary in the absence of data it is clearly preferable to estimate the expected number of errors Λ or the probability distribution for Λ directly from data.

Data requirements for estimating the number of errors per machine language instruction in a code are similar to those described above for the Poisson model. However, it is not necessary to have error detection records as a function of run number or to even know the number of runs. Records in terms of calendar days are adequate. For discrete task codes, the sigmoidal fault discovery model (Eq. (22) in Section 3.2) can be used to estimate the number of errors present or remaining in a code in terms of the number of errors detected. For continuous task codes, the sigmoidal model and the failure rate models of Appendix A appear to work about equally well. However, both the failure rate models and the sigmoidal model are shown in Section 5 to have a tendency to predict a total number of errors present in a code which is only slightly larger than the number of errors actually detected. Thus, if any of these models are used during code testing, the predicted number of errors remaining in the code cannot be used as an acceptability criterion for when code testing can be stopped. Some other quantity, such as the mean number of days between error detections, must be used instead.

Once estimates of the expected number of errors or the number of errors per machine language instruction have been made for several codes, correlation of these values with other code characteristics can be attempted. Some of the characteristics of potential interest in this correlation include code size, complexity, testing history, and other properties shown in Table 3.2. The nature of the relationships of the probabilistic code parameters to these characteristics can be explored using the multivariate regression analysis capabilities found in many standard statistical analysis code packages. Some examples of such analyses using the BMDP system described in Appendix C are shown in Section 5.

In summary, approaches or means to obtaining the relevant parameters in the aralytic framework have been described. Sample calculations using the failure rate models and the sigmoidal model to predict error content and the probability of a line of code being in error are shown in Section 5. Calculations using the Poisson model will be performed using estimates of q_{jk} or Λ_{jk} obtained from these models as well as the other sources described above.

Generic Type of Code	Applications	Key Probability and Reliability Characteristics ^{1,2}	Comments
Discrete task codes	Reactor design	Probability that a specified unit of code is in error ² .	Key reliability characteristics are
(Analysis codes)	Safety assessment	Probability of an output error of a specified type in a given run.	"per execution" oriented.
		Probability of success (failure) in obtaining a key result in a given run(s).	Appropriate failure data include a history of failures vs. total runs.
		Probability that a specified unit of code produces an execution failure in a given run.	
		Probability distributions or at least confidence or uncertainty bands for the above character- istics.	
Continuous task codes	Feedback to operator	Probability that a specified unit of code is in error ² .	Key reliability characteristics
(Data acquisition	Input to simulator	Probability of success or	depend heavily
codes)	Operational control	failure over a specified	Appropriate failure
	Emergency system activation	unreliability) during a specified demand.	data include a history of time to
		Failure rate.	(between) failures.
		Probability distributions or at least confidence or uncertainty bands for the above characteristics.	

Table 3.1 Key Software Probability and Reliability Characteristics

^TAll probability and reliability characteristics may be categorized and normalized according to the type of failure they measure, the failure causes, and consequences of failure.

²Probabilities are defined both for actual code operation failures and for code component errors that have the potential to cause operation failures.

Operational Failures		Causal Mechanism		Controlling Properties and Developmental Variables				
Type of Consequence	Faulty Code Unit	Faulty Modeling Unit	Type of Cause					
 No output (i.e. a crash) Absurd output (detectable) Misleading output (may propagate failures in nuclear plant design or operation) 	 Symbol Operand Constant Variable Line (statement) Storage byte or array (e.g. common block) Logic block (subroutine) Logical directive (e.g. IF statement) 	 Constant Variable Equation Model (set of equations) Table (e.g. property values) 	<pre>In-Code² Random (e.g. key punch) Logical or decisional (incorrect or poor programming to implement modeling) Ex-Code Documentation leading to misuse of code</pre>	Testing QualityProgramming1. Quality1. Size assurance2. Complexity methodsassurance2. Complexity and use2. Size of QA 4. Structure effortand method3. Capability for bench- marking or analytic verificationof coding for book of coding000				

Table 3.2 Categories¹ of Operational Failures, Causal Mechanisms, and Controlling Properties and Development Variables in Software Reliability

¹This table illustrates a plausible way of categorizing failures, their causes, and dominant variables. Other ways could be chosen.

²In-code causes could also be subdivided as follows: 1) "Pure" programming causes such as syntax errors or incorrect transfers of program control, logic, or data; 2) Program modeling causes such as incorrect algorithm approximations or improper treatment of singular points or critical parameters leading to overflow and roundoff errors.

4.0 Software Failure Data

This section describes the failure data accumulated to date and how they and future data have been or will be used to support the goals of this program. Sections 4.1 and 4.2 describe the non-nuclear and nuclear software failure data, respectively, that have been or are expected to be obtained in this program. Section 4.3 discusses conclusi ns on what should be done to satisfy program goals.

4.1 Non-nuclear Data

The collection of the non-nuclear data has been made for two reasons:

- (1) To initiate mathematical modeling and investigation as early as possible. The available non-nuclear data has been in a much more readily useable form than the nuclear data. Tabulations of failure histories are available that could be directly used in investigating or formulating mathematical models. On the other hand, the nuclear data have been "buried" in logged records that require evaluation and conversion to a form adaptable to manipulation with mathematical models. Thus, the detailed investigation of these records has been deferred. However, it is a necessary component of this program.
- (2) To form as wide a data base as possible. Although the applicability of non-nuclear software failure data to nuclear software remains to be shown, comparisons can be made. If the data are directly applicable the statistical data base for predicting software probability of failure characteristics is obviously enhanced. If the data are not applicable, comparing the data will be useful in evaluating the software development and application characteristics of nuclear codes to determine why nuclear and non-nuclear failure data are different.

Non-nuclear software failure data have been and are being compiled from the open literature, Rome Air Development Center (RADC) and private companies. RADC, at Griffiss Air Force Base, maintains an extensive literature and computer library data-base in software reliability based on software failure data from military, aerospace and communication industries. Most of the failure history data come from data acquisition systems rather than batch mode systems.

Failure history data are generally available in either of two forms: (1) individual failure tabulations showing successive execution times between failures or (2) blocked data in the form (n,t), n errors in time t where t may be in any unit of time including execution time. Although the former failure data form may be reduced to the form (n,t), the converse is not true. Most of the data to be presented in this section were obtained during system testing and integration and represent both faults, defined [4.1] as software defects that cause an operational failure, and failures, defined as any unacceptable departure of program operation from program requirements. In some cases, the failure data are referred to as "errors" or "changes" which are ill-defined but presumably represent faults or failures as just defined. Although the intent of this program is to predict probability of failure characteristics

for codes in production status, not test status, it is assumed that extrapolation to production code failure probabilities can be made. Further, these data allow us to explore the mathematical approach of Section 3 while more relevant data are being acquired.

Compilations of failure data [4.2] in the first form from 16 systems ranging in their applications from military to real time command and control systems were obtained from Rome Air Development Center (RADC). These data are illustrated in Table 4.1 and comprise Appendix B. They are given in terms of successive execution times between failures in seconds. Descriptions of the systems from which the data were obtained are shown in Table 4.2. Clearly, these data represent a diversity of systems and system characteristics. Accordingly, as described elsewhere in this report, these data were used to investigate different models that predict probability of failure characteristics and to perform regression analyses to illustrate use of the statistical analysis systems.

Several software project studies, representing sets of failure data in either the above form or the (n,t) form, were obtained from the literature and studied to gain an insight into both qualitative and quantitative characteristics of software development and reliability. Data from these studies and a brief summary of each are as follows:

- (1) The data shown in Table 4.3, taken from Brooks and Motley [4.3], were taken from projects involved in the development of a large scale command and control software system. These data represent a variety of system sizes; higher order programming languages JOVIAL and CENTRAN were used in addition to the assembly language ALC. The parameters N and g represent estimates of the total number of errors in the system at the initiation of the project and the probability that any one of these errors would be detected during a specified portion of the testing period. As described in the table, the lack of a quantitative specification for this test period implies that the estimate for q has no quantifiable significance. Another highlight of these data is illustrated in Projects 4 through 7, in which different error accounting methods were used with grossly different error probability characteristics estimated. The implication of course is that only crude error probabilities can be inferred from published failure data unless the definitions of errors, faults, failures and so forth are very clearly made.
- (2) The data shown in Table 4.4, taken from Goel and Okumoto [4.4], were obtained from a real-time control system for a land-based radar system developed by Raytheon Co. Nearly all of the modules are written in JOVIAL/J3. One observation that can be made from these data is that the ratio of errors introduced in fixing the code to the total number of errors is very small ($\sim 2\%$). If data from other codes support this observation, reintroduction of errors does not constitute a significant contribution to code failure probability. Of course, this may not hold true for other codes. For large scale safety codes that routinely transfer large amounts of information among modules, correction of a code defect in one module may lead to the discovery of defects within modules with which it interfaces.

- (3) The data shown in Table 4.5, taken from Goel [4.5], are from the real-time, multicomputer complex which forms the core of the Naval Tactical Data System (NIDS). The programming language(s) used is not specified. These data show the scatter in observed times to failure that is evident in much of the failure data in the literature. As observed in the last test phase, they also show that a new testing phase is likely to result in an enhanced detection rate even in a release version of a code.
- (4) The data shown in Table 4.6, investigated by Shooman [4.6] but obtained by Dirkson, Hesse and Kientz [4.7], are from three operating systems subsequent to release and from four application systems during the system integration. These data yield estimates of the number of code defects corrected per line of instruction for the given programs. They imply that the given systems averaged roughly 1 (observable) defect per 100 instructions (as estimated using the Shooman model discussed in Appendix A) prior to the initiation of data collection. These data also show that this defect percentage is significantly reduced by the end of the observation period. However, the scatter in the data indicate that quantification of this reduction will have large uncertainties.
- (5) The data shown in Table 4.7, taken from Akiyama [4.8], are from a system whose function is unspecified and written in FASP, the assembler language for FACOM 230-60. Data were available from all phases of software development from module testing to field use. These data represent a base upon which to explore relationships among error detection and the indicated code properties. Obviously nothing meaningful can be obtained with just these data; however, this table does represent an actual example of the type of data needed to draw correlations among probability of failure characteristics (here measured by number of errors detected) and code properties.

The software from which the above failure data were obtained were not written in FORTRAN, the principal programming language used in nuclear codes. Further, as mentioned above, the functions and usage of these software systems are sufficiently different that the applicability of the above data to nuclear systems needs to be questioned. However, they provide not only an insight into qualitative and quantitative characteristics of software systems but a part of the non-nuclear data base for comparison with future data from nuclear systems as well as a data base for testing the mathematical approach of Section 3.

4.2 Nuclear Data

This section discusses specific sources that will be tapped to provide a nuclear software failure data base. As related above, efforts in obtaining and appropriately categorizing nuclear software failure data have been deferred. Such data have only been found in logged records and are often in a form associating dates or time durations with code changes that may or may not have been made to correct a detected code bug which in turn may or may not have been revealed by an operational failure of the code. Nevertheless, the investigation and interpretation of such data is needed to provide a nuclear data base for this program.

One promising nuclear failure data base is that of the ARC (Argonne Reactor Computation) System maintained by the Applied Physics Division at ANL. Roughly 125 codes ranging in size from 2,000 to 50,000 FORTRAN statements comprise this data base which represents both pre-production and production codes. Production as used here refers to codes that are in unlimited use within as well as outside ANL. Pre-production refers to codes in varying stages of testing but prior to release to this unlimited user status. The ARC System was initiated in the mid 1960's and has been in continuous development since that time. Production use of the codes in the system has been under way since about 1970. Although the format of this base is as indicated in the preceding paragraph, implying that considerable time and effort will be required to properly evaluate it, the information gained from this evaluation as well as the data obtained to support the regression analyses of Section 3 justify these expeditures.

Actually the Applied Physics Division will provide data yielding two types of information. The first type is given by the data generated to support the ARC System requirements, namely that all changes to codes within the system and the reasons for these changes, be documented on a standard form. The second type is given by data from individual code developers who maintain their own records, generally in a more detailed form. These data have been generated for the more recent codes, some of which are not yet part of the ARC System, which have been written to standards set by the Committee on Computer Code Coordination (CCCC) established by DOE. By being more specific as to type of bug in some cases, this second type will facilitate categorizing; however, since all codes written by CCCC standards were not documented in this manner, this type comprises less of a code data base than the first type of information.

Two examples of codes for which these types of information have been obtained (but not yet investigated) are the GNIP4C and SYN3D codes. The first is a 20,000 word input processor used to arrange geometry and nuclear data for other codes within the ARC System. The data for this code is considered preproduction data since the GNIP4C code is not available throughout the industry. The second code is a 10,000 card, three dimensional, flux-synthesis, diffusion theory code used in eigenvalue calculations for criticals analyses and burnup calculations. Data for the SYN3D code include both pre-production and post release data.

Actual logging of errors has occurred throughout the nuclear industry. For example, extensive software error records have been kept at Los Alamos National Laboratory and EG&G on their large safety codes for many years. These records take two forms: 1) written records containing information relating the nature
of bugs found and how and when they were corrected; 2) computer update files containing when and how changes due to software errors and other modifications and additions were implemented. For at least one of these codes, the latter is available on microfiche. These data will be explored to facilitate the categorization of both errors and failures.

Although it is planned to survey the information available throughout the industry, early detailed data evaluation and investigation will probably be limited to ANL and LANL sources to determine the needs to be filled by additional investigations. The data will be investigated for all possible nuclear applications to explore interdisciplinary differences, if any exist. For example, it is likely though not proven, that deterministic physics codes are more "reliable" than safety codes because of their greater ease of benchmarking. Structural analysis codes provide another nuclear application and should be similarly segregated. In summary, extrapolations of results from one type of code to another must be supported by actual failure data.

4.3 Conclusions on Data Needs and Future Acquisition Efforts

The above sections lead us to draw several conclusions relating to data needs and efforts to acquire these data:

- All data obtained to date comprise only a small part of the information base needed to achieve the goals of this program.
- (2) The acquisition and appropriate categorization of nuclear software failure data will be a time-consuming but necessary component of this program. Categorizing the bugs according to the types of code defect consequence and cause categories is required as discussed in Section 3. To accomplish this will undoubtedly require some subjective categorization. Extended categorizations come to mind: for example, an important concern for nuclear safety is with what probability could an operational failure of a code or a documented code bug result in a misleading code result upon which an unsafe design decision or incorrect safety conclusion is made. No data to estimate this propagation probability have been found or indeed can be assumed to exist. Thus, expert opinions would have to be solicited and factored in to assign such a probability.
- (3) The data obtained have been and will be largely from testing phases of codes. Post-release or production status data will be sought. The aforementioned nuclear data sources contain both preproduction and production status data as discussed. Clearly the goals of this program relate to production codes, not test versions. Drawing comparisons between the failure probabilities of pre- and postrelease codes must be done to obtain the benefits of a large prerelease data base. However, the mathematical approach to treating the data is the same.
- (4) It would be extremely fruitful to establish a computerized data bank and information center at the National Energy Software Center at ANL, to properly organize the failure data, integrate it with the mathematical and statistical analysis systems described herein, and to facilitate industry-wide participation (as well as access) in data collection. This is discussed further in Section 6.

Fault Number	Execution Time	Day of Failure	Fault Number	Execution Time	Day of Failure
1	3	1	70	379.	64
2	30.	2	71 -	44.	64
3	113.	9	72	129,	64
4	81,	10	73	810,	64
5	115,	11	74	290,	64
6	9,	11	75	300,	64
7	2,	17	76	529,	65
8	91,	20	77	281,	65
9	112,	20	78	160,	65
10	15,	20	79	828,	66
11	138,	20	80	1011,	66
12	50,	20	81	440,	66
13	11.	20	62	1765	67
14	100	20	84	1064	67
15	100,	20	85	1783.	68
17	670	30	86	860.	68
18	120	30	87	983,	68
19	26.	30	88	707.	69
20	114.	30	89	33,	69
21	325.	30	90	868,	69
22	55,	30	91	724,	69
23	242,	31	92	2323,	70
24	68,	31	90	2930,	71
25	422,	31	94	1461,	72
26	180,	32	95	843,	12
27	10,	32	96	12,	12
28	1146,	33	97	261,	72
29	600,	34	98	1800,	73
30	15,	42	100	1435	74
31	36,	42	101	30	74
32	4,	40	102	143.	74
33	0,	46	103	108,	74
35	227	46	104	0,	74
36	65.	46	105	3110,	75
37	176,	46	106	1247,	76
38	58,	46	107	943,	76
39	457,	47	108	700,	/6
40	300,	47	109	8/5,	77
41	97,	47	110	720	77
42	263,	47	112	1897	78
43	452.	53	113	447	79
44	255,	53	114	386	79
45	197,	54	115	446.	79
40	193,	54	116	122.	79
47	79	54	117	990,	79
40	816	56	118	948,	80
50	1351.	56	119	1082,	80
51	148.	56	120	22,	80
52	21.	57	121	75,	80
53	233.	57	122	482,	80
54	134,	57	123	5509,	81
55	357.	57	124	100,	81
56	193,	59	125	1071	61
57	236,	59	120	271	83
58	31,	59	129	790	83
59	369,	59	120	6150	83
60	748,	59	130	3321	83
61	0,	59	13.	1045.	84
62	232,	59	132	648.	84
63	330,	61	133	5485,	87
64	1222	62	134	1160,	87
66	543	63	135	1864,	88
67	10	63	136	4116,	92
68	16.	63			
69	529,	64			

Illustration of Failure Data: Successive Execution Times Between Failures in Seconds

Table 4.2

Characteristics of Software Systems Studied by Musa

System No.	Phase for Which ³ Data Available Total Instructions/ New or Modified Instructions		Programmers	Faults4,5,6	
1	S,0	21700/	9	136 (2)7
2	S,0	19500 27700/ 6600	5	54 (2)
3	s,0	23400/	6	38 (1)
4	s,0	33500/	7	53 (3)
51	S*	244500	275	831	
6	SS	5700	8	73	
14C	0*	-	110	36	
17	S	61900	8	38	
27	S	126100	8	41	
40	S	180000	8	101	
SS1A ²	0*			112	
SS1B ²	0*			375	
SS1C ²	0*			277	
SS2	0*			192	
SS3	0*			278	
SS4	0*	 Statistics 	-	196	

¹Design changes amounting to about 21% of the source lines of code were introduced after failure 288.

²SSIA, SSIB, and SSIC represent the same software system running in slightly different environments.

³Phases:

SS = Subsystem Test, module testing

S = System Test; after modules or subprograms are integrated into a larger system

O = Operational; System Running in the Operational Environment. Data is for a complete phase unless starred.

⁴A fault is a software defect that causes the failure.

⁵If a failure recurred before the fault that caused it was corrected, it was not counted.

⁶When a failure was spawned as the result of correcting an earlier failure, it was counted as a new failure.

⁷ Data are for the system testing phase unless in parentheses; data for the operational testing phase are shown in parentheses.

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Project	Number of Instructions	Language	Phase ¹	Errors Detected	N ³	q ³
1	1317K	ALC, CENTRAN	T & I	2657	3771	1.17x10 ⁻⁴
2	124K	JOVIAL, J3	T & I	1301	1438	1.27x10 ⁻³
3	80K 40K	ALC JOVIAL, J3B	T & I	1239	3094	2.07x10 ⁻³
42	114К	JOVIAL, J4	T & I, post-integ tion testi	1138 ra- ng	1348	.1166
52	1.1.1.1		-	1483	1824	.1060
62	전 같이 가지랑			2707	3958	.0739
72	그 소문 탄물	1.54.54.54	· · · ·	2362	3446	.0742

Characteristics of Software Systems Studied by Brooks and Motley

¹The test and integration phase refers to a period of testing during and after the integration of two or more programs into a single system. Data from Projects 4 through 7 encompassed this phase as well as an acceptance testing phase.

²Projects 4, 5, 6 and 7 constitute one set of error data counted four different ways and illustrate a need for standardized terminology. (Four attempts to remove different types of non-software related errors from the total set of SPRs (software problem reports) reported during testing by different people according to their own definition of a valid software error).

³N and q denote estimates, using the Brooks-Motley model discussed in Appendix A, of the total number of detectable code errors at the beginning of testing and the probability of any single error being detected during a unit of test effort. Since this unit of effort was not defined in the referenced literature, the physical significance of "q" and even the comparability of q estimates among projects in this table cannot be specified.

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Month	Errors Detected	Errors Caused by Imperfect Debugging
1	122	6
2	98	1
3	82	1
4	75	3
5	113	2
6	85	3
7	105	2
8	47	1
9	61	1
10	25	1
11	28	0
12	42	1
13	18	0
14	17	0
15	28	0
16	14	0
17	5	0
18	3	0
19	3	0
20	13	0
21	5	0
22	10	0
	TOTAL 999	22

Comparative Histories of Errors Introduced in Debugging and Total Errors Detected as Analyzed by Goel and Okumoto

Table 4.5

NTDS Error Data

Error Number	Time Between Errors in Days
Checkout Phase	
checkout rhase	
1	9
2	12
3	11
4	4
5	7
6	2
7	5
8	8
9	5
10	7
11	
12	6
13	
14	9
15	4
16	
17	
18	
19	0
20	이 같은 것 같이 안 한 것 같아. 한 것 같아요. 이 것 같아.
21	11
22	33
23	01
24	31
25	4
26	삼산 공사는 동안을 하면 것 같은 것을 잘 넣었다. 것 같은 것
Test Phase	
27	87
28	47
29	12
30	9
31	135
User Dhase	
User Phase	258
52	200
Test Phase	이번 것 같은 것은 아파 가지 않았는 것 가격을 잡다.
33	16
34	35

Table 4.6

Change Data and Code Defect Estimates for Large Scale Programs Studied by Dickson, Hesse, Kientz and Shooman

Application A 240,000 Inst.		Application B 240,000 Inst.		Application C 240,000 Inst.		Application D 240,000 Inst.		
Month	Changes	Changes/Inst.	Changes	Changes/Inst.	Changes	Changes/Inst.	Changes	Changes/Inst.
1 2 3 4 5 6 7 8 9 10 11 12 13	514 926 754 662 308 108	2.15 x 10 ⁻³ 3.85 3.15 2.76 1.28 0.45	905 376 362 192 70	3.76 x 10 ⁻³ 1.57 1.51 0.80 0.29	235 398 297 506 174 55 60	0.98 x 10 ⁻³ 1.66 1.24 2.11 .72 .23 .25	331 396 269 296 314 183 158 368 337 249 166 108 31	1.38 x 10 ⁻³ 1.66 1.12 1.24 1.31 0.76 0.66 1.54 1.41 1.02 0.69 0.45 0.13

Supervisory A 210,000 Inst.		Supervisory B 240,000 Inst.		Supervisory C 230,000 Inst.		
Month	Changes	Changes/inst.	Changes	Changes/Inst.	Changes	Changes/Inst.
1	110	0.52 x 10 ⁻³	250	1.04 x 10 ⁻³	225	0.98 x 10 ⁻³
2	238	1.14	520	2.16	287	1.24
3	185	0.88	430	1.80	497	2.16
4	425	2.02	300	1.25	400	1.74
5	325	1.55	170	0.71	180	0.78
6	37	.18	120	0.50	50	0.22
7	5	.02	60	0.25		
8			40	0.17		

37

Table 4.6 (continued)

Code Defect Parameters Estimated by Shooman Model

Program	<u>n</u>	_I	E _T /I _T	<u>0</u> 0
Supervisory	A	210K	6.14 x 10 ⁻³	0.875 x 10 ⁻³
Supervisory	В	240	7.97	0.996
Supervisory	с	230	7.48	1.25
Application	A	240	13.20	2.20
Application	В	240	7.70	1,54
Application	С	240	7.00	1.00
Application	D	240	12.90	0.995
Average			8.92	1.26

- E_{T} = Estimated total number of errors at initiation of data collection.
- I_{T} = Number of machine language instructions in the system.
- ρ_0 = Number of errors/number of instructions/month, averaged over the entire duration of time in months.

Table 4.7

Relationship. Between Errors and Program Properties Program as Reported by F. Akiyama

						Errors Detected During Different Phases ³					
		Prog	ram Pr	operties	Total Detected	DB	AT	BT(1)	BT(2)	F	
Module	Sizel	D	J	D+J	Number of Errors	1	2	1	1	2	
MA	4.032	372	283	655	100	58	25	11	4	2	
MB	1.329	215	44	259	18	9	4	4	0	1	
MC	5,453	552	362	914	93	78	6	8	0	1	
MD	1.674	111	130	241	26	21	0	4	1	0	
ME	2.051	315	197	512	71	54	8	5	1	3	
MF	2,513	217	186	403	37	21	9	6	1	0	
MG	699	104	32	136	16						
MH	3,792	233	110	343	50						
MX	3,412	416	230	646	80						
TOTAL	24,955	2535	1574	4109	493						

¹Number of machine language instructions.

²D: number of decision symbols.

J: number of subroutine call symbols.

³DB: testing individual modules.

AT: subsystem connection test. BT: integration test before release.

F: field.

Duration of these phases are shown in month.

References

- 4.1 J. D. Musa, "Validity of Execution-Time Theory of Software Reliability," IEEE Transaction on Reliability, Vol. R-28, No. 3, August 1979.
- 4.2 J. D. Musa, "Software Reliability Data," Data and Analysis Center for Software, 312 (1980).
- 4.3 W. D. Brooks and R. W. Motley, "Analysis of Discrete Software Reliability Models," RADC-TR-80-84 (1980).
- 4.4 A. K. Goel and K. Okumoto, "Bayesian Software Prediction Models," Vol. V, RADC-TR-78-155, (1978).
- 4.5 A. L. Goel and K. Okumoto, "Time-Dependent Error-Detection Rate Model for Software Reliability and Other Performance Measures," IEEE Transactions on Reliability, Vol. R-28, No. 3, August 1979.
- 4.6 M. L. Shooman, "Probabilistic Models for Software Reliability Prediction, <u>Statistical Computer Performance Evaluation</u>, W. Freiberger, Ed., Academic Press, New York, 1972.
- 4.7 J. D. Dickson, J. L. Hesse, A. C. Kientz, and M. L. Shooman, "Quantitative Analysis of Software Reliability," Proceedings 1972 Annual Reliability and Maintainability Symposium, IEEE, 1972.
- 4.8 F. Akiyama, "An Example of Software System Debugging," IFIP, 71, Ljubi, ena, Yugeslavia, 1971.

5.0 Summary of Results

This section presents illustrative results of software failure probability characteristics generated using the analytic tools described in Section 3. Section 5.1 reports results using models supporting the probabilistic framework described in Section 3.2. Section 5.2 reports illustrative regression analysis results on the correlations among failure probabilities and code variables. All these results are based on preliminary, incomplete, unverified, and possibly non-representative failure data and are presented only to illustrate the methodology. Section 5.3 describes estimates of software failure characteristics obtained from the literature and from informal surveys. It too is considered preliminary, incomplete, and possibly non-representative. However, it is used to illustrate that even in a very small sampling of observations and opinions, the estimated characteristics exhibit a wide uncertainty band and heavy dependence on code properties.

5.1 Illustrative Failure Probability Characteristics Results

This section describes results using the published failure rate models and the sigmoidal fault discovery model (Eq. (22) in Section 3.2) developed as part of this research program. Calculations were performed with these models to determine estimates of the number of software errors per line of instruction and the probability of code failure. As described above, the results here are presented to show the feasibility and usage of the methods, not to infer probabilistic results relevant to nuclear codes.

Studies performed using the published failure rate models described in Appendix A provided quantitative insight into both the published mathematical methods and the software failure data described in Section 4. Considerable effort was expended in understanding these models and the role they could play in the mathematical framework of this program. However, for brevity this section reports only a summary of the results obtained and representative illustrations of individual calculations.

First, a quantitative comparison of the above software reliability models was made to evaluate their relative merits. In particular, calculational results were obtained with the deterministic failure rate models of Jelinski-Moranda [5.1], Musa [5.2], Goel [5.3], and Brooks-Motley [5.4], and the stochastic model of Littlewood-Verrall [5.5], applied to the published data [5.6] shown in Table 4.1. An illustration of the results is presented in Figure 5.1 which shows the reliability as a function of code execution time. These reliability functions were obtained using the maximum likelihood estimates (MLE) of the individual model parameters based on the first 100 detected faults of Table 4.1. In all cases, the MLE equations were highly non-linear. Convergence to the solution of these non-linear equations was slow for many of the failure rate models, particularly for the stochastic model. A choice of initial values was critical in obtaining any solution, with a good set of initial values resulting in quickly converged solutions. However, as in any system of highly non-linear equations, it is not a trivial matter to determine whether the converged solution is unique.

For the results presented in Figure 5.1 and Tables 5.1 and 5.2, only the binominal distribution form of the Brooks-Motley model was treated; further,

the error reduction factors, B in the Musa model and α in the Brooks-Motley model were assumed to be 1. The MLEs of the model parameters varied greatly depending upon over what range of failure data they were taken, as indicated in Table 5.1. However, for a given data set, Table 5.1 also shows that the analogous parameters for the different models agree reasonably well. For example, constants of proportionality ϕ in the Jelinski-Moranda model, 1/T N in the Musa model, b in the Goel model, and q in the Brooks-Motley model, are⁰ comparable for all cases. These reflect the probability of error detection and are seen to decrease with error detection as reliability improves.

Figure 5.1 and Table 5.1 reveal some of the generic traits of the failure rate models. Littlewood [5.8], by applying goodness-of-fit tests has shown that his stochastic model provides a better fit to the data than the deterministic models. In addition he observed that the stochastic model was more conservative in its estimate of the reliability. Figure 5.1, as well as other examples not reported here, substantiates Littlewood's observation regarding conservatism. A nonconservative feature of the deterministic models is illustrated in Table 5.1. The parameter \hat{N} estimating the total number of errors in the system at the initiation of detection becomes larger as more fault data are factored into parameter estimation; i.e. \hat{N} becomes larger as more errors are detected. Since the Littlewood-Verrall does not predict N per se, quantitative estimates of its conservatism are not readily made, although its conservatism relative to the deterministic models is shown in Figure 5.1.

Table 5.2 lists software failure probability characteristics of nine of the systems documented in Appendix B and studied with the Brooks-Motley binomial model. As was the case for the Table 4.1 data, \hat{N} , the estimated initial total number of faults in the system, is always just slightly larger than the observed number of faults. In one case, N was computed to be equal to the number of observations. Thus, it is clear that the underprediction of N is a characteristic of the failure rate models studied here, not simply a characteristic of a specific set of data.

To illustrate the use of the sigmoidal fault discovery model to determine the asymptotic number of faults for a given software system, the failure data of Appendix B were analyzed. As shown in Section 3, the model

 $n(t) = \frac{N}{2} \{1 + tanh[k(t-t_0)]\}$ (1)

requires the estimation of the three parameters N, k, and t, with N being the measure of error content. Good initial estimates of the parameters must be supplied for successful regression to be performed. The PAR derivative free nonlinear regression module of the BMDP system was used in the present instance. This module is described in detail in Appendix C. Basically, PAR computes least square estimates of parameters using an iterative pseudo-Gauss-Newton algorithm. For the examples considered here, data near the end of the testing period were weighted more heavily than data near the beginning of the period.

Failure data for nine systems, namely Systems 1, 2, 3, 4, 5, 6, 7, 27, and 40 of Appendix B were analyzed. Figure 5.2 shows a comparison of the observed data and the values predicted with (1) for System 4. This system is a real time command and control software package of 33,500 lines extent, programmed by seven programmers with a total of 53 observed fault-producing failures over a 70-day period. Reasonable agreement is evident indicating that the assumptions

underlying the model are supported by this example. An estimate of 55 total faults was obtained. This number is slightly larger than the number of faults actually found and is in reasonable agreement with failure rate model predictions.

Table 5.3 summarizes the values of N, k, and t for the nine systems to which the sigmoidal model was applied. The initial estimates used in the regression analysis were chosen as follows: The initial value for N was chosen to be the total number of faults observed during the testing period; the initial value of k was set equal to the reciprocal of the total number of observed faults; and t was set equal to the time when half the total observed faults were discovered. Comparison of the values obtained for N with the values listed in Table 5.2 for N shows that the sigmoidal model predicts a total number of faults somewhat larger than the total number of observed faults just as do the failure rate models. These results indicate that the sigmoidal model is a satisfactory alternative to the failure rate models for predicting N.

5.2 Illustrative Regression Analysis Results Correlating Software Failure Probabilities with Code Properties

To illustrate the use of regression analyses to correlate failure probability characteristics with code properties and development variables, the exercise described herein was performed. A log-linear form for the error content per line of instruction, or alternatively the probability per line of instruction that a software fault would cause code failure was assumed as follows:

$$\ln n = a_0 + a_1 t + a_2 I + a_3 P,$$
 (2)

where

t = time in working days, I = number of lines of instruction, P = number of code programmers,

and

n = (N-n)/I,

where

N = the estimated number of total errors

and

n = the observed number of errors.

The choice of independent variables in (2) was determined by the information available about the systems on which the data were taken. The choice of time in working days, rather than time between current and previous failure was made because time in working days is thought to more correctly reflect debugging activity and measure of level of software exercise and fault detection. The choice of the number of lines of instructions as an independent variable was made because a nonlinear relationship between the number of faults and the number of lines of instructions was expected. The choice of the number of programmers was made because the information was available and the oresence (or absence) of a multitude of programmers working on any software system can obviously affect the production, detection, and correlation of faults. Choice of the log-linear form for n was made to accommodate the several orders of magnitude change expected in n.

Fits of the log-linear expression were accomplished using BMDP's PlR multiple linear regression module, described in detail in Appendix C. Basically, PlR computes a multiple linear regression equation on all data and on groups or subsets of the data. The parameter a in (2) can be set to zero prior to the analysis if desired. If a grouping variable is specified to form groups, homogeneity of regression coefficients across groups is tested. It is also possible to specify case weights.

(3)

Figure 5.3 shows a comparison between results obtained with (2) and the observed data for the same nine data sets to which the sigmoidal model was applied. Table 5.4 lists the estimated values of the parameters a. Rather extreme variations ocur at small values of time between the predicted and observed values (i observed values show a lot of scatter in this range as well). As a consequence, the "F" ratio relating fit to the resulting residuals is relatively high (F = 40). Figure 5.4 shows a lognormal plot of the residuals. The straight line represents the expected behavior if the residuals are normally distributed. Significant departures from a normal distribution are evident. The serial correlation of the residuals is relatively high indicating that (2) does not successfully remove the inherent trends in the data.

Of the three independent variables considered in (2), increases in two of the variables act to reduce n while increases in the third cause n to increase. Increasing the number of working days causes n to decrease reflecting successful debugging efforts. The decrease in n with the number of lines of instruction suggests that the number of faults in the code may not be proportional to the number of lines of instruction. The tendency for n to increase with the number of programmers suggests inefficient coordination of the programming effort. It should be remembered that these results are only illustrative. The fact that N increases with the number of observed faults confuses the interpretation of the results. Therefore definitive conclusions cannot be drawn from this analysis.

Again, the above work was done for illustration of the methods and no conclusions are to be drawn. However, the effects of the variables on error content satisfied "reasonableness". Further, the log-linear function used to describe fault probability per line of instruction should also prove useful in future analyses.

5.3 Surveyed Estimates of Software "ailure Probability Characteristics

This section describes observations from the literature as well as solicitations of experts to obtain estimates of software failure probability characteristics. In particular, observations for both code-specific and generic frequencies of software errors or bugs per line of instruction are discussed. In addition, conclusions of a small sample of experts regarding the failure probabilities of large nuclear safety codes are presented.

Consider, first, the frequencies of errors per line of instruction. Boehm reported in a summary paper [5.10] that "each new release of OS/360 contains roughly 1000 software errors." OS/360 is an IBM 360 operating system software package consisting of roughly 300,000 to 500,000 machine instructions. This translates to $\sim 2 - 3 \times 10^{-3}$ errors per instruction. Boehm also reported on code development by IBM utilizing a variety of on-line programming tools, programming systems, and innovative structuring of the software to achieve highly reliable software. With this concept, an 83,000 instruction system for the *New York Times* was developed. During the testing phase 21 errors were found; since release 25 additional errors have been found. For this system, the minimum estimated number of errors per instruction at the time of release is given by $\sim 3 \times 10^{-4}$. Additional error discoveries will increase this estimate. Other published values include a value of 5×10^{-3} errors per machine instruction observed by Rubey [5.11] and a "historical-rule-of-thumb" value by Shooman [5.12] of 10^{-2} for the testing and integration i.e. In an informal survey of approximately 10 code developers at ANL, all responded with basically the same rough estimate of 10^{-3} errors/line of coding for codes that are in production status. An informal survey was also conducted with roughly 10 widely published "experts" in the software reliability field. This survey yielded estimates of 3-12 faults/1000 machine instructions during integration testing, roughly 10^{-2} faults per instruction during system testing, and $1 - 2 \times 10^{-2}$ faults per instruction of various probabilities would be dangerously misleading without first examining the systems in question. It was further opined that the post-release failure data depend strongly on testing effectiveness.

The quick survey discussed above produced estimates of 10^{-2} to 10^{-4} software errors per line of instruction; however, the stage of development of the codes was not always clear. Further, it could not be always discerned whether line of instruction referred to machine instruction or line of coding. Advanced programming languages such as FORIRAN generally normalize to 5-10 machine instructions per line of code. This value depends on the language as well as on the compilers in the computer hardware. Thus, probably the only conclusion that can be drawn here is that the number of software errors in a code is highly dependent on both the specific code properties and on the computer hardware on which the code is executed. As implied above, this conclusion was supported in the informal discussions held with the surveyed personnel.

Concerning failure probabilities of large safety codes, the dicussion was initiated with the question, "In your opinion, what is the probability that a large hypothetical accident code properly executes to provide a qualitatively correct estimate of an <u>important</u> parameter?" "Important" here was loosely defined as one upon which a safety conclusion might be based. "Properly executes" was defined to mean that the software was functioning correctly independent of whether the physical or phenomenological modeling was correct. A "qualitatively correct" result was defined to imply that conclusions drawn would not change because of deviation from "perfect". The conclusions were as follows:

- 1. For a specific code, the application of the code determined the probability. For example, the reliability of an accident analysis code would depend on the type of transient analyzed.
- 2. A safety code is much more reliable after widespread use than when it is released. A newly released code is not reliable.
- 3. The reliability of the code is strongly dependent on the expertise of the user. An inexperienced analyst is unreliable.

For an application for which the code has been "verified" by experience and the input prepared by an expert, opinions of the probability of successful operation generally were above 0.95, although only qualitative statements about confidence were made. The overall conclusion to be drawn from these discussions is that a subjective estimate of the "generic" probability of failure of a large safety code in calculating a hypothetical accident has little meaning. Rather, the estimate would depend upon the code itself, the use of the code, the analyst applying the code, and the experimental history of the code. The need for quantifying the effects of these kinds of dependencies is clearly shown.



Figure 5.1 Illustrative Failure Rate Model Predictions of Software Reliability



Figure 5.2 Sample Regression Results -- Detection of Faults vs. Time

49



Figure 5.3 Sample Regression Results -- Time Behavior of Faults Per Line of Instruction

50



Figure 5.4 Sample Regression Results -- Log Normal Plot of Residuals

Table 5.1

Fault Data Range	1-50		1-100		1-136	
	Ñ	¢,Î _o ,Ê,q	Ñ	\$, T _o , b, q	Ñ	\$, T _o , b, q
Jelinski and Moranda	59	0.1807-3	106	0.6652-4	142	0.3482-4
Musa	59	93.787 (0.1807-3) ²	106	141.812 (0.6652-4) ²	142	202.270 (0.3482-4) ²
Goel	61	0.1667-3	107	0.6459-4	143	0.3409-4
Brooks and Motley	60	0.1784-3	109	0.6250-4	144	0.3382-4

Maximum Likelihood Estimates of Model Parameters¹ as a Function of Fault Detection Data

¹Definitions are as follows: \hat{N} is an estimate of the number of errors in the code at the initiation of detection; $\hat{\phi}$, $\hat{T}_{,}$, \hat{b} , and \hat{q} are parameters that measure or enter into the measure of error detection probability. Exact definitions are found in Appendix A.

 $^2 \; \frac{1}{\hat{T}_0 \hat{N}}$, which is comparable to $\hat{\varphi}, \; \hat{b} \; \text{and} \; \hat{q}$

System	I/I*	N	N/I	Ñ	Ñ - N	ĝ	$\hat{q}(\hat{N}-N)$
1	21,000/19,500	136	6.5-3	142	6	3.5-5	2.1-4
2	27,700/6,600	54	1.9-3	56	2	2.9-5	5.8-5
3	23,400/11,600	38	1.6-3	38	0	6.8-5	0
4	33,500/9,000	53	1.6-3	61	8	6.2-5	5.0-4
5	244,500	831	3.4-3	900	69	8.9-8	6.1-6
6	5,700	73	1.3-2	91	18	3.2-2	5.8-1
17	61,900	38	6.1-4	40	2	1.2-5	2.4-5
27	126,100	41	3.3-4	43	2	7.5-7	1.5-6
40	180,000	101	5.6-4	103	2	2.1-7	4.2-7

Table 5.2 Estimated¹ Software Failure Probability Parameters² for Selected Data Systems

¹The Brooks-Motley failure rate model was used to provide the estimates.

²Parameter definitions:

- I: total number of instructions.
- I*: number of instructions that were modified during fault collection; this data was available only for Systems 1, 2, 3, and 4.
- N: actual number of faults detected when projection estimates were made.
- N: estimated total number of faults at the initiation of fault data collection; obviously $\hat{N} \ge N$.
- N-N: estimate of the number of faults remaining in the system that would be detected eventually; this is a measure of code failure potential.
- q: the probability that, given a fault in the code, it will be detected in a unit of test effort (here 1 s);
- q(N-N): the expected number of faults detected in a unit of test effort and is numerically equivalent to the probability of code failure in a unit of test effort (here 1 s) provided q(N-N) << 1.

T	6. 7	1.2	r	-
la	DI	e	5.	3

Parameters of Interest in the Fault Discovery Model

System	<u>N</u>	<u>k</u>	to
1	161 <u>+</u> 4 faults	$3.76 \times 10^{-2} \pm 2 \times 10^{-3} \text{ day}^{-1}$	66 <u>+</u> 1 days
2	53 <u>+</u> 1	$5.00 \times 10^{-2} \pm 5 \times 10^{-3}$	35 <u>+</u> 1
3	53 + 4	$1.87 \times 10^{-2} \pm 4 \times 10^{-3}$	52 <u>+</u> 14
4	55 + 2	$3.83 \times 10^{-2} \pm 4 \times 10^{-3}$	29 <u>+</u> 2
5	922 + 64	5.09 x $10^{-3} \pm 4 \times 10^{-4}$	258 + 14
6	77 + 3	$4.44 \times 10^{-2} \pm 3 \times 10^{-3}$	23 + 1
17	37 <u>+</u> 1	$6.31 \times 10^{-2} \pm 4 \times 10^{-3}$	24 <u>+</u> 1
27	42 + 4	2.39 x $10^{-2} \pm 5 \times 10^{-3}$	29 <u>+</u> 6
40	102 + 3	$6.76 \times 10^{-3} \pm 1 \times 10^{-3}$	130 ± 10

Note: Quoted uncertainties represent one standard deviation.

Table 5.4

Parameters of Interest in the Log-Linear Probability Model

Coefficient	Value
a ₀	-7.08329
a ₁	$6.19 \times 10^{-3} \pm 1 \times 10^{-3} \text{ days}^{-1}$
a ₂	$-8.10 \times 10^{-3} \pm 1 \times 10^{-3}$ (thousand lines) ⁻¹
a ₃	$6.98 \times 10^{-2} \pm 1.6 \times 10^{-2} (\text{programmers})^{-1}$

Note: Quoted uncertainties represent one standard deviation.

References

- 5.1 Z. Jelinski and P. Moranda, "Software Reliability Research," <u>Statistical</u> <u>Computer Performance Evaluation</u>, W. Freiberger, ed. Academic Press, New York, 465 (1972).
- 5.2 J. D. Musa, "A Theory of Software Reliability and its Application," IEEE Trans. on Software Engineering, Vol. SE-1, No. 3, 312 (1975).
- 5.3 A. L. Goel, "Software Error Detection Model with Applications," Journal of Systems and Software 1, 243 (1980).
- 5.4 W. D. Brooks and R. W. Motley, "Analysis of Discrete Software Reliability Models," RADC-TR-80-84 (1973).
- 5.5 B. Littlewood and J. L. Verrall, "A Bayesian Reliability Growth Model for Computer Software," 1973 IEEE Symposium on Computer Software Reliability, 70 (1973).
- 5.6 J. D. Musa, "Software Reliability Data," Data and Analysis Center for Software, (1980).
- 5.7 J. J. More, B. S. Garhom, and K. E. Hillstrom, "Users Guide for Minpack-1," ANL-80-74, 1980.
- 5.8 B. Littlewood, "A Critique of the Jelinski-Moranda Model for Software Reliability," 1981 Proceedings Annual Reliability and Maintainability Symposium, 357 (1981).
- 5.9 Private Communication, W. Vesely, January 15, 1981.
- 5.10 B. W. Boehm, "Software and Its Impact: A Quantitative Assessment," Datamation, 48, May (1973).
- 5.11 R. J. Rubey, "Quantitative Aspects of Software Validation," International Conference on Reliable Software, 246 (1975).
- 5.12 M. L. Shooman, "Software Reliability Data Analysis and Model Fitting," Workshop on Quantitative Software Models, 182 (1979).

6.0 Summary, Conclusions, and Future Directions

This section will summarize the preceding sections and outline the status and future directions of the tasks involved in this research program.

Section 2 discussed validation and verification of software. Although this was not a direct part of the FY 81 research program there is a clear need to compile a recommended list of procedures for validation and verification of nuclear safety software to provide the best assurance that code results that cannot be checked by conventional means are correct (within the framework of the modeling). Two examples of "unverifiable" results in the area of nuclear safety are presented for illustration: (1) An energy release from a postulated steam explosion in an LWR accident; and (2) A probability of a certain mode of failure in an LWR plant. In the first example, the phenomenology of large scale fuel coolant interactions is not known well enough to allow reasonable prediction even if the boundary and initial conditions for such an event could be accurately established (they cannot be). Thus, analytic and experimental benchmarks are not available and bounding considerations provide the only clue to reasonableness of results. In the second example, consider the specific case of a very complicated and uncertain system evaluated using a fault tree network and for which the predicted probability of a certain mode of failure is 10⁻¹⁵. Here, not only do the system behavior uncertainties dominate, but the magnitude of the answer is beyond the analysts "intuition" for ascribing reasonableness to a result. Thus, an outline of ways to provide assurance that such "unverifiable" results are correct is needed.

Section 3 presented the mathematical approach of this program and Section 5 presented results generated using some of the supporting models. As stated throughout this report, these results were generated only to illustrate the methods and were based on an analysis of failure data that are preliminary, unverified, and possibly non-representative of software relevant to nuclear safety. Further, what has been done to date has been to estimate parameters that support the framework of this program. Actual implementation of the Poisson model to predict probabilities of failure and associated uncertainty bonds for nuclear codes remains to be done.

Specific areas within the approach that must be dealt with include the proper classification of software errors, faults, and failures both for data organization and subsequent regression analysis. With respect to the regression, the optimum choice of dependent variables needs to be established. For example, in the Poisson model, the characteristic probability q_{jk} is an obvious candidate. However, expected numbers of errors, i.e. $M_{jk} q_{jk} may$ in certain circumstances be amendable to regression. Finally, regression analysis may be performed directly on the reliability. Investigation of the number of errors per instruction in a code is useful since this quantity is a convenient figure of merit with which to measure the developmental progress of a code. A Bayesian approach to updating reliability estimates was also introduced.

Although all these concepts bear investigation, an overriding factor in this program is that the data that will become available in the near term, say several years, simply do not justify the development of sophisticated mathematical modeling. Thus, a pragmatic approach dictates that after the initial

computer establishment of the framework, data acquisition efforts will dominate model development work.

Section 4 described the data acquired to date. Short term data collection efforts will be directed to nuclear codes. Other sources of relevant data include the human error literature and solicitation of "experts". However, this is a minor part of the overall data acquisition effort. A development effort that would unify these and subsequent data acquisition efforts is the creation of a readily accessible computerized information base and resource center. The components of this information base would include the necessary mathematical framework and associated models with which to make reliability predictions as well as the software failure data with which to verify and upgrade this framework. To facilitate access to and use of this information base with its concomitant analytic capability, as well as to provide a continuous updating capability that could accept failure data from throughout the industry, a Software Reliability Information Center and Computerized Data Bank and Analysis System is proposed. This would be established at the National Energy Software Center and could be made accessible to the nuclear industry. The need for this centralized data and analysis center has been emphatically demonstrated to our staff in our attempts to obtain documented code failure data. Such data have been logged by very few code developers in any consistent fashion in the nuclear industry and have been difficult to obtain. A failure data center should provide an industry-wide focus on software reliability, as well as providing a comprehensive, centrally located data base.

Section 5 described the results to date. As cited they were strictly used to illustrate the models. In the following year, gross estimates of reliability characteristics and their dependencies, similar to those provided, but based on an improved nuclear data base will be provided using the mathematical approach of this program. Tie-ins between the non-nuclear and nuclear data base will also be investigated.

In summary, a general mathematical framework for predicting reliability characteristics has been established. At the core of this framework is a Poisson model for the number of code errors of a specified type in a given computer application. Methods of implementing this framework have been developed and include both mathematical modeling and the proper specification and handling of data. The product of this framework includes predictions of reliability characteristics such as expected number of errors and probability of failure in a given computer run. By accumulating failure data and specifying the code characteristics associated with each data set, multivariate regression techniques can be used to identify the important code parameters and controlling developmental parameters. Actual computer implementation of this framework was started. Efforts in the near term have been described above.



Appendix A

Summary of Published Failure Rate Models

This appendix provides a review and summary of predictive mathematical software reliability models developed in the military, aerospace and communication industries. As a background for this review, the mathematical basis of software reliability is first provided. Summaries of the individual models are then provided with highlights discussed in a "comments" paragraph. Results of the application of a selected group of these models to failure data are presented in the text of this report.

In general the models discussed here assume functional forms for the code failure rate or the number of errors remaining in the code. These models are then fit to failure data to estimate, generally using maximum likelihood, values for the model parameters. The models are subdivided into:

- Deterministic models using error detection vs. time histories to evaluate the parameters of deterministic functions assumed for the failure rate;
- (2) Deterministic models geared to similar parameter estimation using histories that measure numbers of errors in time intervals;
- (3) A stochastic model that uses error detection vs. time histories to estimate the parameters of probability density functions assumed for the failure rate and time to failure.

Prior to the presentation of the models, a short introduction to the mathematics of reliability is provided to facilitate the discussion of the models. The reliability is the probability of successful operation over time t and is given as a function of time by

$$R(t) = P(t > t)$$

where t is the time of failure. The probability of failure is given as the complement of reliability, namely

$$F(t) = 1 - R(t) = P(t < t) = \int_{0}^{t} f(x) dx$$

where the failure density function f is given by

$$f(t) = \frac{dF(t)}{dt} = -\frac{dR(t)}{dt}$$

The hazard function or failure rate is defined by

$$Z(t) = \frac{P(t \le t \le t + dt)}{P(t > t)dt} = \frac{f(t)}{R(t)} = -\frac{1}{R(t)} \frac{dR(t)}{dt}$$

and is the conditional probability of failure in the interval $t < t \leq t + dt$ given survival up to time t. From the definition of the hazard rate, the reliability function can be shown to be

$$R(t) = e^{-\int_{0}^{t} Z(x) dx}$$

The mean time to failure (MTTF) is given by

MTTF = $\int_{0}^{\infty} tf(t)dt$,

from which integrating by parts leads to

$$MTTF = \int_{0}^{\infty} R(t) dt.$$

In general, because software bugs are corrected as they are detected, the failure rate Z(t) is a decreasing function of code lifetime. This characteristic gives rise to a reliability function that increases with the life of the code.

Reliability models developed by Jelinski-Moranda (J-M) [A.1], Musa [A.2], Goel [A.3], Shooman [A.4], Schick-Wolverton [A.5], Brooks-Motley (B-M) [A.6], and Littlewood-Verrall [L-V) [A.7] are summarized in the following paragraphs. These models represent a group of the most frequently cited models in the literature. The first six of these models are considered deterministic because the failure rate or number of remaining errors is expressed as a deterministic function of a small number of parameters. As cited above, the deterministic models use either time-to-failure data or number-of-failures-ina-time-interval data to estimate parameters. The Littlewood-Verrall model is considered stochastic because the failure rate is treated as a random variable with a probability density function dependent upon a small number of parameters. The Jelinski-Moranda model served as a basis for the development of other deterministic models. In fact, Musa, Shooman, Goel, Schick-Wolverton and Brooks-Motley models are often referred to as Jelinski-Moranda type models because of the similarity in the formulation of the hazard function. In all of these models, the hazard function is assumed to be proportional to the number of errors remaining. To facilitate reference to the literature the nomenclature used in the original papers is preserved in this report.

A.1 Deterministic Models Using Time-to-Failure Data

A.1.1 Jelinski-Moranda Model

 $Z(t_{i}) = \phi[N - (i - 1)]$

where N = total number or errors initially in the system

i = number of errors found in debugging time interval t;

Comments

The Jelinski-Moranda (J-M) model assumes that each software bug has an equal probability of being detected, an unrealistic assumption since more easily-detectable errors are eliminated first. A potentially more serious drawback with this model is that the MLE (maximum likelihood estimate) may predict infinite values for model parameters.

$$Z(t) = K[E_T/I_T - \varepsilon_c(\tau)] = K \varepsilon_r(\tau);$$

thus

$$R(t) = e^{-K\varepsilon}r(\tau) t$$

and

MTTF
$$(\tau) = \frac{1}{K\left[\frac{E_{T}}{I_{T}} - \varepsilon_{c}(\tau)\right]}$$

where

 E_{τ} = total initial # of errors (unknown)

 $I_{T} = # of instructions$

 $\varepsilon_c = \# \text{ of errors corrected (normalized to I_T)}$

 ε_r = # of errors remaining (unknown) normalized to I_T

- τ = debugging time measured from the beginning of the system testing period
- t = code operating time measured from the beginning of the system testing phase (may consist of many testing periods). In some cases $\tau = t$

K = constant of proportionality (unknown)

Comments

- The Shooman model is essentially a J-M model where all parameters are normalized to the number of instructions.
- 2) The reliability function R(t) is defined for all $t \ge 0$. The time variable t is a measure of the operating time since the initial activation of the system and τ is a measure of the calendar time since the beginning of the system integration.

3) There are two unknown parameters, E_T and K, in R(t). To solve for these two parameters, the MTTF function, is evaluated at two different debugging times, τ_1 and τ_2 , where $\tau_1 < \tau_2$ and ε_c (τ_1) < ε_c (τ_2). With the computed E_T and K, R(t) can be <u>evaluated</u> for τ_3 for which ε_c is known; if ε_c is not known, an estimate must be made based on the previous data. The adequacy of this technique depends on the knowledge of the failure detection rate; the extrapolated value of R(t) based on τ_i for τ_n for large n may be very poor unless the failure detection rate is almost constant.

î

$$Z(\tau) = KfE_r = \frac{1}{T_o} [1 - \frac{i - 1}{M_o}]$$

- = accumulative execution time
- E_ = # of errors remaining
- f = linear execution frequency, = average instruction execution rate number of instructions in the program
- K = a constant of proportionality, the "error exposure ratio", which relates the error exposure frequency to r
- i = number of errors detected

$$T_0 = initial MTTF$$

$$M_0 = N_0/B$$

where

N = number of initial errors

- B = error reduction factor
 - = average ratio of the error reduction rate to the failure
 occurrence rate

Comments

- 1) The Musa model appears to be widely accepted in the field [A.8]. Z (τ) is proportional to the number of errors remaining and the linear execution frequency. The time variable τ in Musa's model is the accumulative execution time or the actual CPU time utilized in executing the software.
- The parameter B can be adjusted to treat incorrect debugging (additional errors introduced by attempts to correct errors), as well as corrections for the learning process that the

programmers experience. The parameter B is postulated by Musa to be nearly constant for large projects due to averaging effects. This factor is usually positive and less than one; however, it can be greater than one when correcting one error leads to other errors. In practice, B is difficult to quantitatively determine. For B = 1, the Musa model reduces to the J-M model.

A.1.4

Schick and Wolverton Models

$$t = \phi \left[N - (i - 1) \right] t_i;$$

 $Z = \phi \left[N - (i - 1) \right] (-at_i^2 + bt_i + c)$

modified S-W model

S-W model

where

t; = debugging time

proportionality constant

N = initial error content

i = number of errors detected

a,b,c = positive constants

Comments

- The Schick and Wolverton model is a modified J-M model to better account for the variation of the failure rate with time.
- The modified Schick and Wolverton model expresses the time variable as a quadratic function, which implies that the failure rate initially increases to a maximum before becoming a decreasing function.

A.2 Deterministic Models Using Failures-in-Fixed-Time-Intervals Data

The models described above are used with error data characterized by recording the time between failures. The next two models express the hazard function in terms of the number of remaining errors as do the J-M type models, but are designed to use failure data in the form of number of failures in time t, (n,t) denoting the number of errors detected in time t. The model by Goel utilizes a nonhomogeneous Poisson process for software failure detection. The model by Brooks and Motley features the use of binomial and Poisson distributions (only the binomial distribution is presented here) to represent the number of errors in a test.

A.2.1 A. L. Goel Model

This model describes a nonhomogeneous Poisson process (NHPP) whose mean value function is derived by a deterministic analysis of the software failure process.

Let n(t) be the cumulative number of software errors detected by time t and a be the total number of initial errors or the total number of errors to be detected eventually; then n(t) has following properties

$$n(t) = \int_{a}^{0} \text{ when } t = 0$$
when $t = \infty$

The number of errors detected in $(t, t + \Delta t)$ is assumed to be proportional to the number of remaining errors;

$$n(t + \Delta t) - n(t) = b(a - n(t))\Delta t$$

where b is a proportionality constant. Then

$$n(t) = a(1 - e^{-Dt})$$

Let $\{N(t), t \ge 0\}$ be a nonhomogeneous Poisson process (NHPP) describing the number of errors counted in (0,t) and having the following properties:

1.
$$N(0) = 0$$

2. {N(t),t>0} has independent increments

3. P{2 or more events in $(t, t+\Delta t)$ } = o (Δt)

4. P{exactly ! event in $(t,t+\Delta t)$ = $\lambda(t)\Delta t + o(\Delta t)$;

where $\lambda(t)$ is the hazard function and is called the intensity function. Further, let

$$m(t) = \int_0^t \lambda(s) ds.$$

Then for t>0, N(t) has the Poisson distribution

$$P[N(t) = y] = \frac{\{m(t)\}^{y}}{y!} e^{-m(t)}, y \ge 0$$

with the expected value of N(t) given by

$$[N(t)] = m(t);$$

m(t) is called the mean value function of the NHPP. For the homogeneous case where λ is a constant, m(t) = λ t and

$$P[N(t) = y] = \frac{y}{y!} e^{-\mu}$$

where the mean value $\mu = \lambda t$. Choosing the mean value function m(t) to be equal to the cumulative number of errors n(t) yields

$$m(t) = a(1 - e^{-bt}),$$

implying that

 $\lambda(t) = abe^{-bt}$

and

$$R(t) = \exp \left[-\int_{s}^{s+t} \lambda(x)dx\right] = \exp\left[-a\{e^{-bs}-e^{-b(s+t)}\}\right],$$

where

s = time the last failure occurs

This conditional reliability function describes reliability since the last failure.

Comments

- A deterministic model for n(t) is required to obtain the functional representation of m(t) which satisfies the NHPP conditions, rather than the converse.
- 2. This model analyzes data sequences of the form (ε_i, t_i) where ε_i is the number of failures in calendar time t_i .

A.2.2 Brooks and Motley Model

This model treats each program as a set of modules. The expected number of errors in the portion of the total system which is under test on occasion i can be found by taking the summation over all modules which are being tested. The estimate of the number of errors prior to test i for the portion of the system under test is given by

$$N_{i} = \sum_{j \in J_{i}} (W_{j}N - \alpha N_{i-1,j})$$
$$= N \sum_{j \in J_{i}} W_{j} - \alpha \sum_{j \in J_{i}} N_{i-1,j}$$

where

J; = the set of modules tested on occasion i

 $\sum_{j \in J_i} \equiv \text{summation over all modules that are elements} \\ \text{of the set } J_i$

 $\sum_{j \in J_i} W_j$ = the fraction of the system which is under test
- "N = number of errors in system before the first testing occasion
- α = probability of correcting errors without reinserting additional errors

$$N_{i-1,j} = \sum_{m=1}^{1-1} n_{mj}$$
 and is the number of errors actually detected in

that portion of the system prior to the ith occasion.

If one defines

q = probability that any given error is detected during a unit of test effort

and

t = the system test effort on occasion i; it can be given as the
 execution (CPU) time, calendar time, number of tests, etc.,

then the probability of detecting any given error during the i-th test occasion will be

 $q_i = 1 - (1 - q)^{t_i}$.

Since \mathbb{N}_i errors are exposed during the i-th test occasion, the probability of detecting x, errors will be given by the binomial distribution

$$p(x_i) = \begin{pmatrix} N_i \\ x_i \end{pmatrix} q_i^{x_i} (1 - q_i)^{N_i - x_i}$$

The probability of detecting no more than M errors during the i-th test occasion is given by

$$\sum_{x=0}^{M} p(x) = \sum_{x=0}^{M} {\binom{N_i}{x}} q_i^{x} (1 - q_i)^{i-x}$$

The relationship between the Brooks-Motley model and other J-M type models becomes more apparent if t, is regarded as execution or calendar time. Then the reliability as a function time t, after the end of the (i-1)st testing period is obtained by setting M=O in the preceding equation. Then

$$R = (1-q_i)^{N_i} = (1-q)^{t_i N_i}$$

and for the failure rate,

$$\lambda_{i} = -\frac{1}{R} \frac{dR}{dt_{i}} = -N_{i} \ln(1-q) \approx \frac{N_{i} q_{i}}{t_{i}}$$

where

$$n'_i = N_i q_i$$

is the expected number of failures in time t_i . The failure rate is proportional to the number of undetected errors remaining in the system.

Comments

- 1. Basic assumptions are very similar to the J-M model, where the proportionality factor ϕ in the J-M model and q in the B-M model are the measure or probability of error detection.
- 2. The B-M model has an advantage over the others in that it treats modular level and system level reliabilities. This is desirable in analyzing complex reactor design and safety codes. The B-M model examines the modular level reliability by assigning "probability-of-usage" weights, W,, the determination of which may be non-trivial. This treatment accommodates the fact that programs may be used for years without error if program use is restricted to a set of well validated modules.
- 3. α is similar to B in the Musa model. All three parameters, N, q, and α , are estimated by the maximum likelihood method.
- This model can make estimates of the following items based on the data of form n failures in period j.
 - current and future reliability,
 - time to achieve specified reliability,
 - probability of passing a reliability requirement test.

A.3 A Stochastic Model

The final model is a stochastic model, developed by Littlewcod and Verrall, which treats the execution time and the failure rate as random variables.

Littlewood and Verrall Model

This model attempts to account for the random process of input selection and the stochastic nature of the times to failure and the associated failure rates with a mathematically tractable model. The function

$$pdf(t_i|r_i = \lambda_i) = \lambda_i e^{-\lambda_i t_i}$$
, $t_i > 0$

represents the probability per unit time that the i-th failure occurs at time t, given that the failure rate $r_i = \lambda_i$, and

$$pdf(\lambda_i|i,\alpha) = \frac{\psi(i)^{\alpha}\lambda_i^{\alpha-1} e^{-\psi(1)\lambda_i}}{\Gamma(\alpha)}, \quad \lambda_i > 0$$

gives the probability per unit λ that the failure rate between the i-lst and i-th failure is λ_i . $\psi(i)$ is a monotonically increasing function of i, e.g., $\psi(i) = \beta_1 + \beta_2 i$. The exponential distribution for time between failures is chosen because failures are considered as constituting a random process (Poisson process) and the choice of a family of Gamma distributions for the failure rates is justified by its flexibility (having two parameters, $\psi(i)$ and α), correct range $(0, \infty)$ and mathematical tractability.

Combining these two,

$$pdf(t_i|\psi(i),\alpha) = \alpha \left[\frac{\psi(i)}{t_i + \psi(i)}\right]^{\alpha} \frac{1}{t_i + \psi(i)}$$
.

The failure rate function is given by

$$Z(t_i) = \frac{\alpha}{t_i + \psi(i)} ,$$

from which the reliability function is

$$R(t_i) = \left(\frac{\psi(i)}{t_i + \psi(i)}\right)^{\alpha}$$

Comments

- 1. The stochastic nature of time and failure rate are considered.
- 2. $\psi(i)$ is a scaling factor and aids in estimating the qualitative behavior of the reliability without computing R(t). If $\psi(i)$ increases more rapidly than i, a faster than linear reliability growth is implied.

References

- A.1 Z. Jelinski and P. Moranda, "Software Reliability Research," <u>Statistical</u> <u>Computer Performance Evaluation</u>, W. Freiberger, ed. Academic Press, New York, 465 (1972).
- A.2 J. D. Musa, "A Theory of Software Reliability and its Application," IEEE Trans. on Software Engineering, Vol. SE-1, No. 3, 312 (1975).
- A.3 A. L. Goel, "Software Error Detection Model with Applications," Journal of Systems and Software 1, 253 (1980).
- A.4 M. L. Shooman, "Software Reliability Data Analysis and Model Fitting," Workshop on Quantitative Software Models, 182 (1979).
- A.5 G. J. Schick and R. W. Wolverton, "An Analysis of Competing Software Reliability Models," IEEE Transactions on Software Engineering, Vol. SE-4, No. 2, 104 (1978).
- A.6 W. D. Brooks and R. W. Motley, "Analysis of Discrete Software Reliability Models," RADC-TR-80-84 (1980).
- A.7 B. Littlewood and J. L. Verrall, "A Bayesian Reliability Growth Model for Computer Software," 1973 IEEE Symposium on Computer Software Reliability, 70 (1973).
- A.8 H. B. Chenoweth, "Modified Musa Theoretic Software Reliability," 1981 Proceedings Annual Reliability and Maintainability Symposium, 353 (1981).

Appendix B

Software Failure Data Sets

This appendix contains 16 data sets generated by John Musa [B.1]. These data sets were obtained from the Rome Air Development Center and are stored on tape.

Reference

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B.1 J. D. Musa, "Software Reliability Data," Data and Analysis Center for Software, 312 (1980).

i	ti	ďi		i	ti	ďi	i	ti	ďi
1	3.	1	5		148.	56	101	40.	74
5	50.	5	5	2	21.	57	102	143.	74
3	113.	4	5	5	253.	57	105	100.	14
4	*1.	10	5	£	134.	57	104	Ű.,	74
5	115.	11	5	ş	351.	57	105	3110.	75
6	9,	8.5	5		193.	59	100	1247.	76
7	2.	17	5	1	236.	54	107	943.	76
8	91.	50	5	5	51.	59	108	100.	76
9	112.	50	5	,	364.	54	109	H75.	11
10	15.	50	0	2	748.	59	110	245.	77
. 11	134.	50		t	е.	59	111	129.	71
12	59.	50	0	2	535.	54	112	1897.	78
13	11.	50	6	5	3:0.	59	115	447.	79
14	24.	50	6	2	305.	61	114	300.	74
15	103.	50	6	>	1555.	50	115	440.	74
10	66.	50	0	>	543.	63	116	122.	19
17	070.	30	6	7	10.	63	117	490.	79
10	140.	.50	6	8 . I	16.	63	116	944.	80
14	20.	3.0	0	*	524.	64	114	1002.	60
20	114.	30	7	3	379	P3 44	120	22.	80
21	325.	50	/		44.	64	121	75.	60
22	>>*	50	1	2	150'	64	155	402.	80
23	646.	51	1	5	610,	64	123	5509.	81
24	60.	31	1	3	500.	64	154	100.	81
23	422.	. 51	1	5	500.	64	125	10.	81
20	100.	32	1	5	529.	65	159	1071.	58
24	10.	32	/		201.	65	121	371.	85
20	1140.	33		5	160.	65	158	140.	63
1.0	600.	34	1		828.	6.0	159	6150.	83
8.4	12.	40			1011.	00	130	3321.	85
24	50.	02		5	445.	00	151	1045.	84
34		40	0		296.	00	132	64H.	84
30		0.0		2	1/25.	6/	155	5485.	87
25	327	40	0	5. L	1004.	01	139	1160.	87
10		40	0	2	1703.	60	135	1864.	66
17	170	40		2	000.	00	150	4116.	45
3.11		40	0		403.	60			
14	457	47	e	7	107.	64			
0.0	4.000	0.7		9. G	35.	64			
41	47.	47	9	. · · ·	120				
42	256	67	9		33.33	7.0			
0.5	452		9		2010	10			
44	255	5.5		8. 1	2430.				
45	197	54			641	12			
40	195	54			12	72			
47	0.	54	9	7	261	12			
48	74.	59	0	A	1800	23			
44	810.	50	0	2	865	23			
50	1351-	50	1.0	5	1415	14			

Table B.1 Failure Data Set 1 of J. D. Musa

.

i= fault number, t_i = execution time, d_i = day of failure

i	ti	di	i	ti	di	i	ti	ďi
1	191.	1	21	638.	32	41	160.	46
2	722.	2	22	273.	32	42	4225.	4.5
3	280.	11	23	1212.	33	43	15600.	93
4	290.	11	24	612.	33	44	0.	93
5	290.	14	29	615.	33	45	0.	53
6	385.	23	26	1215.	33	46	300.	53
7	570.	23	27	2715.	37	47	9021.	57
8	610.	23	2.9	3551.	37	48	2519.	64
9	365.	23	29	800.	38	49	6450.	64
10	370.	23	30	1910.	39	50	2348.	67
11	175.	23	31	0900.	38	51	2750.	69
12	360 .	27	32	3300.	38	52	6675.	71
13	800.	27	33	1510.	41	53	6945.	71
14	1210.	2.0	34	175.	42	34	7859.	72
1.9	407.	29	3.5	1926.	47			
1.0	50.	29	36	103.	43			
11	660.	29	37	601.	43			
18	1507.	31	3.8	50.	43			
19	625.	31	39	769.	43			
20	912.	32	40	900.	45			

Table B.2 Failure Data Set 2 of J. D. Musa

i= fault number, t_i = execution time, d_i = day of failure

. .

Table B.3 Failure Data Set 3 of J. D. Musa

i	ti	ďi	1	ti	ďi	i	ti	ďi
1	115.	1	16	788.	18	3.1	10571.	4.7
2	υ.	1	17	222.	18	31	564	47
3	83.	3	10	12.	18	5.5	27/0.	41
44	170.	3	19	615.	18	54	552.	48
5	194.	5	20	589.	20	35	5545	50
6	150.	3	15	15.	26	56	11696	50
7	1077.	3	22	390.	20	37	6124	54
8	15.	3	23	1853.	27	18	2546	55
2	15.	3	24	1357.	30			
10	92.	3	25	4508.	36			
11	50.	3	20	H34.	3.8			
12	71.	3	21	3400.	44			
1.5	000.	6	28	D	4.0			
14	1189.	8	24	4561	42			
15	40.	в	30	3180.	44			

i= fault number, t_i = execution time, d_i = day of failure

i	t _i	di	i	ti	di	1	ti	di	
1	5.	1	21	103.	26	41	υ.		
5	73.	1	22	10.	26	42	643.	46	
3	101.	1	23	115.	27	43	A81.	40	
4	491.	5	24	17.	27	44	109.	48	
5	5.	5	25	284	27	45	469.		
6	5.	5	26	296	27	46	710.	42	
7	28.	ŝ	27	215	27	47	604.	44	
8	130.	5	28	116	27	68		4.8	
9	478.	9	29	283	11	0.0	774	50	
10	325.	a	10	50	11	50	354	50	
11	147.	10	11	108	11	51	14.17	50	
12	198.	10	i2	279	11	5.2	16300	20	
13	22.	10	11	140	12	51	16700.	70	
1.4	56.	10	19	62B	12		1200,		
15	020.	20	15	183	12				
16	92.	20	16	2463					
17	520.	20	17	100					
1.6	1020	24	18	21.28	41				
19		26	19	285	46				
20	92.	26	40	171.	43				

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table b.4 failure baca set 4 of 0. D.	Table	B.4 Fai	lure	Data	Set	4 01	J.	D.	Musa
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i = fault number, $t_i = execution$ time, $d_i = day$ of failure

Table B.5 Failure Data Set 5 of J. D. Musa

, p	27	0.0	100	140	100	100	104	102	102	105	501	10.4			111	111	112	113	115	113	115	117	110	116	115	151	122	123	0.71		130	134	154	154	134	155		140	1.1	141	140	201	145	153	154	130	150	1.51	101	194		191	
ţ,	7441.	1966	25512.	.1.6,66	5260.	1509.	71289.	8999*	*un2ne*	190560.	C2660.	135951.	.12412		12123.	1215-	*2"45.	13151.	10104.	18122.	61553.	R6200.	3269.	12512.	24540.	100000.	00000	22500.	10101	1174R.	.4069.	\$5313.	17040.	5150.	40201	-00¢			14600.	4400.	a0400.	ledo.	I uowen.	295080.	50A40.	FOLDF.	10614.	******	.01100	17,000.	1 4500.	11000	
ŗ	500		222	623	653	<30	631	632	255	<54	652	630	631		210	1.2	202	243	244	245	540	247	602	544	620	142	555	625	200	154	152	256	239	600	102	202	503	300	200	201	205	407	\$10	112	216	273	214	512	912	218	224	080	2.7.2
, p	11	14	14	15	51	24	78	7.6	84	78	30	7 1	0 1	0 1	0.0	20	00	40	40	00	. 0.6	16	16	56	26	26	26	25	22	20	10	26	26	10	95	56	5.5	100	10	44	94	73	77	34	0.0	56	58	0.5	1.0	10		3	
t,	21 4 4 10	De Tom	40/20	weed.	175080.	2965.	1 54447.	3455.	23407.	\$03*.	• 0		140400.		1 564441	2941	1440.	6160.	. 11	173.	17246.	12000.	35+30.	35265.	5511.	\$145.	3524.	610.	.0002		1001	4500.	10005.	18532.	6240.	15000.		31709.			.0	• 6	2450.	1909.	3840 ·	37500.	*0	Sloc0.	50115.	2313.	12000	11111	* 31 1 1 F
.t	041	1 7	171	110	173	1/4	5/1	110	121	174	114	0 1 1	141	201	1.86	28.5	180	181	845	169	190	141	201	193	141	561	1 36	171	0.0	2000	100	202	203	205	205	505	102	202	210	211	212	213	214	215	210	217	214	513	022	122	222	222	
, p	-		20	27	43	2.2	1.4	6.0	45	45	51	0	17		1.5		20	20	15	15	51	51	15	52	52	25	53	53	~ ~	20	1 1	24	54	54	55	55	45	20	0 0	90	24	29	63	43	0.1	67	R.B.	44	0	14	10		2.5
t.	2114.1	11200	13720.	11220.	10860.	46380.	2220.	.26400.	4500.	6989	>400.	25400.	13500.	-4400°	15700.	1440	180.	0800.	7480.	3000.	2100.	13860.	9640.	13980.	12120.	6660.	17040.	15240.		.000	.000 ×	5400.	300.	\$00.	Sqea0.	\$640.	19200.	2000.	110700.	43200.	27900.	10800.	27060.	35230.	21120.	5.45 wu.	19200.	600°.	20000.	25100.	14100.	2044U.	* X2610
ŗ	111			115	117	411	11.2	1.20	121	122	125	124	100	100	101	001	110	111	132	133	154	135	136	137	130	139	140	101	146	6 n l	101	146	101	601	149	150	151	156	153	155	156	151	158	154	160 .	141	241	165	101	105	100	101	101
.p	30		1	22	25	22	24	23	23	52	5.2	52	52	52	24.	20.	24	24	50	50	54	30	52	3.5	35	33	35	35	35	22	0.4		36	30	30	15	51		11	87	8.2	30	3.4	34	5.5	1.5	39	34	54	53	2.4		
ţ.	1419		1001	12760.	15184.	+04°	10050.	5540.	5400.	5auo.	10004.	* 11-11 *	247.		51600.	1000		17100	CH110.	6720.	2580.	16600.	61869.	38049.	25200.	÷09	58140.	1400.	24163.	* C2020.		10200.	400.	4429.	5100.	5490.	17100.	1264.	* 0000 T	184	.0404	21501.	563.	3120.	15380.	11280.	4020.	2461.	1 abul.	.009 ·	*005	*004	* 204
į	51	10	0.05	6.0	10	29	6.5	19	54	99	61	20	2.4		1.1	31	70	15	11	11	7.8	14	80	18	2.8	83	84	92	0	10	00	00	10	20	56	24	50	0	10	20	100	101	102	103	104	105	100	101	901	501	110		316
·p	-		~ ~			4	7	1	Þ	2	10	æ	2	7 3			101					11	12	14	12	14	13	15	2			5		51	16	9	10	4					z	91	18	1.8	14	19	14	20	20	20	0.2
t,		- 12616	.0*11¢0.	14240	21460.	630.	2201.	lesau.	77000.	12500.	42340.	2740.	16020.	7200.	160.			RADO.	181	7180.	600.	6420.	12400.	300.	25800.	.003.	15720.	19380.	*<80°.	15000.		2280	a70.	10080.	\$220.	.0000	1000.		* 0000 ×	12400.	24000	60.	1740.	60.	2520.	2289.	16990.	1540.	16620.	7580.	\$000.	11560.	11560.
•					5	0	2	æ	0	10	11	12	13	2	-		14			17	55	25	54	25	99	27	28	58	90	16	25	Ia	35	30	37	34	39	40		10	10	58	40	47	8.7	40	Su	15	24	53	50	55	20

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Table B.5 (Cont'd)

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p.	276	212	211	278	618	217	280	280	200	683	280	107	102	102	202	202		203	244	642	200	680	440	288	2007	400	107	19.4	188	20%	204	× 0.9	902	407	705	290	240	1+2	242	245	242	542	245		222	1 1 1 1	200	240	244	294	244
Ţ,	67850.	41342.	540.	40350 .	« DE00.	10115.	252.46	050.	19442.	\$ 600.	7100.	62100.	34140.	12360.	*0200*	.00022	12165.	20105	25050.	1724474.	12000.	\$400°	* UU*	11400.	3480.	.005	\$50#0*	*0000*	1000	5300.	a2019 .	\$600°	* 745.6	\$2400.	10000	18400.	.1540	50462.	53432.	.114S	38451.	ul 750.	39512.	* 20177	* 1 × 0 0	* * * * * *	7550	4-00-	3300.	160.	2540.
-	345	506	507	200	204	210	215	513	514	515	516	215	516	615	075	175	225	200	575	220	527	528	554	530	155	532	555	233	4 6 6	537	538	539	240	175	346	544	243	240	125	Sak	544	550	155	225	200	224	200	145	528	5.54	2013
, p	254	255	255	650	152	100	152	252	251	251	558	454	258	256	280	200	107	107	282	203	263	205	205	502	200	200	650	200	243	267	201	267	207	102	200	264	264	200	270	270	213	270	212	276	613		274	234	275	275	512
t,	\$60C.	b1200.	1800.	71700.	23772.	1149.	22500.	1200.	11460.	5400 .	abeen.	29434.	24075.	5712.	43813.			10000.	112027	29045.	92406.	155541.	1500.	4320.	106140.	#020°	*0490*	300.	121011	6900.	1500.	11020.	8484.	14400.	54026	*1041*	- 7-200	0.	10048.	\$01a.	6050.	3600.	84115.	"060 I	.000C4	******	10000	36400.	Dunak	#5308.	5994.
-	949	450	451	452	453	224	454	457	450	454	0.48	461	200	403	101	107	00	07	0 3 4 9	470	471	472	473	074	475	476	110	010	1040	140	482	4A3	4.9.4	485	007	1 4 4 4	0 8 1	490	107	264	493	494	56.0	007	1.5.0	0.00	****	100	100	503	504
, i	230	230	e 34	234	612	234	240	240	240	240	240	240	< 4 U	112	6a1	242	102	1 5 2	200	202	242	243	243	243	243	243	244	222	240	200	244	244	245	5#2	545	500	404	246	102	672	209	250	250	253	250	623	253		25.0	254	521
t,	5126.	0414°	1211.	16450.	4054	5337.	-10456	6141.	. 67	3126.	2866.	4068.	.024a	32475.	5560.	· < 0 6 1 2	1040.	.1022	1010	6368.	4269.	34300.	BRBU.	. 0.9	120.	1040.	47940.	10000	10001	1000	720.	2820.	.0114	3708.	.0416	7760	36140	11140.	45821.	74019.	10554.	40320.	44540.	.01155	32216.	- n1452	24614.	* 1000	* 10 1 F	13462.	19614.
÷	345	594	345	340	397	398	008	401	208	40.5	404	50*	901	401	0.08	101	010	114	212		415	010	417	415	011	020	120	222	463	570	426	120	424	620	430	100	110	0.14	210	430	437	438	454	940	100	268	500	100	10.7	100	607
, p	208	208	202	204	112	211	212	212	213	214	214	214	214	213	214	512	512		112	212	212	218	21%	510	514	512	510	222	224	228	520	229	231	231	231	152	515	515	222	234	234	254	254	254	634	235	250	635	630	234	233
ţ,	31523.	592.	205%2.	Saud.	-2910C.	. 00412		1200	59100.	40630.	3600.	1500.	7500.	17Hau.	Saug.	7556.	15500.	.1200.	31754.	1185.	2156.	30074.	37440.	17100.	. 6000	2734.	.1959	110700.	*0200a	01800.	47523.	545.	32412.	22524.	5497.	- 1000 ·	Cours.	*	68460.	13566.	7305.	.1969	1360.	10207.	6500.	16441.	6454.	4500.		. 781	164.
ŗ	357	330	330	540	105	245	100	555	345	347	548	349	350	155	350	555	374	522	1210	358	159	360	501	302	363	364	365	300	105	0.047	1018	371	372	513	374	515	010	214	179	380	341	384	383	584	345	386	347	507	200	101	392
, in 1	1 60	6.9	6.9		*0		11		100	6.0	80	8.6	1.2	5 4	× T	54	8.9	80 1	0 0	0.0		0.6	10	50	55	50	5.5	5.5	5.5			01	- 10	01	03	50	~ ~		20	50	90	90	90	90	06	10	10	10	10		80
t,	120574. 1	23450. 1	22020. 1	28380. 1	4523. 1	12277. 1	- 100 I	200410	14580. 1	6240. 1	10000. 1	37800. 1	32400. 1	I .00ss1	8400. 1	7868. 1	1880. 1	\$255.1		21617 1	34200.1	25200. 1	25985. 1	B0#15. 1	48264. 1	1891. 1	24. 1	932. 1	1785. 1		1 800 1	41800. Z	22500. 2	120. 0	42180. 2	80700. 2	e 2		CIR.	2001. 2	24454. 2	154. 2	2294. 2	11823. 6	15780. 2	18420. 2	5400. 6	3600. 2	5000. 6	1020. 0	2700. 4
-	181	282	283	284	285	280	28.8	080	062	102	292	295	594	562	290	102	298	552	200	100	101	304	305	306	307	308	300	310	511	316	110	315	510	317	516	516	250	125	101	174	325	526	327	528	329	550	551	336	355	534	530

Table B.5 (Cont'd)

i	t _i	ďi	i	ti	di	i	^t i	di	i	t _i	ďi	i	ti	ďi
561	57600.	245	e17	54600.	528	673	464.	343	724	14285.	154	105	\$5013.	542
205	52400.	695	614	119700.	550	014	1710,	343	730	1511.	354	140	e 5 4 6 11 .	393
203	51180.	530	619	Taesa.	531	675	V-200*	343	751	2520.	354	101	1090.	243
204	21120.	295	020	3.	351	070	105720.	546	132	156644.	301	148	30434.	344
203	15600.	640	021	1500.	551	611	45000.	341	155	13740.	3-51	/89	· \$6.01.	142
200	46500.	291	022	52200.	332	6/8	21000.	347	134	23433.	562	190	55004.	345
207	32444.	200	023	64600.	224	674	100.	341	135	030.	305	141	114632.	244
500	141904.	10.0	1.25	5400.	334	2.61	2000.	341	730	.010	302	196	74470.	100
570	19720	E Cras	2.20	7200	111	6.2.2	0.	347	137	1155	300	143	10000.	491
571	55119.	501	621	REGOU.	245	6.83	61320	3.un	719	5200H	181	244	111503	400
512	28462.	302	628	21603.	146	DAG	6120.	348	740	147.	363	140	15/275	409.
573	10400.	502	627	1900.	335	685	20100.	SOR	741	2408.	363	747	14700.	404
574	298332.	505	030	υ.	530	585	6.	549	112	1415.	563	148	\$528.	409
575	10518.	3:15	031	3600.	330	037	7200.	343	745	37191.	384	799	150432.	412
576	15581.	505	032	3600.	530	688	35400	349	744	11142.	305	800	15400.	414
277	5930.	305	033	14400.	355	284	960.	\$49	745	21/34.	500	101	271500.	420
578	40773.	300	134	a8403.	357	690	1140.	349	740	10057.	500	902	58550.	441
514	70000.	307	r 35	30500.	3 5 7	031	3190.	349	747	2ns2n.	300	003	108020.	124
580	31200.	307	035	7200.	337	605	31500.	349	748	35543.	307	824	1324.	424
581	4200.	301	031	3000.	3 5 7	693	· 0209	349	749	15710.	300	0.05	0415.	464
285	175080.	309	0.50	37800.	358	044	154700.	353	750	10034.	368	000	8401.	254
503	52050.	510	037	12300.	330	095	124080.	353	751	81343.	370	901	420.	454
584	163650.	311	640	5100*	33A	090	1500.	353	152	11104.	370	808	50.	424
205	600.	511	041	1200.	358	697	8250.	353	753	14/01.	5/1	603	0.	424
200	840.	311	542	5100.	330	DAU	20400.	555	754	114212.	5/3	810	1/52.	424
201	01100.	112	6.3.0	30000.	330	700	4340.	323	155	51234.	175	011	32049.	929
589	1020	112	644	1000.	114	7.03	1050	11.1	157	7417	175	814	10424.	034
590	1244	412	646	120	LIR	702	SIVO.		758	6273	115	819	12460	427
591	2090	312	047	2880	5.2 M	762	8405	151	754	20762	676	815	12.	427
592	52200.	513	048	1080.	338	704	73591.	154	700	9690.	375	816	3400.	154
593	106760.	314	649	06120.	339	705	3:08.	354	701	6514.	310	e17	2020.	427
594	03000.	315	650	0.	339	706	4078.	354	762	0053.	510	618	9.	427
595	2:60.	315	051	30000.	339	707	398.	354	763	152344.	374	e19	\$50.	427
598	115920.	517	652	75000.	340	708	9962.	354	704	57508.	300	520	30352.	420
597	19500.	317	653	7200.	340	709	2255.	354	765	.5435	380	Sel.	. d15c	424
598	192600.	319	054	15300.	340	710	1900.	354	700	7440.	380	822	1472.	420
599	1180.	319	655	900.	540	711	0045.	354	767	280.	580	025	1154.	454
600	45614.	514	656	3000.	344	712	2551.	354	768	4128.	SPU	8-24	Shud.	459
001	8848.	350	657	·) .	340	713	48994*	355	709	616.	340	425	14454.	450
005	52792.	350	658	1800.	340	714	35001.	355	110	15417.	3/51	826	3421.	454
603	40797.	351	059	88200.	341	715	0322.	555	771	7002.	581	827	41215.	450
604	02.	321	660	3000.	341	715	2070.	355	115	2200.	381	020	5405.	4.51
005	37381.	321	601	1800.	341	717	147320.	359	113	1555.	541	024	5120.	431
600	40320.	130	002	3600.	341	718	2000.	554	114	20223.	371	250	53004.	432
607	19110	122	600	1400.	341	120	2000	150	115	CARAL.	14.1	031	130001	436
000	51800	146	665	7443	542	725	1394	150	777	20154	141			
610	39000.	324	603	2061	346	123	102	250	774	21232	144			
611	15000.	124	660	3600	542	723	2100	154	779	Shille?	485			
012	28800	524	0.07	75530	343	724	3500	359	7.10	5075	545			
011	35523	325	659		343	125	10590	359	731	125221	387			
614	14400	325	670	50.52	543	725	2911	359	782	129043.	390			
615	52048.	320	671	/83.	343	121	1298.	359	703	52757.	341			
010	138752.	\$27	672	982.	543	728	3815.	359	784	1942.	571			
i=	fault n	umber	, t _i = ex	ecution	time,	, d _i = d	ay of fa	ailur	е					

i	ti	ďi	í	ti	ďi	í	t _i	d _i
1	5.	1	31	23.	10	21	5.	11
2	14.	1	32	1.	10	50	30.	57
5	59.	1	5.5	672.	24	0.5	14	4.14
4	32.	2	54	189.	24	0.4	14.12	34
5	e .	2	35	м3.	26	65	2.	10
6	5%.	2	30	52.	20	60	do.	45.02
2	ć.	. 2	57	۴.	26	07	221.	40
8	25.	2	1M	1.	26	0.8	0.	42
9	2.	2	10	41.	27	64	891.	52
10	5.	5	40	7.	15	10	25.	25
11	· · ·	0	91	43.	28	71	4.	5.5
51	1.	0	42	1.	28	12	457.	50
1.5	50.	0	45	а.	24	13	60.	5.4
1.4	21.	1	44	5.	55			
15	190.	12	45	1.	28			
10	205.	12	46	10.	59			
1.7	6 .	12	47	70.	29			
1.8	5.	12	48	00.	50			
19	м.	12	49	2.	30			
20	1.	12	50	2.	50			
21	12.	12	51	3.	50			
55	36.	1.5	52	109.	51			
25	3	1.5	53	29.	32			
24	1.	15	54	88.	53			
25	74.	14	55	55.	35			
20	45.	1.4	50	27.	35			
21	230.	14	57	24.	35			
85	121.	15	58	27.	35			
29	10.	16	59	140.	\$7			
30	9.	16	60	33.	37			

Table B.6 Failure Data Set 6 of J. D. Musa

i= fault number, t_i = execution time, d_i = day of failure

Table B.7 Failure Data Set 14 of J. D. Musa

i	ti	di		į.	ti	di		i	t _i	ďi
1	111520.	3		0	65173.	63	-	31	1563300.	129
2	2074820.	27	1	1	2370.	03		32	513000.	135
5	514500.	35	1	6	1501.	0.3		55	177500.	137
14	1140.	33		9	228315.	66		34	2469000.	105
5	\$120.	5.5	2	0	51440.	01		35	1078200.	135
6	317440.	37		1	44420.	6.7		50	170700.	187
1	15420.	37	2	2	850040.	17				
8	60000.	38	2	5	Solboy.	61				
4	140160.	39	2	4	34300.	62				
10	931620.	50		5	515280.	88				
11	12240.	51	2	13	250980.	91				
12	731700.	60		1	390780.	20				
1.5	250640.	65	2	8	91260.	97				
14	2465.	63		4	1225620.	111				
15	195.	63	3	0	120.	111				

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i= fault number, t_i = execution time, d_i = day of failure

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1	ti	d _i	1	ti	di		i	ti	di	
1	8100.	5	10	600.	21	-	31	15500.	41	-
2	4800.	17	11	15060.	23		32	1800.	41	
4	900.	9	14	. 951	23		3.5	5400.	.41	
- i -	459.	4	19	500.	23		34	5900.	42	
5	430	4	20	1200.	24		35	2400.	50	
	ouñu.	11	21	300.	25		30	1200.	43	
,	2400	14	22	1200.	26		57	14510.	55	
	21:00	15	23	1200.	26		30	9000.	56	
	2100	15	24	4300.	27					
4.15	1261	11	25	IVEUD.	30					
1.5	1200	14	20	3000.	32					
12	400	18	27	600.	32					
13	9000	14	28	9600.	35					
14	A 110	20	29	6400-	3.7					
15	2400.	51	50	-100.	3.8					

Table B.8 Failure Data Set 17 of J. D. Musa

i=fault number, t_i = execution time, d_i = day of failure

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Table B.9 Failure Data Set 27 of J. D. Musa

i	ti	d _i	i	ti	ďi	i	ti	di
1	20336.	1	16	38005.	19	31	336600.	+0
2	11/76.	1	17	16200.	23	32	268140.	63
3	40933.	1.1	10	0000.	23	33	74880.	65
4	34194.	1.1	19	1000.	24	34	286200.	+5
5	17130.	2	20	10000.	24	35	25320.	65
6	148446.	2	21	220.	24	36	7080.	65
1	1495.	2	22	35500.	24	37	59820.	66
3	1030 .	2	23	81000.	25	38	87900.	67
9	15830.	2	24	643045.	33	39	76200.	84
10	21937.	10	25	47857.	34	40	89280.	69
11	2485.	12	26	194800.	35	41	1209600.	79
12	11000.	15	27	170400.	37			
13	2880.	15	28	108540+	55			
14	61182.	15	29	73800.	57			
15	4800.	18	30	1800.	60			

i= fault number, t_i = execution time, d_i = day of failure

1				~i	^u i	,	۲i	ai
	320.	1	41	640.	100		1+7200.	250
2	14540.	2	42	610.	100	56	10000.	670
3	9000.	9	45	2800.	102	65	110200.	200
4	2080.	21	4.4	119.	104	84	441800.	266
5	5700.	24	45	22080.	104	85	654200.	215
6	21800.	21	46	00054.	106	40	334560.	114
1	20409.	27	17	52153.	112	67	1458800.	290
8	113540.	37	9.8	12540.	110	BH	an720.	241
9	112137.	41	4.4	734.	114	8.9	144230.	500
10	a40.	42	50	10193.	114	90	215227.	50.5
11	2100.	4.5	51	7041.	114	91	H6400.	344
12	28195.	45	52	51555.	114	40	00040.	305
15	2173.	44	53	24515.	115	44	1414400.	304
14	1203.	44	54	290890	125	44	4160.	320
15	10865.	44	55	1280.	125	95	\$200.	120
15	4230.	45	56	22044.	127	in	199200.	. 124
1/	Bubu.	44	51	19150.	127	47	150100.	111
18	14845.	44	58	2611.	127	98	518100.	557
19	11444.	50	59	19170.	1.17	9.0	145500	8.0.8
20	5551.	51	6.0	55/94.	124	100	11100.	141
21	6553.	51	F1	42612.	155	101	265600	\$47
22	6414.	55	62	26/1000	115			2.11
25	4124.	50	63	87074	110			
24	51323.	58	64	149606	141			
25	17010.	SH	65	14400	141			
20	1890.	64	66	\$4500.	1.41			
21	5400.	69	61	19600	141			
28	62312	7.6	68	114195	148			
24	24826	16		296015	148			
50	20115.	79	70	177499.	149			
31	10.1.	14	71	214622	150			
12	14040	8.5	12	156400	154			
11	15058	85	73	155800.	150			
3.4	12111	85	74	10800	156			
15	41612	85	75	267000	159			
16	4160	87	7.5	2098811	185			
47	82040	91	77	610080	190			
3.8	13149	91	.7.0	7680	190			
10	\$425	37	70	2629662	220			
40	5833.	99	80	2946700.	254			

Table B.10 Failure Data Set 40 of J. D. Musa

i= fault number, t_i= execution time, d_i= day of failure

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i	ti	di	i	ti	ďi	i	ti	ďi
1	H6530.	2	41	15420.	50	81	453PD.	101
2	713700.	11	42	900.	50	58	335510.	105
3	071220.	17	45	45200.	50	63	35554.	100
14	61470.	18	14	380610.	53	84	251820.	1.117
5	77150.	18	45	500850.	57	85	199710.	10.9
0	2usn50.	20	36	69930.	58	86	459440.	115
1	145660.	15	07	3000.	56	01	169160.	114
8	\$5590.	21	48	990.	58	88	49900.	115
9	16560.	21	2.9	14400.	54	89	210.	115
10	9350.	25	50	2030.	50	40	90.	115
11	67500.	55	51	17910.	36	91	50070.	110
12	4540.	22	52	5670.	54	92	162100.	117
15	123219.	23	53	210060.	60	93	208350.	119
1.4	5400.	23	54	B2020.	01	4.0	· 020 .	119
15	678960.	24	55	151700.	50	95	28449.	119
10	57000.	5.9	56	57130.	63	10	150480.	151
17	45680.	50	57	\$510.	63	97	35000.	151
18	26460.	50	58	188620.	64	9.8	100.	121
19	100620.	51	39	88560.	65	99	110160.	122
20	12150.	51	60	170150.	67	100	513950.	125
23	154350.	32	61	5150.	07	101	45420.	121
22	104010.	34	50	.050895	7.0	102	Sava.	151
25	3150.	34	65	141400.	71	105	10710.	127
24	10350.	34	64	13000.	71	104	531430.	150
25	157500.	35	05	900.	71	105	400350.	154
20	66870.	36	00	92450.	72	100	300900.	137
21	245700.	3.9	67	213480.	74	107	580500.	145
219	167510.	40	68	31950.	74	108	1200.	145
29	21120.	40	69	373500.	77	109	178200.	143
5.0	224Nbv.	42	70	232020.	79	110	124380.	140
51	21020.	42	71	117900.	P 0	111	\$13470.	147
30	3780.	42	12	64260.	80	112	158940.	148
3.5	105570.	43	7.5	4500.	60			
34	2700.	43	74	253710.	Pa			
35	69540.	44	75	302940.	86			
30	62730.	45	76	71460.	87			
37	20540.	45	77	515970.	56			
38	413550.	43	78	260010.	94			
39	55170.	49	79	91920.	95			
40	41940.	49	80	673470,	100			

Table B.11 Failure Data Set SSIA of J. D. Musa

i= fault number, t_i = execution time, d_i = day of failure

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Table B.12 Failure Data Set SSIB of J. D. Musa

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0	325	333	\$ 3 9	340	540	540	341	1 1 1	1	1.5	545	220	220	141	101	147	549	550	305	507	364	309	204	511	574	313	015	105	105	205		190	340	398	348	000	412	2	0			C1 7	242	0.00	0.87	989	97.7
t,	40743.	540023.	a14760.	*4480**	.100.	5100.	13140.	1360.	1<300.	.0.55	.00255	.06111	1 40.00	1 1 1 1 1 1	- MADAN	1+0423.	110400.	120780.	1104520.	119527.	*1.006A	25160.	1 3800°	155000.	185220.	104860.		*0*000*	24180.	54200.		22420.	1200.	037800.	35700.	841600.	253140.	288360.	1000.	101200.	*0.5CO1	172440	748.00	554940.	11700.	\$ 32a80.	20100.
-	241	246	613	204	502	605	201	202	500	610	112	212	010	110	414	112	610	\$10	220	122	262	223	624	552	200	122	622	422	630	162	110	239	235	230	237	238	239	0.82	241	202	6.2	377	400	1010	208	683	250
10	234	634	234	234	234	e 35	255	230	630	202	645	100	255	1 2 2 2	25.4	*57	259	263	200	266	200	692	270	215	217	278	500	262	285	282	4 9 9	100	293	300	301	305	\$05	305	201	503	115	120	200	322	124	524	325
t,	1560.	.000	22624.	*00°	.000	21099.	35880.	54.140.	101340.	241140.	516869.	1/1500.	120700.	176420	*120601	26146.	n5423.	58020.	2100.	\$96240.	10069.	132700.	70520.	163080.	348000.	+64990.	190200.	123540.	5520°	400.	100200	15460	5220.	559140.	95280.	247360.	25220.	\$460.	186240.	* 3 * 5 1 0 1	*061652	* 020202		.000001	15220.	15300.	48500.
-	151	152	153	154	155	150	151	158	124	100	101	104	102		441	1.41	104	109	011	171	112	173	174	175	170	111	178	179	180	181	100	194	185	180	181	186	1631	140	161	142	145	301		101	198	199	200
di	145	143	1 45	152	153	160	160	101	108	168	169		113		174	174	180	180	161	186	201	202	211	213	213	514	216	220	220	222	224	224	229	224	229	229	224	622	650	230	630	152		210	244	234	2.34
t,	ealu20.	35480.	154040.	Sabab0.	130560.	556580.	16740.	539#2U.	.05499.	15920.	04/40.	.0015	1 1 1 1 1 1	* 1000	*0.020		507840.	55740.	82580.	409020.	1167840.	69120.	668700.	160320.	11220.	22380.	219300.	266340.	2100.	157740.	.000000	*05001	346652	1080.	9000	6600.	120.	240.	240.	68700.	1040.	11340.	*0.56.56.2	.004	1260	540.	1200.
-	101	102	103	104	105	106	101	108	104	110	111	231	113		112	111		611	120	121	122	123	124	125	126	121	128	129	130	131	152	123	511	136	137	138	139	140	141	142	145	144	6.5.1	140	14.8	149	150
d.i	15	25	53	53	50	20	28	65	41	19	59	50	50		00	12	22	15	15	75	16	10	16	10	10	11	18	00	80	84	58		88	0.6	06	0.6	95	101	102	102	105	104	C01	201	101	125	133
ti	208020.	5234U.	40.530.	24820.	257334.	180.	1 J 3 5 6 0.	155920.	187100.	18420.	170040.	52540.	10200.		*001021	16000	1480.	163020.	13980.	9120.	29400.	9300.	1200.	3600.	1500.	19990.	75600.	159600.	.000.	292083.	140040.	*00000*	1200.	156320.	3000.	34240.	244460.	446460.	+00966	180.	12300.	.002200.	102160.	211540.	S 36NMO	46920.	790260.
-	15	52	5.5	15	55	26	15	23	15	09	19	20	50	7 5	C 0	004	99	0.0	10	11	12	73	74	15	16	11	18	61	90	81	82	50	1 1	60	87	8.9	8.5	9.0	16	26	56		5.6	40	10	56	100
	2																																														
d,	-	-	-	1	v	2	~	2	~	3	ę				0 -				8	*	10	14	15	15	17	21	53	24	54	54	52	20	NY C	24	50	5.8	30	30	34	34	20	23	2	Na a	40	11	6.0
tj	5400.	10200.	24069.	.000B	\$ \$900.	16500.	0300.	01540.	5400.	51780.	1140.	6500.	-0012	10201	.04140	. 2000 ·	15000.	14400.	30,000.	131460.	5870U.	273480.	015ta	25100.	175860.	310020.	148920.	19680.	12720.	4920 ·	70320.	. 1540.	- 14401C	16540.	1550.	44160.	20100.	48300.	210060.	\$0180.	168030.	175580.	.02005	.09115	146200	15900.	185040.
	-	~	2		5	0	~	80		10	-	15	13	2 :			1.1	19	20	21	22	23	54	25	26	27	58	53	30	15	32	55	16	36	37	34	39	40	10	42	43	1 1 1	C	0.0		0.0	50

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Table B.'2 (Cont'd)

10	629	150	633	0.54	015	638	0 58	658	639	639	639	639	0 5 0	140	589	100	150	559	\$2*	055	\$59	020	050	656	000
t,	142029.	14440.	113040.	Squod.	78500.	146380.	25900.	5520.	\$0400°	1004.	2081.	25520.	41520.	11040.	1+3144.	106100.	245700.	04000	123400.	57000.	21360.	22140.	20100.	12180.	268260.
-	1<5	552	553	524	355	356	357	358	354	350	301	362	303	204	305	300	567	300	364	370	371	512	373	374	375
d,	610	012	012	612	515	514	114	617	017	\$19	619	619	019	624	620	620	120	523	624	625	625	626	626	627	627
t,	1320.	156540.	3900.	45860.	68340.	13440.	31360.	123120.	39180.	130620.	4960.	9360.	4320.	49740.	*0254	7560.	55500.	30900.	140940.	29150.	21440.	33420.	20700.	50040.	10740.
-	326	527	328	329	552	331	332	333	334 .	\$35	336	537	338	339	340	105	342	543	344	345	346	347	348	549	150
d,	546	548	549	550	155	155	552	552	554	155	558	561	571	576	576	576	582	584	145	604	604	607	507	019	610
t,	91740.	1569.	61380.	41520.	70000.	4800.	68040.	15660.	130860.	213060.	73260.	242760.	722440.	423360.	420.	1440.	340860.	128700.	940500.	226980.	21900.	157200.	360.	225600.	: 9860.
	501	502	303	304	305	305	307	308	509	310	311	312	513	519	315	316	317	318	319	320	321	322	323	324	325
d,	501	505	505	503	508	512	512	513	514	515	510	516	515	526	527	527	125	531	534	534	534	105	543	546	546
t.	7A120.	304060.	20003.	209700.	18000.	195000.	2700.	79560.	A5450.	£0100.	77160.	4000	5640.	016120.	57120.	16120.	180.	203820.	210000.	2640.	600.	523500.	147420.	199740.	25800.
-	276	277	270	612	280	281	262	283	284	285	286	187	288	284	062	102	292	295	294	295	296	297	298	544	100
d.	447	107-	600	454	454	46.0	400	470	010	511	475	415	415	414	482	284	284	285	480	268	560	910	855	200	2005
t,	1560.	28200.	39050.	511580.	315460.	131460.	37260.	150480.	23440.	400080.	140.	2011.	.040.	202200.	219160.	3960.	45520.	504020.	11820.	447660.	249460.	145040.	26940.	142920.	12100
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i= fault number, t_j = execution time, d_j = day of failure

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Ţ.		*0912-22	660.	2400.	2760.	.0880.	*0.7 ×		81960.	228560.	100020.	165560.	. 62860.	* 0+0 <c2< td=""><td>. 140.</td><td>1940.</td><td>430000.</td><td>12000.</td><td>40020°</td><td>10660.</td><td>1063.</td><td>¢100*</td><td>. 10201 .</td><td>.02020.</td><td>805170.</td><td>10780.</td><td>4/100.</td><td>110520.</td><td>640.</td><td>266760.</td><td>55400.</td><td>10800.</td><td>114100.</td><td>452500.</td><td>17400.</td><td>>43060.</td><td>.040141</td><td></td><td>111140</td><td>+1960.</td><td>1400.</td><td>442060.</td><td>22260.</td><td>241500.</td><td>.0500 ×</td><td>144200.</td><td>37260.</td><td>122580.</td><td></td><td></td><td></td></c2<>	. 140.	1940.	430000.	12000.	40020°	10660.	1063.	¢100*	. 10201 .	.02020.	805170.	10780.	4/100.	110520.	640.	266760.	55400.	10800.	114100.	452500.	17400.	>43060.	.040141		111140	+1960.	1400.	442060.	22260.	241500.	.0500 ×	144200.	37260.	122580.			
ŗ		222	623	428	622	230	100	233	250	235	630	152	010	000	107	242	643	244	245	245	247	643	190	152	252	253	254	\$52	659	1620	000	200	201	262	245	101	202	147	842	264	270	271	212	213	274	275	615	212			
ďį		101	561	551	196	144	201	203	245	210	112	212	215	122	227	229	254	234	234	242	070	600	270	282	290	\$0.9	300	310	516	010	116	320	330	332	354	334	244	118	114	139	540	342	343	344	345	543	544	350	151	156	
t,	210128	18940	14200.	6 200°	141280.	Poulog.	146280.	2880.	43200.	.040404	30700.	. 0255001	207440	810840.	437.60.	130920.	200260.	23150.	. 25020.	.000165	112410.	. 346460	107400	.061529	507300.	.01004101	1080.	30540.	.00100		13050.	142140.	510780.	161500.	151580.	*0010	228900	a \$ 20.	9300.	75720.	13060.	118020.	33900.	29940.	86440°	116960.	8400.	5 5000 ·	4 3 W 2 U -	117940.	
į	140	170	1/1	172	175	175	176	111	178	179		101	181	144	185	185	101	166	681	140	101	101	194	561	196	197	198	661	100	202	203	204	205	200	202	000	210	2112	212	213	214	512	216	217	618	219	220	122	222	224	1000
d,	130	130	150	130	130	134	133	134	134	134	1 1 1	137	139	139	139	140	140	140	0.01	0.01	1 4 4	501	146	147	671	150	152	151	155	155	158	158	159	165	541	166	113	173	173	177	178	178	178	178	179	160	100		195	061	
t,	1906.	1680.	7980.		25420.	99360.	45.560.	62040.	24350.	5420.	#10AD	128040.	154540.	1+900.	5750.	41760.	· 015	240.	.000	Sao.	305040.	102480.	50H40.	86980.	130980.	172140.	144000.	14100.	155820.	5100.	242820.	33840.	34140.	419968.	24520	57720.	454200.	\$6060.	3780.	226440.	13560.	160.	20760.	10340.	#5660.	11//20.	10000	*1000*	19560	319260.	
+	113		115	011	891	119	120	121	122	123	521	120	127	128	129	150	151	196	123	145	136	137	139	139	140	141	142	621	105	146	147	148	601	150	251	153	154	155	156	151	158	159	160	191	162	103	504	144	167	166	
di	50	20	15	20	55	54	15	04		10	62	64	50	20	4.9	50	200	20	2.0	84	85	16	93	56	96	46	101	101	103	107	101	801	211	211	115	113	119	611	122	123	125	140	120	121	101	101	121	127	130	130	
t,	240.	*00°*	15080.	1140.	02150.	54620.	112140.	202930.	*	41880.	3 4 5 0 0 .	115360.	66960.	178740.	2540.	.0010	*001		890540.	1020.	13550.	284407.	85140.	177500.	45360.	370073	172140	950.	23700.	190020.	55260.	48550.	204900.	NA 700	60.	-	337920.	31320.	245760.	55430.	191260.	-08692°	120.	114540.	.0811	15.00	1200.	2820.	51460.	3446	
i	25	20	***	19	24	53	0.0	0.0	2.4	20	0.0	10	11	12			34	11	78	44	80	81	82	83	500	100	87	88	69	0.6	16	26	22	10	96	1.6	96	66	100	101	102	501	104	201	100	108	601	110	111	112	
, i p	1	~	~ ~	2	\$	~		• •	9		5	5	Ð					a	13	11	14	15	15	15	200	12	22	50	27	58	30	32	10	35	35	58	30	0.0	41	1 17	7 1	51		54	50	50	07	10	46	50	
t'i	.0002	.00010	15620.	<2000.	13580.	4520.	2100.	*0*0*0	5100.	28800.	59620.	. USel	.02565	.00000	10500	\$25.00 ·	2280.	*2100.	107440.	42840.	145920.	43380.	43580.	.02020.	.00000	44400.	110220.	209100.	*0880°	8580U.	98820.		52740.	54960.	32100.	129660.	110760.	15000.	57260.	C4150.	16740		*0000*	10320.	2280.	28800.	15960.	52140.	28920.	52920.	
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i= fault number, t_i = execution time, d_i = day of failure

Table B.14 Failure Data Set SS2 of J. D. Musa

i	ti	di	i	ti	di	i	ti	di	i	ti	di	i	ti	di	
	25100		0.1	960.	199	81	525360.	347	121	3160.	447	161	\$55600.	502	1
	119.80		8.2	100910	201	82	253620.	350	155	59680.	447	162	56340.	543	
č .	137600.	2	41	678500	210	83	70800.	351	123	119220.	900	163	\$76100.	507	
2	300.	-	8.0	11100	210	Ba	936720.	161	124	7500.	9 8 8	164	47430.	507	
4	351030	. 7	05	1200	210	85	132480	165	125	192840.	452	165	1337220.	559	
2	151920.	17	4.5	1200.	210	84	74200	366	120	20580.	452	166	eau290.	500	
0	1579800.	37	40	307640.	214	8.7	110720	370	127	755000.	461	167	157#7#0.	578	
1	374280.	42		1150374	231	Pa	161580	172	128	5944.	461	168	588720.	584	
	111000.	45	40	1334270.	312	80	11540	172	129	393240.	466	169	195930.	586	
9	113460.	45		41500.	313	0.0	3200	172	150	157140.	468	170	2790.	580	
10	625300.	25	20	2760.	236	90	2896 20	176	131	1114663.	482	171	2520.	586	
11	351950*	56	51	231000.	232	0.7	13500	376	132	78660.	483	172	1040.	586	
15	540150.	59	25	227920.	230	92	1980	176	133	6360.	483	173	248450.	588	
15	74160.	60	23	60700.	219		c99.0	177	134	358140.	487	179	21690.	588	
14	214560.	63	24	1340.	234	0.0	79.20	377	115	41700.	488	175	347130.	591	
15	225000.	66	55	501A30.	245		20.00	377	115	131700.	490	176	108370.	590	
10	2060.	66	56	15560.	245	76	2000.	177	117	1000/04.	091	177	13680.	594	
17	539340.	73	57	1106100.	250		10300.	100	134	7920.	091	178	020890.	599	
18	203000.	77	58	1500*	220	48	1036360.	370	119	212420.	495	179	119340.	600	
19	900.	17	59	363780.	500	99	183660.	572	100	102300	497	140	219600.	500	
20	557220.	85	60	664A0.	205	100	1484160.	010	141	18180	497	181	12870.	602	
51	667440.	92	61	4350.	265	101	14140.	410	147	115784-	502	182	1524330.	615	
25	319800.	99	59	16020.	265	105	554100.	617	102	71180.	503	1.4.3	151800.	019	
23	124560.	100	63	109440.	267	103	1080.	417	103	63P4.	501	184	114930.	050	
24	389520.	105	64	138560.	510	104	23400.	417	105	010560.	505	185	95760.	621	
25	1680180.	125	65	720.	510	105	517740.	962	105	08540	500	186	240570.	623	
26	713220.	134	66	2700.	270	106	54250.	425	147	96340.	510	187	A75360.	854	
27	1056020.	147	67	53880.	271	107	54660.	959	147	214804	Et a	188	70940.	629	
85	193700.	149	68	10950.	271	108	37800.	021	140	73440	EIE	189	1011350.	641	
29	521340.	155	69	1806980.	503	109	30340.	a27	144	EE080.	515	190	15210.	601	
30	4980.	155	70	84900.	294	110	78500.	a28	150	334004	510	191	1566270.	A55	
31	2065920.	180	71	301200.	298	111	534660.	a 3 a	151	5-07704	510	103	8010.	455	
32	143540.	182	12	020.	298	112	54960.	935	152	3041201	522	146			
33	522660.	188	73	1020.	298	113	56580.	936	153	3000.	522				
34	572160.	196	79	1156140.	312	114	1951A0.	034	154	325295	522				
35	119920.	198	75	733500.	321	115	68940.	439	155	34440.	763				
30	720.	198	76	435540.	327	116	70380.	040	150	91500.	524				
37	9900	198	77	507000.	333	117	401700.	0.05	157	300.	524				
38	1980.	198	78	106080.	335	118	87240.	446	158	220300.	531				
39	3060	198	79	204300.	337	119	32520.	446	159	0374104	530				
0.0	81900	199	80	232140.	300	120	40920.	447	160	184020*	230				

i= fault number, t_i = execution time, d_i = day of failure

i = fault number, $t_j = execution time$, $d_j = day of failure$

, i	500	105	205	205	205	505	205	215	2010	100	\$23	530	537	508	545	200	2	500	100	455	593	593	564	564	268	200	201	274	578	595	585	585			209	616	616	623	623	621	929	629	6.53	0.00	159		254	454	451	
t,	522240.	66000.	a3500.	2040.	. Drc.	226320.	.000125	*000100*	.00.000	10201	940.	\$12760.	819240.	#01660.	160380.	71640.	*024501	.0505	11100	\$47900.	A89400.	11520.	23850.	:5870.	123030.	26010.	.00261		198360.	623280.	3330.	7290.	.001/2	100800.	109890.	1615560.	14940.	680760.	25220.	376110.	181840.	64320.	465190.	1507370.	155120.		122110.	21960.	163600.	
į	225	226	227	228	529	530	162	110	010	515	216	237	238	510	240	241	202	503	222	206	247	248	229	250	152	252	653	255	955	257	258	652		242	263	269	265	266	267	268	264	270	112	212	213	245	276	277	278	
, i	320	120	528	532	333	336	530	114	114	114	339	140	105	346	347	348	0.5	920	150	051	150	554	354	355	356	357	205	16.8	377	383	383				190	190	398	398	398	565	399	500	600	202	111		850	650	090	
tj	840.	*nt5	Sequer.	332280.	-00105	1090000	* ****	1084	11580-	2160.	192724.	e78442	843604	378120.	5850U.	* 78 3 4 *	*****		1540.	3180.	5700.	226564.	9840.	e9060.	68880.		1005202	3402231	794968.	505680.	544204	2190204		6240.	1.00050	424.	667320.	1201	7200.	100049	26420.	447620.	*C25×55	100 100 V	770580	- 000 d	1156240.	907140.	58500.	
-	169	170	171	172		541	176	177	178	179	180	141	182	183	787			101	0.81	190	161	192	193	194	561	001	108	144	200	201	202	203	205	506	207	208	209	210	211	212	512	112	512	610	214	610	220	221	222	
, ⁱ	222	252	229	229	236	5.0	110	214	236	237	238	238	241	242	202	292	5.00	243		208	548	249	250	524	262	263	246	245	94.	266	271	272	102		585	296	286	286	287	242	162	162	10-	152	142		114	512	119	
t,	500.	*005	521252.	.050.	*019016	1020.	-071	180	600.	53760.	82940.	180.	273000.	SORRO.	840.	1140.				1920.	16460.	77040.	79760.	738140.	197000.	100800.	AA160	27540.	55020	120.	296796.	.08105		106200.	480.	117360.	6480.	60.	97940.	398580.	391380.	180.	190.	* D D 2	. UPC	240000	847080.	26460.	349520.	
-	113	114	115	110	111		120	121	122	123	124	125	126	121	128	521		101	111	134	135	136	137	138	139	0 7 1	101	191	144	145	146	147	001	150	151	152	153	154	155	156	151		154	0.1	101	201	164	165	166	
d,	101	111	116	120	100	101	130	101	101	101	101	144	1 4 4	57		147		150	155	155	100	103	150	100	00	001	172	172	173	174	174		141	181	186	197	189	681	194	194	0.1	511	202	202	202	20.8	210	210	215	
t,	351901.	309230.	330220.	848640.	.021	74140	262500.	879300.	100.	8160.	180.	237920.	120.	.00401	12960.	.005	* C24 20*	188000 .	56280.	a20.	alasta.	240750.	206640.	4740.	.01104	500°	012080.	1001	87600.	48240.	41930.	510612.		000	240300.	73740.	169800.	-	302290.	3360.	* 0 t 1 2	82260.	*0265CS		*05/01	a lake	166740.	*00*	376110.	
-	22	es.	59	0.0		20		\$9	99	19	69	69	61		21		24	12	11	78	19	0.0	10	20				87	88	89	00	10	20	94	50	96	16	96	66	100	101	201	501		401	101	108	109	110	
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t,	107400.	17220.	180.	32880.			333720.	17820.	40460.	18/80.	.049	.000	19860.				180	17260	2100.	72050.	258704.	480.	21900.	#78620.	.00100	*0100	688460.	2220.	758880.	166620.	6290.		14700.	3420.	2520.	162480.	520320.	96720.	418200.	a 54760.	* no15bc	. 0360.	*****	****	2220	1080	157340.	91860.	22800.	02020
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Failure Data Set SS3 of J. D. Musa

Table B.15

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Table 8.16 Failure Dat	
Table 8.16 Failure Dat	

, i	590	284	1880	100	505	508	503	506	503	503	504	605	503	510	115	236	517	520	520	521	224	524	528	550	155	555	555	557	558	578	519	595	586	610	615	010					
t,	SURIAD.	\$100.	£77900.	636420.	\$75340.	222360.	A6580.	2760.	2820.	1560.	a20.	1200.	1440.	107040.	31400.	304260.	\$9100°	261960.	21720.	68780 ·	262380.	a560.	131020.	1722300.	47520.	301170.	1020.	158000.	113310.	1613550.	.00120	446340.	+2520 *	1000620.	a20060.	245280.					
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i= fault number, t_j = execution time, d_j = day of failure

Appendix C

Statistical Analysis Systems

The BMDP biomedical computer program is a system developed at UCLA to provide an easily used set of statistical algorithms [C.1]. Its various modules deal with such topics as:

- 1. Data screening and description;
- analysis of variance;
- regression; and
- multivariate analysis.

Two modules from the BMDP system were utilized to fit the fault discovery and fault probability expressions introduced in Section 3.

<u>PAR</u> <u>Derivative-free Nonlinear Regression</u>: PAR computes least square estimates of parameters in nonlinear regression. The program is used with regression functions for which analytical expressions for the derivatives are not provided. An iterative pseudo-Gauss-Newton algorithm is used to compute the parameter estimates. Case weights and inequality constraints on arbitrary linear combinations of parameters can be specified and parameters can be held fixed at initial values. Parameters can also be estimated by maximum likelihood.

In general, this program minimizes the weighted residual sum of squares

$$RSS = \sum_{i=1}^{n} w_i [y_i - f(s_i, p)]^2$$
(1)

subject to linear constraints

$$c_{g}(p) = b_{g1}p_{1} + \ldots + b_{gm}p_{m} - b_{g} = 0$$
 (2)

where

s_i = set of t independent variables (x_{i1},x_{i2},...,x_{it}) for the ith case, u_i = dependent variable for the ith case, w_i = weight for the ith case, n = set of nonzero weighted cases used, p = set of m parameters (p₁,...,p_m), f = function to fit.

Step 1

The constraints (if any) are solved in terms of $m_1(<k,<m)$ of the parameters. For simplicity of notation we assume that these m_1 parameters are p_1, \ldots, p_m .

$$P_{g} = \sum_{j=1}^{m} b_{gj} p_{j} + b_{g} .$$
(3)

Redundant constraints are ignored.

Step 2

The solution to the least squares problem is obtained through iterations, described in Steps 2-7. At each iteration the model is linearized at the previous value of $p = (p_1, \ldots, p_m)$ to

$$\widetilde{y}_{i} = \sum_{j=1}^{m} z_{ij}(\widetilde{p}_{j}) + \widetilde{e}_{i}$$
(4)

where the p,'s are the parameter values to be estimated.

$$\widetilde{y}_{i} = y_{i} - f(x_{i}, p)$$
(5)

and

$$z_{ij} = \frac{\partial f(x_i, p)}{\partial p_j}$$
(6)

If there are any linear equality constraints,

$$\widetilde{y}_{i} = y_{i} - f(x_{i}, p) + \sum_{\substack{k=m_{1}+1}}^{m} \frac{\partial f(x_{i}, p)}{\partial p_{k}} [p_{k} - \sum_{\substack{j=1}}^{m_{1}} b_{kj} p_{j} - b_{k}]$$
(7)

and

$$z_{ij} = \frac{\partial f(x_i, p)}{\partial p_j} + \sum_{\ell=m_1+1}^{m} b_{\ell j} \frac{\partial f(x_i, p)}{\partial p_{\ell}}$$

$$j = 1, \dots, m_1$$
(8)

Step 3

The program forms the matrix

$$A = \begin{bmatrix} A_{11} & A_{12} \\ \\ A_{12} & A_{22} \end{bmatrix}$$
(9)

(stored in lower triangular form) where $\rm A_{11}$ is m x m, $\rm A_{12}$ = $\rm A_{21}$ is m x 1, and $\rm A_{22}$ is 1 x 1, with elements

$$\mathbf{n}_{kj} = \sum_{i=1}^{n} \mathbf{w}_i \mathbf{z}_{ik} \mathbf{z}_{ij}$$
(10)

where

$$Z_{i,m+1} = y_i$$
.

If there are linear constraints, m becomes m_1 .

Step 4

Using the Gauss-Jordan algorithm for matrix inversion, the diagonal elements of A_{11} are pivoted on in a stepwise manner. Let $A = (a_{ij})$ be the pivoted matrix A. At each step the index of the pivot element is the one that maximizes $\widetilde{a}_{rr}/a_{rr}$ among all r such that

 $1 \le r \le m$ and r has not been used previously as a pivot index $\widetilde{a}_{rr}/a_{rr} > T$ where T is the tolerance.

Note that $1 - \tilde{a}_{rr}/a_{rr}$ is the squared multiple correlation of "variable" Z with previously entered Z "variables."

Step 5

Let \tilde{p}_r be the provisionally updated value of p_r , for r = 1, ..., m. (If there are linear constraints,

$$\widetilde{p}_{\ell} = \sum_{j=1}^{m_1} b_{\ell j} \widetilde{p}_j + b_{\ell}$$
(11)

for $\ell = m_1 + 1, \dots, m_n$) The updated values are then defined by

$$\hat{p}_{j} = p_{j} + d(\tilde{p}_{j} - p_{j})$$
 (12)
 $j = 1, ..., m$,

where $d(o \le d \le 1)$ is the largest value such that maximums and minimums for the parameter values are satisfied. If there are linear constraints then

 $\hat{p}_{\ell} = \sum_{j=1}^{m_1} b_{\ell,j} \hat{p}_j + b_{\ell}$ $\ell = m_1 + 1, \dots, m$ (13)

Maximum and minimum values for these $\boldsymbol{p}_{\scriptscriptstyle 0}$ are not necessarily satisfied.

Step 6

A new value for the residual sum of squares RSS is computed. If the new RSS is larger than the previous RSS, the step size d is halved and the p_j are recomputed. This step halving is continued until the value of RSS is not greater than its value on the previous iteration or until the maximum number of increment halvings is reached.

(14)

Step 7

The program returns to Step 2 and iterations are continued until either a maximum number of iterations specified is reached or until

$$|RSS^{(N+1)} - RSS^{(N)}|/RSS^{(N+1)} < C$$
.

After each iteration the parameter values and residual sum of squares are printed. After the last iteration the program reports the asymptotic correlations and standard deviations for the estimated parameters. For each case after the last iteration, PAR lists the predicted and observed values versus selected variable values, residuals versus selected variables, and normal and detrended normal probability plots of residuals can be requested.

<u>P1R Multiple Linear Regression</u>: P1R computes a multiple linear regression equation on all data and on groups or subsets of the data; equations with or without an intercept can be chosen. If a grouping variable is specified to form groups, homogeneity of regression coefficients across groups is tested. It is also possible to specify case weights.

In general the following steps are followed in P1R:

Step 1

The weighted means and covariance matrix (c) are computed. Actual minimums and maximums for each variable are also determined.

$$\bar{\mathbf{x}}_{i} = \frac{\sum_{\ell}^{\Sigma \mathbf{w}} \mathbf{x}_{i\ell}}{\sum_{\ell}^{\Sigma \mathbf{w}} \mathbf{x}_{\ell}} \qquad \mathbf{c}_{ij} = \frac{\sum_{\ell}^{\Sigma \mathbf{w}} (\mathbf{x}_{i\ell} - \bar{\mathbf{x}}_{i}) (\mathbf{x}_{j\ell} - \bar{\mathbf{x}}_{j})}{\frac{n-1}{n} \sum_{\ell}^{\Sigma \mathbf{w}} \mathbf{x}_{\ell}}$$
(15)

where

n = number of nonzero weighted cases to be used,

w = weight for case &

 x_{io} * value of the ith independent variable for case ℓ .

Step 2

After all data have been read, the standard deviations are computed,

$$S_i = \sqrt{C_{ii}}$$
 (16)

(17)

The means, standard deviations, minimums and maximums, and (if requested) the correlation and/or covariance matrices are printed.

Step 3

The regression intercept and coefficients are estimated. The general form of the equation is

$$y_{g} = a + b_{1}x_{1g} + b_{2}x_{2g} + \dots + b_{q}x_{qg}$$

where

y = the dependent variable = x_d ,

- a = the intercept,
- x = independent variable,
- b = regression coefficient,
- q = the number of independent variables used.

The regression coefficients are determined by stepwise pivoting the covariance matrix. No variable is pivoted whose squared multiple correlation with previously pivoted variables exceeds 1 - tolerance.

$$ij = \frac{\frac{\sum w_{\ell} \times i_{\ell} \times j_{\ell}}{\sum w_{\ell}}}{\sum_{\ell} w_{\ell}}$$
 (18)

Step 4

The intercept (a) and each regression coeifficient (b,) are computed

$$a = \overline{y} - \Sigma b_{i} \overline{x}_{i} = \overline{x}_{d} - \Sigma b_{i} \overline{x}_{i}$$
(19)

$$b_{i} = c_{id}$$
 (20)

For all data and all requested groups, the output includes mean, standard deviation, minimums and maximums, multiple R, and standard error of estimates for each variable; an analysis of variance table consisting of regression and residual sum of squares, degrees of freedom and mean squares; F statistic and probability for the regression equation; and the regression coefficients, their standard errors, and t statistics and probabilities. The covariance or correlation matrix; scatter plots, normal and detrended normal probability plots of residuals, and partial residual plots; residuals, predicted values and data can be requested for each case.

Reference

C.1 W. J. Dixon and M. B. Brown, ed., BMDP-79 Bio-medical Computer Programs P-Series, University of California Press, Berkeley, 1979.

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