

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)	
)	
THE CONNECTICUT LIGHT AND POWER)	
COMPANY,)	
THE HARTFORD ELECTRIC LIGHT COMPANY,)	DOCKET NO. 50-245
WESTERN MASSACHUSETTS ELECTRIC)	
COMPANY, AND)	
NORTHEAST NUCLEAR ENERGY COMPANY)	
)	
(Millstone Nuclear Power Station,)	
Unit 1))	

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

Northeast Nuclear Energy Company et al, (licensee) is a holder of Provisional Operating License No. DPR-21 which authorizes the operation of the Millstone Nuclear Power Station, Unit No. 1 (the facility) at steady-state power level not in excess of 2011 megawatts thermal. The facility is a boiling water reactor (BWR) located at the licensee's site in Waterford, Connecticut.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and

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schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed between July 1, 1981 and March 1, 1982. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently, on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff had proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

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III.

Northeast Nuclear Energy Company responded by letters dated April 16, June 4, 10 and 24, and December 30, 1982 and February 11 and 28, 1983. In these submittals, Northeast Nuclear Energy Company confirmed that most of the items identified in the March 17 letter have been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitment or status were developed by the staff from the Generic Letters and the licensee-provided information.

There are several items in Generic Letter 82-10 that, as noted in Attachment 2, have licensee schedules which are yet to be determined. These items are, therefore, not included in this Order. The licensee considers some of the items addressed in this Order to be completed or to require no modifications. These items are so noted in Attachments 1 and 2. The staff's evaluation of the licensee's delays for implementation of the remaining items is provided herein.

The licensee's schedule for a few items does not meet the NUREG-0737 implementation dates. The licensee has identified other plant modifications to compensate for delays in completing NRC-TMI requirements, unavailability of qualified components, and necessity for plant outage and lost-power production as some of the reasons for delaying the implementation of these modifications. The licensee has proposed to complete these modifications in accordance with the attached schedule.

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The staff has not completed the evaluation of item II.E.4.2 part 7, "Isolation of Purge and Vent Valves on Radiation Signal," or item II.F.1.6, "Continuous Indication of Containment Hydrogen Concentration." These items will be considered later for a separate order after final resolution, and therefore are not included in this Order.

We have reviewed the licensee's submittals and determined that the proposal is an integrated approach to satisfy all the requirements as soon as practical. We find the proposed schedule delay to be acceptable considering costs and interim measures that are being taken by the licensee.

Since the post-TMI requirements are being imposed to improve the level of safety of nuclear power plants, the delay in schedule for implementation of certain items identified in the attached tables does not detract from the level of safety existing at the Millstone Station, Unit 1. We find that the licensee has made a diligent effort to satisfy all the proposed requirements and the proposed schedule is acceptable.

In view of the foregoing, I have determined that the commitments are required in the interest of public health and safety and should, therefore, be confirmed by Order.

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Accordingly, pursuant to Section 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall:

Implement and maintain the specific items described in the Attachments to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachments.

IV.

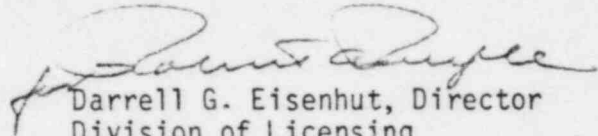
The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR A HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,
this 14 day of March,
1983.

Attachments:

1. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-05
2. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-10

MILLSTONE STATION, UNIT 1

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.3.1	Simulator Exams	10/01/81	Include simulator exams in licensing examinations	Complete
II.B.2	Plant Shielding	01/01/82	Modify facility to provide access to vital areas under accident conditions	Complete
II.B.3	Post-Accident Sampling	01/01/82	Install upgrade post-accident sampling capability	April 1, 1983
II.B.4	Training for Mitigating Core Damage	10/01/81	Complete training program	Complete
II.E.4.2	Containment Isolation Dependability	07/01/81	Part 5-lower containment pressure setpoint to level compatible w/normal operation.	Complete
		07/01/81	**Part 7-isolate purge & vent valves on radiation signal	Under evaluation by the staff
II.F.1	Accident Monitoring	01/01/82	(1) Install noble gas effluent monitors.	*
		01/01/82	(2) Provide capability for effluent monitoring of iodine	*
		01/01/82	(3) Install incontainment radiation-level monitors	Complete
		01/01/82	(4) Provide continuous indication of containment pressure	Complete
		01/01/82	(5) Provide continuous indication of containment water level.	Complete
		01/01/82	** (6) Provide continuous indication of hydrogen concentration in containment.	Under evaluation by the staff.

*End of outage prior to cycle 10 currently scheduled for Spring 1984.

**Not part of Confirmatory Order.

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)
11.K.3.15	Isolation of HPCI & RCIC Modification	07/01/81	Modify pipe break detection logic to prevent inadvert- ent isolation	N/A
11.K.3.22	RCIC Suction	01/01/82	Modify design of RCIC suction to provide automatic transfer to tanks	N/A
11.K.3.24	Space Cooling for HPCI/RCIC	01/01/82	Confirm adequacy of space cooling for HPCI/RCIC	N/A
11.K.3.27	Common Reference Level	07/01/81	Provide common Reference Level for Vessel Level Instrumentation	Complete

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-10

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)
I.A.1.3.1	** Limit Overtime	10/01/82	Revise administrative procedures to limit overtime in accordance w/NRC Policy Statement issued by Generic Ltr. No. 82-12, dtd 6/15/82	Under evaluation by the staff NNECO letter dated 10/01/82
I.A.1.3.2	** Minimum Shift Crew	To be superseded by Proposed Rule.	To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units	To be addressed when Final Rule is issued.
I.C.1	** Revise Emergency Procedures	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be determined
II.D.1.2	RV and SV Test Programs	07/01/82	Submit plant specific reports on relief valve and safety valve program.	Complete
II.K.3.18	ADS Actuation	09/30/82	Submit revised position on need for modifications	Complete in accordance with NNECO submittal 02/22/83
II.K.3.30 & 31	** SBLOCA Analysis	1 year after staff approval of model	Submit plant specific analyses	To be determined following staff approval of model
III.A.1.2	** Staffing Levels for Emergency Situations	Superseded by 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be Determined
III.A.1.2	** Upgrade Emergency Support Facilities	" " "	" " " "	" " "
III.A.2.2	** Meteorological Data	" " "	" " " "	" " "
III.D.3.4	Control Room Habitability	To be determined by licensee	Modify facility as identified by licensee study.	12/31/84

** Not Part of Confirmatory Order.