

NOTICE OF VIOLATION

Northern States Power Company
Prairie Island Nuclear Generating Plant
Units 1 and 2

Docket Nos. 50-282; 50-306
License Nos. NPF-42; NPF-60

During an NRC inspection conducted on April 18-22 and May 2-6, 1994, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, the violation is listed below:

Prairie Island Technical Specification 4.2.A.2 states inservice testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(f), except where specific written relief has been granted by the NRC.

- A. ASME Code, Section XI, IWP-3100, "Inservice Test Procedure," states that, "The resistance of a system shall be varied until. . .the measured flow rate equals the corresponding reference value."

Contrary to the above, as of May 2, 1994, the test procedures for the residual heat removal (RHR) pumps did not vary the measured flow rate to equal the corresponding reference value.

- B. ASME Code, Section XI, IWV-3427, "Corrective Action," states "Valves with leakage rates exceeding either the values specified by the Owner, or those rates given in IWV-3426 shall be replaced or repaired."

Contrary to the above, as of May 2, 1994, no corrective actions were taken when the containment sump isolation valves exceeded the leak rate limits established in the local leak rate test procedures.

- C. ASME Code, Section XI, IWV-3522, "Exercising Procedure," states "Check valves should be exercised to the position required to fulfill their function. . . ." The IST program identified the refueling water storage tank to RHR pump suction check valves (SI-7-1 and SI-7-2) have an open safety function.

Contrary to the above, as of May 2, 1994, the test procedures for check valves SI-7-1 and SI-7-2 contained inadequate acceptance criteria to exercise the open safety function.

This is a Severity Level IV violation (Supplement I.D.3).

Pursuant to the provisions of 10 CFR 2.201, Northern States Power Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of

Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued to show cause why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Lisle, Illinois this
2nd day of June 1994