Docket No. 50-282 Docket No. 50-306

Northern States Power Company ATTN: Mr. D. D. Antony Vice President, Nuclear Generation 414 Nicollet Mall Minneapolis, MN 55401

Dear Mr. Antony:

This refers to the inspection conducted by Messrs. A. Dunlop and R. Mendez of this office on April 18 through May 6, 1994. The inspection included a review of activities for your Prairie Island Nuclear Generating Plant, Units 1 and 2. At the conclusion of the inspection, the findings were discussed with those members of your staff identified in the enclosed report.

The areas examined during the inspection are identified in the report. These areas consisted primarily of a review of the Prairie Island program for inservice testing (IST) of pumps and valves and of the effectiveness of the program regarding the performance of check valves. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

The inspection's results point to a number of concerns. Identified IST program and implementation issues indicate some deficiencies in Code and plant components knowledge necessary to effectively implement the IST program in all cases. Deficiencies in the check valve program indicated appropriate management oversight to assure a quality program had not been applied. And, although your self assessment/quality verification program had identified a number of deficiencies, it did not identify the significant IST and check valve issues in the attached report. Please provide your assessment of these issues along with your response to the Notice of Violation identified below.

Based on the inspection's results, certain of your activities, as specified in the enclosed Notice of Violation (Notice), appeared to violate NRC requirements. The violation is of concern because certain requirements of ASME Boiler and Pressure Vessel Code, Section XI were not adequately implemented into the IST program. Specifically, the residual heat removal pumps were not tested at a specified reference value, corrective actions were not taken when valve seat leakage exceeded acceptance criteria, and test procedures contained inadequate acceptance criteria for full flow testing several safety injection check valves.

We also request that you respond in writing to inspection follow-up item 94005-02 concerning your check valve program. The development and implementation were considered weak since the recommended actions in industry guidance documents were not followed to address check valve degradation and failure concerns.

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You are required to respond to this letter and should follow the instructions specified in the enclosed Notice when preparing your response. In your response, you should document the specific actions taken and any additional actions you plan to prevent recurrence. After reviewing your response to this Notice, including your proposed corrective actions and the results of future inspections, the NRC will determine whether further NRC enforcement action is necessary to ensure compliance with NRC regulatory requirements.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, the enclosure, and your responses to this letter and its enclosures will be placed in the NRC Public Document Room.

The responses directed by this letter and the enclosed Notice are not subject to the clearance procedures of the Office of Management and Budget as required by the Paperwork Reduction Act of 1980, Pub. L. No. 96.511.

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

G. C. Wright, Chief Engineering Branch

Enclosures:

Notice of Violation
 Inspection Reports
 No. 50-282/94005(DRS);
 No. 50-306/94005(DRS)

cc w/enclosures: E. L. Watzl, Site Manager Prairie Island Site M. Wadley, Plant Manager OC/LFDCB Resident Inspectors, Prairie Island, Monticello John W. Ferman, Ph.D. Nuclear Engineer, MPCA State Liaison Officer, State of Minnesota State Liaison Officer, State of Wisconsin Prairie Island, LPM, NRR J. Norberg, NRR/EMEB TSS, DRP S. Stein, SRS

bcc: PUBLIC IE-01

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Sincerely,

original signed by

G. C. Wright, Chief Engineering Branch

Enclosures:

1. Notice of Violation

2. Inspection Reports
No. 50-282/94005(DRS);
No. 50-306/94005(DRS)

cc w/enclosures: E. L. Watzl, Site Manager Prairie Island Site M. Wadley, Plant Manager OC/LFDCB Resident Inspectors, Prairie Island, Monticello John W. Ferman, Ph.D. Nuclear Engineer, MPCA State Liaison Officer, State of Minnesota State Liaison Officer, State of Wisconsin Prairie Island, LPM, NRR J. Norberg, NRR/EMEB TSS. DRP S. Stein, SRS

bcc: PUBLIC IE-01

| RIII   | RIII         | RIII        | RIII   | RLII   |
|--------|--------------|-------------|--------|--------|
|        | see previous | concurrence |        | 1 1Red |
| Dunlop | Mendez       | Gardner     | Kropp  | Wright |
| 5/ /94 | 5/ /94       | 5/ /94      | 5/ /94 | 6/2/94 |