

DCS MS-016

SEP 9 1982

Docket Nos. 50-282  
and 50-306

Mr. D. M. Musolf  
Nuclear Support Services Department  
Northern States Power Company  
414 Nicollet Mall - 8th Floor  
Minneapolis, Minnesota 55401

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Dear Mr. Musolf:

SUBJECT: COMPLETION OF CONTAINMENT PURGE GENERIC ISSUE B-24 AND NUREG-0737  
ITEM II.E.4.2 - PRAIRIE ISLAND UNIT NOS. 1 AND 2

In our letters dated November 28, 1978, October 29, 1979, November 2, 1981 and March 8, 1982 we identified generic concerns of purging and venting of containments that are applicable to the Prairie Island Nuclear Generating Plant Units No. 1 and No. 2. You have responded to our concerns by letters dated January 5, April 12, July 10, November 14, 1979; March 17, June 3, November 7, 1980; May 6, December 3, 1981 and April 30, 1982 on which we based our review. From this review we conclude that containment purging at Prairie Island may resume, using the low volume purge and ventilation system (4000 CFM) for both units on a limited basis. Purging on a limited basis by the low volume purge system is predicated upon your completing plant modifications committed to by your letter dated April 30, 1982 and having plant procedures and amendments to the Technical Specifications in place as a prerequisite to commence purging and venting operations.

You are requested to submit proposed changes to Technical Specifications within 90 days from the date of this letter covering the areas detailed below.

1. The containment penetrations of the high volume (33,000 CFM) purge and ventilation systems will be sealed shut by means of blind flanges isolating the purge and vent valves from the containment atmosphere when the units are operated above cold shutdown.
2. The double gasketed seals of the blind flanges of the high volume purge and ventilation system will undergo a type B leak test as prescribed by 10 CFR Part 50 Appendix J prior to operating the units above cold shutdown if the flanges have been removed during cold shutdown or during a refueling outage.
3. The containment penetrations of the low volume purge and ventilation systems will be sealed shut by means of blind flanges during plant operations above cold shutdown except when purging is required for safety related reasons.

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4. As a prerequisite to purging with the low volume purge and ventilation systems (4,000 CFM), the containment isolation valves will undergo a type C leak test as prescribed by 10 CFR Part 50 Appendix J to assure the isolation function of the purge and vent valves during purging operations.
5. The double gasketed seals of the blind flanges at the containment penetrations of the low volume purge and ventilation systems must undergo a type B leak test as prescribed in 10 CFR Part 50 Appendix J each time the blind flanges are installed as a prerequisite to operating the units above cold shutdown and after purging operation is completed when the reactor is above cold shutdown.
6. Purging by the low volume purge and ventilation system (4,000 CFM) shall be minimized and shall be limited to safety related reasons.

Your proposed schedule for completing the modifications on the high volume purge and ventilation during the next refueling outage for each unit (i.e., summer of 1982 for Unit 2 and the fall of 1982 for Unit 1) and the schedule for completing the modifications of the low volume purge and ventilation system by the 1982 refueling outage for Unit 1 and the 1983 refueling outage for Unit 2 are acceptable.

As a result of our review of your submittals we have concluded that you have adequately responded to our long term generic concerns regarding the containment purge issue. Except for resolving the matter of issuing amendments to the Technical Specifications, the long term generic concerns for containment purge are resolved. In addition, because of the similarity of these long term generic concerns with the positions identified in Item II.E.4.2 of NUREG-0737 THI Action Plan, we also completed our review of your responses to these positions. We find your responses to the position of Item II.E.4.2 of NUREG-0737 THI Action Plan are acceptable and therefore the issues of Item II.E.4.2 are resolved.

A copy of our Safety Evaluation Report is enclosed.

Sincerely,

Original signed by  
Robert A. Clark

Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division Licensing

Enclosure:  
Safety Evaluation

cc: See next page

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AEB  
WPasedag  
7/19/82

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DATE	6/14/82	6/15/82	7/13/82	6/15/82	6/17/82	6/17/82	7/1/82

Northern States Power Company

cc:

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