NRC FOR (7-77)	LICENSEE EVENT REPORT	CLEAN NEGULATION LOURINIDUTON
	CONTROL BLOCK:	FOURED INFORMATION
7 8	9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICE	ENSE TYPE JO 57 CAT 58
	AEPORT LO 15 10 10 12 16 0 12 11 16 18 13 18 1 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74	0 3 1 6 8 3 0 75 REPORT DATE 80
02] [On 2/16/83, unit 1 was operating at 95% power, unit 3 was opera	ting at 100% power,]
03	and unit 2 was in a refueling outage. During integrated leak r	ate test on unit 2, J
014] [the primary containment leak rate was found to exceed the allow	able leak rate of]
05	0.75 La (S. 4.7.A.2.b). There was no requirement to have primary containment	
06] [established. The leakage was below the design limit of La. No redundant systems	
07	were required to be in service and operable. There was no danger to health and	
08	safety of the public.	80
7 8	B 9 SYSTEM CAUSE CAUSE COMPONENT CODE COM CODE CODE SUBCODE COMPONENT CODE SUBCO	P. VALVE
7 8	$\begin{bmatrix} S & A \\ 9 & 10 \end{bmatrix} \begin{bmatrix} 12 \\ 11 \end{bmatrix} \begin{bmatrix} 12 \\ 12 \\ 12 \end{bmatrix} \begin{bmatrix} 13 \\ 13 \\ 13 \\ 13 \end{bmatrix} \underbrace{V A L V E X \\ 13 \\ SEQUENTIAL \end{bmatrix} V A L V E X \\ 13 \\ OCCUBRENCE \\ REF$	DORT REVISION
	TO REPORT VEAR REPORT NO. CODE TO NUMBER 8 3 - 0 0 5 - 0 3	
	ACTIONFUTUREEFFECTSHUTDOWNATTACHMENTNPRD-4TAKENACTIONON PLANTMETHODHOURS22ATTACHMENTNPRD-4B18Z19Z20Z2100013334353637404123N24CAUSE DESCRIPTION AND CORRECTIVE ACTIONS27	31 32 PRIME COMP, SUPPLIER COMPONENT MANUFACTURER L 25 Z 9 9 9 9 10 43 44 47
10	- Columna of 1 share on the state of the sta	e gasket. Tightening
11	of flange bolts stopped leak and reduced leak rate to approximate	ately 0.5%. Testing
12	on this and similar flanges is an unresolved item. Investigation continues and will	
13	be documented and resolved in a follow-up report prior to the e	end of the upcoming
14	unIt 1 refueling outage.]
7 8	FACILITY STATUS * POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCO H 0 0 0 0 0 0 0 0 0 9 10 12 13 44 45 46 46	struction 32
	ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATI	ION OF RELEASE 36
7 8	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)	03
1 7 7 8	I O I O O D Z GOL NA	en
1 7 8		88
10	LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION (43)	
	9 PUBLICITY ISSUED DESCRIPTION (45) N (4) (45) (45) (45) (45) (45) (45) (45)	NRC USE ONLY
	NAME OF PHERAPER Eddie Holder	(205) 729-0885

Tennessee Valley Authority

Browns Ferry Nuclear Plant

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83005 Technical Specification Involved 4.7.A.2.b

Reported Under Technical Specification 6.7.2.b. (4) * Date Due NRC 3/17/83

Event Narrative:

On February 16, 1983, unit 1 was operating at 95-percent power, unit 3 was operating at 100-percent power, and unit 2 was in a refueling outage. During the integrated leak rate test on unit 2, the primary containment leak rate was found to exceed the allowable leak rate of 0.75 La (1.5 percent/Day - Technical Specification 4.7.A.2.b). This leakage was below the design limit of 2 percent La. A leak was found on 2-FCV-64-20 at the flange gasket and flange bolts were tightened to stop the leak. On February 19, 1983, the primary containment leak rate was reduced to approximately .5 percent/Day. There was no danger to the health and safety of the public. No redundant systems were required to be in service and operable. Testing on this and similar type flanges is considered an unresolved item. Investigation is continuing and will be documented and resolved in a follow-up report prior to the end of the upcoming unit 1 refueling outage.

* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor *Revision: