DOCKET NO. 050-0331 DATE 3-15-83 COMPLETED BY WIIIiam Miller TELEPHONE 319-851-7238

OPERATING STATUS

1.	Unit Name Duane Arnold Energy Center
2.	Reporting Period February, 1983
3.	Llcensed Thermal Power (MWt): 1658
4.	Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5.	Design Electrical Rating (Net MWe): 538
6.	MaxImum Dependable Capacity (Gross MWe): 545
7.	Maximum Dependable Capacity (Net MWe): 515

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since the Last Report, Give Reasons:

Notes

9. Power Level to Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

		This Month	Yr-to-Date	Cumulative
11.	Hours In Reporting Period	672.0	1416.0	70800.0
12.	Number of Hours Reactor Was Critical	266.7	1010.7	51267.0
13,	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	266.7	1010.7	49945.9
15,	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	311704.8	1194016.8	61567456.0
17.	Gross Electrical Erergy Generated (MWH)	105698.0	410045.0	20624962.0
18.	Net Electrical Energy Generated (MWH)	96462.4	380160.6	19294433.0
19.	Unit Service Factor	39.7%	71.4%	70.5%
20.	Unit Availability Factor	39.7%	71.4%	70.5%
21,	Unit Capacity Factor (Using MDC Net)	27.9%	52.1%	52.9%
22.	Unit Capacity Factor (Using DER Net)	26.7%	49.9%	50.7%
23.	Unit Forced Outage Rate	0.0	0.0	17.5
24.	Shutdowns Scheduled Over Next 6 Months (Type,	Date, and Duration of E	ach):	

Refueling commenced on February 11, 1983

25. If Shut Down At End Of Report Period, Estimated Date of Startup: April 25, 1983

PDR

*Turbine Rating: 565.7 MWE

Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe 8303220062 830315 PDR ADOCK 05000331 R

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331

UNIT Duane Arnold Energy Center

DATE 3-15-83

COMPLETED BY WILLiam Miller

TELEPHONE 319-851-7238

MONTH February, 1983

DAY AVERAGE DAIL	POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
(MW 0~ 1	(e†)		(MWe-Net)
1 387		17	0
2 386		18	0
3 385		19	0
4382		20	0
5 305		21	0
6 390		22	0
7 378	17772324 V. 18973	23	0
8 377		24	0
9 375		25	0
10 373		26	0
11 341		27	0
120		28	0
130		29	N/A
140		30	N /A
150		31	N/A
16 0			

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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Unit	Name	Dua	ne	A	rno	l d	Energy	C
Date								
Compl	otod	by	W I	11	lam	M	ller	

enter

UNIT	T SH	UTDO	WNS	AND	POWER	REDUCT	IONS
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REPORT	MONTH February.	198

No.	Date	Typal	Duration (hours)	Reason ²	Method of Shutting Down Reactor 3	Licensee Event Report ∦	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
4	2/5/83	S	N/A	В	N/A	N/A	N/A	N/A	Power reduction to reduce exposure to personnel conducting condenser leak testing
5	2/12/83	S	405.3 (Thru 2/28/83)	с	1	N/A	N/A	N/A	Scheduled refueling shutdown

	Z	3	4
F: Forced	Reason:	Method:	Exhibit G-Instructions
S: Scheduled	A-Equipment Fallure(Explain)	1-Manual	for Preparation of Data
	B-Maintenance of Test	2-Manual Scram.	Entry Sheets for Licensee
	C-Refueling	3-Automatic Scram.	Event Report (LER) File (NURE)
	D-Regulatory Restriction	4-Other(Explain)	0161)
	E-Operator Training & License Examination		
	F-Administrative		5
	G-Operational Error(Explain)		Exhibit 1-Same Source
(9/77)	H-Other(Explain)		

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REFUELING INFORMATION

1. Name of facility.

A. Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

A. February 11, 1983

3. Scheduled date for restart following refueling.

A. April 25, 1983

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes, the following outstanding changes will have to be included in the Technical Specifications:

- A. MAPLHGR Tables & MCPR Limits C. Containment Load Case B. Snubbers Avoidance
- Scheduled date(s) for submitting proposed licensing action and supporting information.
 - A. January 14, 1983 (RTS-147)
 C. March 10, 1983 (RTS-146)
 B. March 2, 1983 (RTS-140)
- 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
 - A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.
- The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

A. a) 368 b) 448 (status prior to refueling outage)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

A. 2050

 The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

A. 1998

MAJOR SAFETY RELATED MAINTENANCE

Docket No	050-0331 .							
Unit Duan	Arnold Energy Cent	er						
Date 3-1	-83							
Completed	by William Miller							
Telephone	319-851-7238							

DATE	SYSTEM	COMPONENT	DESCRIPTION
2/8/83	RCIC	MOV-2515 - MO full flow test valve	Loose valve stem anti-rotation collar repair
2/11/83	MSIV-LCS	Fuse	Replaced fuse - random failure caused loss of MSIV-LCS "D" leakage flow instrumentation
2/11/83	RW Supply	Flow Transmitter and Recorder	Instrument calibration and baseline IST cause RW Supply pump to exceed ASM E developed head allowable
2/12/83	Reactor Instrumentation	Instrument Line Excess Flow Check Valve	Investigation continuing - Improper value position indication
2/14-15/83 and continuing	Varlous Safety Related Systems	Snubbers	Comprehensive inspection repair and replacement of snubbers (RO Report pending)
2/21/83	ADS	Pressure Switches on RHR for ADS permissive	Recalibration - instrument drift In conservative direction

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

2-1-83 Normal operation at 393 MWe.

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2-5-83 Power reduction to 260 MWe to reduce personnel exposure during scheduled condenser leak testing.

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2-8-83 RCIC declared inop due to loose valve stem anti-rotation collar.

RO Report 83-004

2-11-83 River Water Supply System 1P117B and D pumps declared inoperable when developed head exceeded ASME XI allowables. Instrumentation recalibration required.

RO Report 83-005

2-11-83 MSIV LCS "D" declared inoperative due to a blown fuse.

RO Report 83-006

2-12-83 Reactor Instrumentation Instrument Line Excess Flow check valve position problems.

RO Report 83-007

- 2-12-83 Commence scheduled refueling shutdown.
- 2-12-83 0239 Generator Breakers opened.
- 2-14-83 Uncontrolled release of 20 gallons of low level activity condensate service water. ETSV Report 83-1.
- 2-21-83 During cold shutdown surveillance testing, RHR pressure switches for ADS found out of tolerance in conservative direction.

RO Report 83-006