

OPERATING DATA REPORT

DOCKET NO 50-289
 DATE March 15, 1983
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit 1
 2. Reporting Period: February, 1983
 3. Licensed Thermal Power (MWt): 2535
 4. Nameplate Rating (Gross MWe): 871
 5. Design Electrical Rating (Net MWe): 819
 6. Maximum Dependable Capacity (Gross MWe): 840
 7. Maximum Dependable Capacity (Net MWe): 776
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
 10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672.	1416.	74449.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	840.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	13840033.
19. Unit Service Factor	0.0	0.0	41.9
20. Unit Availability Factor	0.0	0.0	41.9
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	40.9
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	39.1
23. Unit Forced Outage Rate	100.0	100.0	53.6
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE March 15, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

MONTH February 15, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-289

DOCKET NO.

THI-1

UNIT NAME

March 15, 1983

DATE

C. W. Smyth

COMPLETED BY

(717) 948-8551

TELEPHONE

REPORT MONTH February, 1983

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	83-02-01	F	744	D	1	N/A	ZZ	ZZZZZZZ	Regulatory Restraint Order

- 1 F - Forced
S - Scheduled
- 2 Reason:
A - Equipment Failure (Explain)
B - Maintenance of Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administrative
G - Operational Error (Explain)
H - Other (Explain)
- 3 Method:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Other (Explain)
- 4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (EER) File (NUREG-0161)
- 5 Exhibit I - Same Source

OPERATING SUMMARY

The Unit was shut down the entire report period by order of the NRC. The Reactor Coolant System was partially drained to permit preparation for OTSG repairs. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

During the month of February, restart modifications continued and the following major maintenance items were performed.

OTSG A/B TUBE REPAIR

The Once Through Steam Generator (OTSG) repair program continued with the completion of Felt Plug Blowing (tube cleaning) and milling tube ends on the upper tube sheets. Preparations were made for flushing both steam generators and the flushing sequence commenced near the end of the month.

RIVER WATER (SW-P-1B) OVERHAUL

The scheduled overhaul of river water pump SW-P-1B was completed with the reassembly of the pump and motor and satisfactory IST testing.

LOCAL LEAK RATE TEST PROGRAM

The Local Leak Rate Testing Program continued with satisfactory testing of the Reactor Building equipment hatch, Penetration 105 and 106, and the following valves:

- | | |
|--------------|---------------|
| 1. CF-V-12A | 7. HR-V-22A/B |
| 2. FS-V-401 | 8. HR-V-24A/B |
| 3. HP-V-1 | 9. LR-V-1 |
| 4. HP-V-6 | 10. LR-V-49 |
| 5. HR-V-2A/B | 11. PP-V-193 |
| 6. HR-V-4A/B | |

SPENT FUEL BRIDGE OVERHAUL

The Spent Fuel Bridge Overhaul was completed with the reassembly of the inner mast, reinstallation of mast trolley brake adjustments and weld repairs to the powertrak assembly.

RCS INSPECTION PROGRAM

A Pressurizer Valve Inspection Program commenced during the month of February with valves RC-RV-2 and RC-V-2 opened for inspections. Corrosion was found in the valves and further inspections will be carried over into the month of March. Analysis is being performed on the corrosion products found. These findings were reported in LER 02-003.

NUC SERVICE CLOSED PUMP (NS-P-1B)MODIFICATION

The Nuclear Services Closed Cooling Pump NS-P-1B mechanical seal modification was completed during the month with the reassembly of pump, seal installation, coupling alignment and satisfactory testing.

WASTE GAS SYSTEM OUTAGE

A scheduled Waste Gas Disposal outage commenced during the month with valves WDG-V-5, WDG-V-6, WDG-V-7, and WDG-V-8 opened, chemistry wipe samples taken, diaphragms and operator O-Rings replaced and valves reassembled. Various repair activities were accomplished on the waste gas system valves. Diaphragm changeouts, preventive maintenance items, flange leaks, etc. will be continued into the month of March.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

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14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
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INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

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MONTH February 15, 1983

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7	<u>0</u>
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10	<u>0</u>
11	<u>0</u>
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27	<u>0</u>
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1983

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¹
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²
 Reason:
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