NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

Update Report
Previous Report Date 6-30-80

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | LICENSEE CODE: GAEIH1 (2) | LICENSE NUMBER: 00-00000-00 (3) | LICENSE TYPE: 41111 (4) | CAT (5)

CON'T 0 1 | REPORT SOURCE: L (6) | DOCKET NUMBER: 0500032 1 (7) | EVENT DATE: 053180 (8) | REPORT DATE: 031083 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 With Unit 1 in cold shutdown an inspection of mechanical snubbers was
0 3 performed. 45 out of 61 snubbers were found to be locked up. This is a
0 4 non-repetitive occurrence. There are no backups for these snubbers.
0 5 Locking of these snubbers prevented thermal movement of the associated
0 6 piping which could have damaged it. No damage to piping was noted dur-
0 7 ing a visual inspection. This event posed no threat to public health or
0 8 safety. On Unit 2, 3 out of 45 snubbers were found to be locked up.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	XX (11)	E (12)	B (13)	SUPORT (14)	D (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	80	—	055	/	01	X	--	2

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
C (18)	C (19)	C (20)	Z (21)	0000	Y (23)	N (24)	A (25)	I137 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event was found to be internal corrosion of the snub-
1 1 bers. All snubbers that were found locked up were replaced prior to
1 2 startup of the respective unit. All INS snubbers have been replaced with
1 3 mechanical snubbers manufactured by Pacific Scientific, with the excep-
1 4 tion of three snubbers that were replaced with rigid restraints.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	G (28)	000 (29)	NA	C (31)	Special Inspection

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	000 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	000 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

8303210354 830310
PDR ADOCK 05000321
S PDR

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

NAME OF PREPARER H. L. Sumner - Supt. Plt. Eng. Serv. PHONE: 912-367-7851

GPO 917-926

LER #: 50-321/1980-055, Rev. 2
Licensee: Georgia Power Company
Facility Name: Edwin I. Hatch
Docket #: 50-321

Narrative Report
for LER 50-321/1980-055, Rev. 2
Update Report - Previous Report Date 6-30-80

A snubber allows the pipe to move due to thermal expansion yet restricts sudden movement due to a seismic event. A locked up snubber would prevent thermal movement which could cause damaging stresses in the pipe. While inspecting the snubbers, the associated piping was also visually inspected. No cases of pipe damage were noted. This event posed no threat to public health or safety.

The cause of this event was found to be internal corrosion of the snubbers. On Unit 2 the failure mode is believed to be galling of the stainless steel snubber internal screw. All of these snubbers were manufactured by International Nuclear Safeguards Inc. All snubbers that were found locked up have been replaced with Pacific Scientific mechanical snubbers with the exception of three. These were replaced with rigid restraints with prior approval of the AE.

The AE is in the process of performing computer analyses and revising the Class I Stress Reports where the magnitude of thermal movements or other conditions warrant. These analyses will be performed in conjunction with ongoing activities for IE Bulletin 79-14 using "as-built" information.

A Class 2 analysis has been performed and only one potential high stress area has been noted. This was an area inboard of the RWCU inboard motor operated isolation valve and included the valve and an elbow adjacent to the valve. This was class 1, 6" schedule 80 stainless steel piping. UT examination of this area was performed and no indications were found, therefore indicating that the lock up of mechanical snubbers in this area did not result in pipe failure.