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March 15, 1983

Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Additional Information Related to NUREG-0737, Item II.D.1.2,
Relief and Safety Valve Performance

Our letters dated July 1, 1982 and July 19, 1982 provided information related to implementation of NUREG-0737, Item II.D.1.2, at the Prairie Island Nuclear Generating Plant. Our safety and relief valve installation was described and information was provided showing this installation was bounded by the valves and test parameters of the EPRI Safety and Relief Valve Test Program. We noted that work remained in the area of piping and support evaluation and review with Westinghouse and Crosby of the necessity of making ring adjustments to the safety valves. The purpose of this letter is to provide updated information related to these open items.

Unit 1 Piping and Support Evaluation

The Unit 1 thermal hydraulic analysis is complete. Results show the need for two additional snubbers downstream of the safety valves plus the addition of insulation to loop seals upstream of the safety valves. The insulation was installed during the Unit 1 refueling outage (Nov-Dec 82). Material availability prevented installation of the additional snubbers.

Additionally, Fluor Power is investigating the affects of increased support loadings, which resulted from the new Westinghouse thermal-hydraulic analysis. While major changes to existing supports are not anticipated, undoubtedly some changes will be required. Work is now underway to identify material or modifications required to accomplish the work at the next refueling (autumn 83).

Unit 2 Piping and Support Evaluation

Analytical work is underway at Westinghouse on the thermal-hydraulic analysis. Fluor is closely monitoring the Westinghouse work to get an early start on design of additional and/or modified supports. Our goal is to install any required modifications during the upcoming Unit 2 refueling (summer 83). Similar modifications, as were required for Unit 1, are expected for unit 2.

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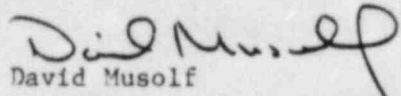
NORTHERN STATES POWER COMPANY

Director of NRR
March 15, 1983
Page 2

Safety Valve Adjustments

Westinghouse and Crosby Valve are still working on a scheme to optimize valve ring settings. Westinghouse has indicated that agreement with Crosby on approach will occur during early 1983. Outcome of the joint study is unknown at this time.

Please contact us if you have any questions related to our plans and schedule for resolving these remaining issues.



David Musolf

Manager - Nuclear Support Services

DMM/bd

cc: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
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