

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183 License No. DPR-77

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated February 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 183, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of its date of issuance, to be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Jordanie J. Wit

Frederick J. Hebdon, Director Project Directorate II-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 27, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 183

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

1.4

INSERT

3/4 3-27a

3/4 3-27a

TABLE 3.3-4 (Continued)

	ENGINEERED SAFETY FEATUR	E ACTUATION SYSTEM INSTRUMENTATION	TRIP SETPOINTS
FUNCTIONAL UNIT		TRIP SETPOINT	ALLOWABLE VALUES
	ii. RCS Loop AT Equivalent to Power > 50% RTP		
	Coincident with Steam Generator Water Level Low-Low (Adverse)	≥15.0% of narrow range instrument span	≥14.4% of narrow range instrument span
	Containment Pressure (EAM)	≤0.5 psig	≤0.6 psig
	Steam Generator Water LevelLow-Low (EAM)	≥10.7% of narrow range instrument span	≥10.1% of narrow range instrument span
d.	S.I.	See 1 above (all SI Setpoints)	
e.	Loss of Power Start		
	1. Voltage Sensors	≥5520 volts	≥5472 volts
	2. Load Shed Timer	1.25 seconds	1.25 seconds ±0.09 seconds
f.	Trip of Main Feedwater Pumps	N.A.	N.A.
g.	Auxiliary Feedwater Suction Pressure-Low	≥ 3.21 psig (motor driven pump) ≥ 13.9 psig (turbine driven pump)	≥ 2.44 psig (motor driven pump) ≥ 12 psig (turbine driven pump)
h.	Auxiliary Feedwater Suction Transfer Time Delays	4 seconds (motor driven pump)	4 seconds ±0.4 seconds (motor driven pump)
		5.5 seconds (turbine driven pump)	5.5 seconds ±0.55 seconds (turbine driven pump)

3/4 3-27a

SEQUOYAH - UNIT 1

Amendment No. 29, 94, 129, 141, 151, 182,



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175 License No. DPR-79

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated February 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.175, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of its date of issuance, to be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

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Frederick J. Hebdon, Director Project Directorate II-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 27, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
3/4 3-27a	3/4 3-27a

TABLE 3.3-4 (Continued)

FUNCTIONAL UNIT		T	TRIP SETPOINT	ALLOWABLE VALUES	
_		ii.	RCS Loop ∆T Equivalent to Power > 50% RTP		
			Coincident with Steam Generator Water Level Low-Low (Adverse)	≥15.0% of narrow range instrument span	≥14.4% of narrow range instrument span
			Containment Pressure (EAM)	≤0.5 psig	≤0.6 psig
			Steam Generator Water LevelLow-Low (EAM)	≥10.7% of narrow range instrument span	≥10.1% of narrow range instrument span
	d.	S.I.		See 1 above (all SI Setpoints)	
	e.	Loss	of Power Start		
		1.	Voltage Sensors	≥5520 volts	≥5472 volts
		2.	Load Shed Timer	1.25 seconds	1.25 seconds ±0.09 seconds
	f.	Trip Pump	of Main Feedwater s	N.A.	N.A.
	g.	Auxi Pres	liary Feedwater Suction sure-Low	≥ 3.21 psig (motor driven pump) ≥ 13.9 psig (turbine driven pump)	≥ 2.44 psig (motor driven pump ≥ 12 psig (turbine driven pump
	h.	Auxi Tran	liary Feedwater Suction sfer Time Delays	4 seconds (motor driven pump)	4 seconds ±0.4 seconds (motor driven pump)
				5.5 seconds (turbine driven pump)	5.5 seconds ±0.55 seconds (turbine driven pump)

SEQUOYAH - UNIT 2

3/4 3-27a Amendment No. 18, 84, 116, 132, 174,

175