

September 15, 1982

SBN-325
T.F. B 7.1.2

United States Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Ms. Janis B. Karrigan, Acting Chief
Licensing Branch 3
Division of Licensing

References: (a) Construction Permit CPPR-135 and CPPR-136, Docket
Nos. 50-443 and 50-444
(b) USNRC Letter, dated July 27, 1982, "Request for Additional
Information", F. J. Miraglia to W. C. Tallman
(c) Telecon of July 27, 1982, Sal Salah (NRC) to Phil Swanson
(PSNH)
(e) Telecon of July 29, 1982, Sal Salah (NRC) to Phil Swanson
(PSNH)

Subject: Response to 610 Series RAIs (Operator Licensing Branch)

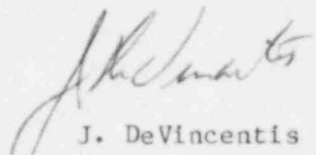
Dear Ms. Karrigan:

We have enclosed responses to the subject Requests for Additional
Information (RAIs) which you forwarded in Reference (b), with the exception of
610.1.

RAI 610.1, which addresses NUREG-0737, Items I.A.2.1, I.A.2.3, I.A.3.1,
and II.B.4, was discussed in the referenced telecons. A response to RAI 610.1
will be submitted in the near future.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY


J. DeVincentis
Project Manager

Boo!

ALL/dd

Attachment

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QUESTION 610.2

Reference FSAR page 13.2-2, Item 4 (Replacement Training)-Provide details of replacement training program and how this replacement training is applied to operators with different previous experience.

Response to 610.2

Details of the Replacement Training for Licensed Individuals is included in the FSAR Subsection 13.2.1.4, Amendment 45, June 1982.

Definition of PSNH policy for waiver of program elements will be included in a future FSAR amendment which will modify Subsection 13.2.1.4a to read:
. . .detailed in Subsection 13.2.1.i.

Waiver of specific program elements defined below may be granted based on a procedurally administered review of each individual's education, formal training and experience. Prior to granting a waiver, a determination will be made that the objectives of the waived element are met for the license level sought. Documentation of the formal review and the conclusions reached will be maintained.

QUESTION 610.3

Reference FSAR page 13.2-7, item 13.2.1.3-6 (On-the-Job Training)- Provide documentation of conformance with the requirements of H. R. Denton's letter of March 28, 1980 on Qualification of Reactor Operators (see Enclosure 4 of the letter).

Response to 610.3

PSNH will revise FSAR Subsection 13.2.1.3b as follows:

b. On-the-Job-Training

Control manipulations and plant evolutions listed below will be performed as follows: The starred items shall be performed annually; all other items shall be performed on a two-year cycle. Multiple failure casualties will be included in the program.

The unit shift supervisor is responsible for the rotation of his shift complement to maximize participation in the below listed plant evolutions. Those control manipulations which are not performed at the plant shall be performed on the simulator. The use of the Technical Specifications will be maximized during the simulator control manipulations. Personnel with senior licenses are credited with activities they direct or evaluate during actual performance.

- *1. Plant or reactor startups to include a range that reactivity feedback from nuclear heat addition is noticeable and heatup rate is established.
- *2. Plant shutdown.
- *3. Manual control of steam generators and/or feedwater during startup and shutdown.
- 4. Boration and/or dilution during power operation.
- *5. Any significant (>10%) power changes due to manual changes in control rod position or boron concentration.
- *6. Loss of coolant including:
 - 1. Significant steam generator tube leaks.
 - 2. Inside and outside primary containment.
 - 3. Large and small, including leak-rate determination.
 - 4. Saturated reactor coolant response.

Response to 610.3 (Continued)

7. Loss of instrument air.
8. Loss of electrical power and/or degraded power sources.
- *9. Loss of core coolant flow/natural circulation.
10. Loss of condenser vacuum.
11. Loss of service water.
12. Loss of shutdown cooling.
13. Loss of component cooling system or cooling to an individual component.
14. Loss of normal feedwater or normal feedwater system failure.
- *15. Loss of all feedwater (normal and emergency).
16. Loss of protective system channel.
17. Mispositioned control rod or rods (or rod drops).
18. Inability to drive control rods.
19. Conditions requiring use of emergency boration.
20. Fuel cladding failure or high activity in reactor coolant.
21. Turbine or generator trip.
22. Malfunction of Automatic Control System(s) which effect reactivity.
23. Malfunction of reactor coolant pressure/volume control system.
24. Reactor trip.
25. Main steam line break (inside or outside containment).
26. Nuclear instrumentation failure(s).