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Portland General Electric Company
Trojan Nuclear Plant
P.O. Box 439
Rainier, Oregon 97048
(503) 556-3713

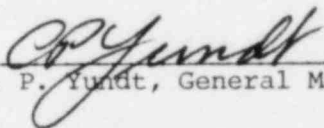
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1982 SEP 17
September 8, 1982
CPY-703-82

Mr. R. H. Engelken
Regional Administrator
US Nuclear Regulatory Commission
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Dear Sir:

In accordance with the Trojan Plant Operating License, Appendix A, USNRC Technical Specification 6.9.1.9b, Licensee Event Report No. 82-13 concerning the excessive leakage through four containment isolation valves that, when combined, resulted in a cumulative leakage that exceeded the allowed limit given in Technical Specification 3.6.1.2 is attached.

Sincerely,


C. P. Yundt, General Manager


R. L. Steele, Manager
Nuclear Projects Engineering

CPY/GCB:ga

Attachments

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REPORTABLE OCCURRENCE

1. Report No: 82-13
2. Report Date: September 8, 1982
3. Occurrence Date: August 13, 1982
4. Facility: Trojan Nuclear Plant, PO Box 439, Rainier, Oregon 97048
5. Identification of Occurrence:

The cumulative leakage through four Type C containment valves measured during periodic local leak rate testing (LLRT) exceeded the Technical Specification limit 3.6.1.2.
6. Conditions Prior to Occurrence:

The plant was shut down for a refueling outage.
7. Description of Occurrence:

During the 1982 refueling outage, LLRT (PET-5-2) was conducted to verify the integrity of containment Type B and C penetrations. Four Type C valves (CV-5661, reactor coolant drain tank gas analyzer isolation; CV-4470, service air to containment isolation; CV-4301, radioactive gas collection header isolation; and MO-5674, containment atmospheric sample isolation) were found to have leakage greater than 11,000 cc/min. As a result the cumulative penetration valve leakage exceeded the Technical Specification limit of 23,615 cc/min. The LLRT equipment has a maximum flow indication of 11,000 cc/min, therefore leakage above this value could not be quantified.
8. Designation of Apparent Cause of Occurrence:

The causes of this occurrence are attributed to the following:

CV-5661 and CV-4301	- component failure
MO-5674	- personnel error
CV-4470	- other (rust buildup)

Each of the high leakage valves had a minor problem that prevented it from achieving its design leak tightness. These are summarized in Table 1 (attached).
9. Significance of Occurrence:

This event had no effect on either plant or public safety since the redundant containment isolation valve in each of the four affected penetrations did not exceed leakage limits. Individual failures were compared to previous test results. No repetitive failure mode for these valves was identified.
10. Corrective Action:

Each valve was repaired prior to plant heatup. All valves were re-tested and found to be within acceptable limits with final leakage values below 60 cc/min.

TABLE 1

PENETRATION NO.	SYSTEM	VALVE NO.	CAUSE	REPAIRS	INITIAL LEAKAGE cc/min	FINAL LEAKAGE cc/min
P-14-1	RCDT Gas Analyzer	CV-5661	Improper Seating	Seat Relapped	>11,000	59.9
P-21	Service Air	CV-4470	Rust on Seat	Seat Cleaned	>11,000	10.0
P-54	Gas Collection Header	CV-4301	One Side of Gate Had no Contact due to a Weak Closing Spring.	Replaced Closing Spring and Gate, and Relapped Seat	>11,000	0.0
P-56-1	Containment Atmosphere Sample	MO-5674	Improper Torque Switch Setting	Adjusted Switch	>11,000	20.0