

Entergy Operations, Inc. Route 8. Box: 1370 Russelvile: AR-72801 Tel: 501-964-3100

10CFR50.46(a)(3)(ii)

May 27, 1994

0CAN059406

U. S. Nuclear Regulatory Commission Document Control Desk Mail Station P1-137 Washington, DC 20555

Subject: Arkansas Nuclear One - Units 1 and 2 Docket Nos. 50-313 and 50-368 License Nos. DPR-51 and NPF-6 Errors or Changes in the Emergency Core Cooling System Evaluation; Annual Report For 1993

Gentlemen:

10CFR50.46(a)(3)(ii) requires licensees to report each change to or error discovered in an acceptable evaluation model or in the application of such model for the emergency core cooling system (ECCS) at least annually and the estimated effect these changes or errors have on the limiting ECCS analysis. The purpose of this submittal is to provide the information required by 10CFR50.46(a)(3)(ii) for Arkansas Nuclear One (ANO), Units 1 and 2 for the reporting period from December 1992 through December 1993.

## ANO-1

No significant changes or errors (peak cladding temperature change of greater than 50 degrees F) were reported, and no changes have been made to the Babcock and Wilcox ECCS evaluation model.

## ANO-2

No significant changes or errors have been identified for the ABB-CE evaluation model used for ANO-2. One error in the COMPERC-II refill/reflood code for large break loss of coolant accident (LOCA) analysis was found and corrected. Correction of the error in COMPERC-II had no effect on the peak cladding temperature for large break LOCA. No models or methods were changed for the large break, small break, or post-LOCA long term cooling calculations. The attached report (CENPD-279 Supplement 5 dated February 1994) is the annual report on the ABB Combustion Engineering ECCS codes and methods for 1993. In February 1994, the CENPD-132 Supplement 3-P-A evaluation

9406030182 940527 PDR ADOCK 05000313 PDR PDR

U. S. NRC May 27, 1994 0CAN059406 Page 2

model was approved for use for ANO-2 large break LOCA analyses by the Staff. This analysis has incorporated all the errors and changes that have been identified in prior years pertaining to the large break LOCA analysis, except those identified in Supplement 5 of CENPD-279.

Should you have any questions, please contact me.

Very truly yours,

Quight C. Momie

Dwight C. Mims Director, Licensing

DCM/nbm Attachment

 cc: Mr. Leonard J. Callan Regional Administrator
U. S. Nuclear Regulatory Commission Region IV
611 Ryan Plaza Drive, Suite 400 Arlington, TX 76011-8064

> NRC Senior Resident Inspector Arkansas Nuclear One - ANO-1 & 2 Number 1, Nuclear Plant Road R issellville, AR 72801

Mr. George Kalman NRR Project Manager Region IV/ANO-1 U. S. Nuclear Regulatory Commission NRR Mail Stop 13-H-3 One White Flint North 11555 Rockville Pike Rockville, MD 20852

Mr. Thomas W. Alexion NRR Project Manager, Region IV/ANO-2 U. S. Nuclear Regulatory Commission NRR Mail Stop 13-H-3 One White Flint North 11555 Rockville Pike Rockville, MD 20852