

OPERATING DATA REPORT

DOCKET NO. 50-416  
 DATE 9/15/82  
 COMPLETED BY J.D. Richardson  
 TELEPHONE 601-969-2510

OPERATING STATUS

1. Unit Name: Grand Gulf Nuclear Station
2. Reporting Period: August 1982
3. Licensed Thermal Power (MWT): 191
4. Nameplate Rating (Gross MWe): 1372.5
5. Design Electrical Rating (Net MWe): 1250
6. Maximum Dependable Capacity (Gross MWe): N/A
7. Maximum Dependable Capacity (Net MWe): N/A
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:  
N/A

|       |
|-------|
| Notes |
|-------|

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: as restricted by O.L. NPF-13 as listed in 3.0 above.

|  | This Month | Yr.-to-Date | Cumulative |
|--|------------|-------------|------------|
| 11. Hours In Reporting Period  | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 12. Number Of Hours Reactor Was Critical   | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 13. Reactor Reserve Shutdown Hours   | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 14. Hours Generator On-Line  | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 15. Unit Reserve Shutdown Hours  | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 16. Gross Thermal Energy Generated (MWH)   | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 17. Gross Electrical Energy Generated (MWH)  | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 18. Net Electrical Energy Generated (MWH)  | <u>0</u>   | <u>0</u>    | <u>0</u>   |
| 19. Unit Service Factor  | <u>N/A</u> |             |            |
| 20. Unit Availability Factor   | <u>N/A</u> |             |            |
| 21. Unit Capacity Factor (Using MDC Net)   | <u>N/A</u> |             |            |
| 22. Unit Capacity Factor (Using DER Net)   | <u>N/A</u> |             |            |
| 23. Unit Forced Outage Rate  | <u>N/A</u> |             |            |
| 24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):<br><u>N/A</u> |            |             |            |

|  |                |                |
|--|----------------|----------------|
| 25. If Shut Down At End Of Report Period, Estimated Date of Startup: | <u>N/A</u>     |                |
| 26. Units In Test Status (Prior to Commercial Operation):            | Forecast       | Achieved       |
| INITIAL CRITICALITY  | <u>8/14/82</u> | <u>8/18/82</u> |
| INITIAL ELECTRICITY  | <u>2/83</u>    |                |
| COMMERCIAL OPERATION   | <u>8/83</u>    |                |

## INSTRUCTIONS FOR COMPLETING OPERATING DATA REPORT

This report should be furnished each month by licensees. The name and telephone number of the preparer should be provided in the designated spaces. The instructions below are provided to assist licensees in reporting the data consistently. The number of the instruction corresponds to the item number of the report format.

1. **UNIT NAME.** Self-explanatory.
2. **REPORTING PERIOD.** Designate the month for which the data are presented.
3. **LICENSED THERMAL POWER ( $MW_t$ )** is the maximum thermal power, expressed in megawatts, currently authorized by the Nuclear Regulatory Commission.
4. **NAMEPLATE RATING (GROSS  $MW_e$ )**. The nameplate power designation of the turbine-generator in megavolt amperes (MVA) times the nameplate power factor of the turbine generator.
5. **DESIGN ELECTRICAL RATING (NET  $MW_e$ )** is the nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
6. **MAXIMUM DEPENDABLE CAPACITY (GROSS  $MW_e$ )** is the gross electrical output as measured at the output terminals of the turbine-generator during the most restrictive seasonal conditions.
7. **MAXIMUM DEPENDABLE CAPACITY (NET  $MW_e$ )**. Maximum dependable capacity (gross) less the normal station service loads.
8. Self-explanatory.
9. **POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET  $MW_e$ )**. Note that this item is applicable only if restrictions on the power level are in effect. Short-term (less than one month) limitations on power level need not be presented in this item.  

Since this information is used to develop figures on capacity lost due to restrictions and because most users of the "Operating Plant Status Report" are primarily interested in energy actually fed to the distribution system, it is requested that this figure be expressed in  $MW_e$ -Net in spite of the fact that the figure must be derived from  $MW_t$  or percent power.
10. **REASONS FOR RESTRICTIONS, IF ANY.** If item 9 is used, item 10 should explain why. Brief narrative is acceptable. Cite references as appropriate. Indicate whether restrictions are self-imposed or are regulatory requirements. Be as specific as possible within space limitations. Plants in startup and power ascension test phase should be identified here.
11. **HOURS IN REPORTING PERIOD.** For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period.  

For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first. Adjustments in clock hours should be made in which a change from standard to daylight-savings time (or vice versa) occurs.
12. **NUMBER OF HOURS REACTOR WAS CRITICAL.** Show the total number of hours the reactor was critical during the gross hours of the reporting period.
13. **REACTOR RESERVE SHUTDOWN HOURS.** The total number of hours during the gross hours of reporting period that the reactor was removed from service for administrative or other reasons but was available for operation.
14. **HOURS GENERATOR ON-LINE.** Also called Service Hours. The total number of hours expressed to the nearest tenth of an hour during the gross hours of the reporting period that the unit operated with breakers closed to the station bus. These hours, plus those listed in Unit Shutdowns for the generator outage hours, should equal the gross hours in the reporting period.
15. **UNIT RESERVE SHUTDOWN HOURS.** The total number of hours expressed to the nearest tenth of an hour during the gross hours of the reporting period that the unit was removed from service for economic or similar reasons but was available for operation.
16. **GROSS THERMAL ENERGY GENERATED (MWH).** The thermal output of the nuclear steam supply system during the gross hours of the reporting period, expressed in megawatt hours (no decimals).
17. **GROSS ELECTRICAL ENERGY GENERATED (MWH).** The electrical output of the unit measured at the output terminals of the turbine-generator during the gross hours of the reporting period, expressed in megawatt hours (no decimals).
18. **NET ELECTRICAL ENERGY GENERATED (MWH).** The gross electrical output of the unit measured at the output terminals of the turbine-generator minus the normal station service loads during the gross hours of the reporting period, expressed in megawatt hours. Negative quantities should not be used. If there is no net positive value for the period, enter zero (no decimals).
- 19-23. For units still in the startup and power ascension test phase, items 19-23 should not be computed. Instead, enter N/A in the current month column. These five factors should be computed starting at the time the unit is declared to be in commercial operation. The cumulative figures in the second and third columns should be based on commercial operation as a starting date.

19. **UNIT SERVICE FACTOR.** Compute by dividing hours the generator was on line (item 14) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent. Do not include reserve shutdown hours in the calculation.
20. **UNIT AVAILABILITY FACTOR.** Compute by dividing the unit available hours (item 14 plus item 15) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.
21. **UNIT CAPACITY FACTOR (USING MDC NET).** Compute by dividing net electrical energy generated (item 18) by the product of maximum dependable capacity (item 7) times the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.
22. **UNIT CAPACITY FACTOR (USING DER NET).** Compute as in item 21, substituting design electrical rating (item 5) for maximum dependable capacity.
23. **UNIT FORCED OUTAGE RATE.** Compute by dividing the total forced outage hours (from the table in Unit Shutdowns and Power Reductions) by the sum of hours generator on line (item 14) plus total forced outage hours (from the table in Unit Shutdowns and Power Reductions). Express as percent to the nearest tenth of a percent.
24. **SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH).** Include type (refueling, maintenance, other), proposed date of start of shutdown, and proposed length of shutdown. It is recognized that shutdowns may be scheduled between reports and that this item may not be all inclusive. Be as accurate as possible as of the date the report is prepared. This item is to be prepared each month and updated if appropriate until the actual shutdown occurs.
25. Self-explanatory.
26. Self-explanatory. Note, however, that this information is requested for all units in startup and power ascension test status and is not required for units already in commercial operation.

**TEST STATUS** is defined as that period following initial criticality during which the unit is tested at successively higher outputs, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the unit is generally considered by the utility to be available for commercial operation.

Date of **COMMERCIAL OPERATION** is defined as the date that the unit was declared by the utility owner to be available for the regular production of electricity, usually related to the satisfactory completion of qualification tests as specified in the purchase contract and to the accounting policies and practices of the utility.



UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-416  
 UNIT NAME Grand Gulf Nuclear Sta.  
 DATE 9/15/82  
 COMPLETED BY J. D. Richardson  
 TELEPHONE 601-969-2630

REPORT MONTH August

| No. | Date | Type <sup>1</sup> | Duration (Hours) | Reason <sup>2</sup> | Method of Shutting Down Reactor <sup>3</sup> | Licensee Event Report # | System Code <sup>4</sup> | Component Code <sup>5</sup> | Cause & Corrective Action to Prevent Recurrence |
|-----|------|-------------------|------------------|---------------------|--|-------------------------|--------------------------|-----------------------------|---|
| N/A | N/A  | N/A               | N/A              | N/A                 | N/A  | N/A                     | N/A                      | N/A                         | N/A   |

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

(9/77)

## UNIT SHUTDOWNS AND POWER REDUCTIONS

### INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely<sup>1</sup>. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

**NUMBER.** This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

**DATE.** This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

**TYPE.** Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

**DURATION.** Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

**REASON.** Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

**METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER.** Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

**LICENSEE EVENT REPORT #.** Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

**SYSTEM CODE.** The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

**COMPONENT CODE.** Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component; e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

**CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE.** Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No. 50-416  
Date 9/14/82  
Completed by: J. D. Richardson  
Telephone 601-969-2510

OPERATING DATA REPORT  
SUPPLEMENTAL INFORMATION

As referenced in letter dated August 13, 1982 (AECM-82/348) MP&L reported that a temporary change to the Grand Gulf Nuclear Station solid radwaste system was made on or about June 23, 1982 in accordance with Technical Specification 3.11.3. As indicated in the aforementioned letter, additional information would be provided in the August Monthly Operating Report.

In accordance with the Grand Gulf Technical Specifications 6.15.1, licensee initiated major changes to the radwaste system shall be reported to the Commission in the Monthly Operating Report. The discussion of each change shall contain:

- a. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
- b. Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;
- c. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;
- d. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments thereto;
- e. An evaluation of the change which shows the expected maximum exposures to individual in the unrestricted area and to the general population that differ from those previously estimated in the license application and amendments thereto;
- f. A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period prior to when the changes are to be made;
- g. An estimate of the exposure to plant operating personnel as a result of the change; and
- h. Documentation of the fact that the change was reviewed and found acceptable by the NRC.

RESPONSE

- a. A summary evaluation has revealed that the modification to the radwaste solidification system does not 1), increase the probability of an occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report 2, does not create the possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report and 3), does not reduce the margin of safety as defined in the basis for any technical specification.
- b. The permanent plant radwaste solidification system could not be readied for use in concert with the present schedule for operation because changes to the permanent plant radwaste solidification system based on vendor recommendations were incomplete. To support plant operations in the interim, a portable solidification system was placed in service.
- c. A detailed description of the portable radwaste solidification is presented in the proprietary Topical Report CNSI-2 Revision 1 (4313-01354-01) entitled "Mobile Cement Solidification System" submitted to the Commission March 22, 1982 by Chem-Nuclear Systems, Inc. (CNSI).
- d. Section 9.0 of the Topical Report referenced in c. above and answers to the staffs questions 23 and 31 pertaining to the topical report and submitted to the staff on June 23, 1982 provide details of a postulated worst case failure of the Mobile Solidification Systems. Comparison of this CNSI postulated event to events analyzed in the Grand Gulf Final Safety Analysis Report (FSAR) sections 15.7.2 and 15.7.3 indicate the FSAR analysis bounds the CNSI event. Therefore, use of the CNSI Mobile Solidification System will not result in releases which differ from those previously predicted nor will there be a change to the maximum exposure to an individual in the unrestricted area.
- e. Refer to the response in part d. above.
- f. This question does not apply since there is no release data to provide a comparison to the predicted releases.
- g. An estimate of the exposure to plant personnel as a result of the use of the CNSI Mobile Solidification System is best explained in Section 4.5 of the referenced Topical Report and the response to the staff's question number 4 submitted by letter on June 23, 1982.
- h. The PSRC reviewed the CNSI system on June 23, 1982 and approved the use of the system on June 24, 1982. This approval is documented in the PSRC meeting minutes.