

DUKE POWER COMPANY

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March 14, 1983

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief  
Licensing Branch No. 4

Re: McGuire Nuclear Station, Unit 1  
Docket No. 50-369  
D2/D3 Steam Generator Design Modification

Dear Mr. Denton:

My letter of March 1, 1983 provided additional information requested by the NRC in Ms. E. G. Adensam's (NRC/NRR) letter dated February 18, 1983 concerning the NRC's review of a modification to the Main Feedwater lines to the steam generator which is intended to alleviate certain forward flushing transients. In accordance with a March 8, 1983 telecon between Mr. P. B. Nardoci (DPC), et.al., and Mr. J. N. Ridgely (NRC/NRR), et.al., the following clarifications to our responses to certain items are provided:

Question 6:

Our response should have stated that no additional safety grade active valves were added as a result of the modification.

Question 8, Part 1:

Our procedure for evaluating the high energy pipe rupture impact of the McGuire Steam Generator Modifications is considered equivalent to the requirements of the Standard Review Plan Section 3.6.2 MEB 3-1, Rev. 1, with regard to changing break locations as a result of piping reanalysis.

Break freeze criteria identical to that used in this instance is stated in Section 3.6.2.1.2.1 of the Catawba Nuclear Station FSAR, and we understand that this approach has been accepted by the NRC for Catawba.

Question 8, Part 2:

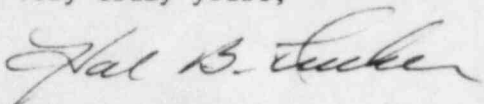
For the high energy piping associated with the McGuire Steam Generator Modification, the effects of postulated pipe failure and jet impingement were considered. Where unacceptable interactions with safety related targets existed in the spray paths shown on Figures CAP-A-1 thru 4 and CAO-D-1&2, rupture restraints are appropriately provided as outlined in Table 1.

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Should there be further questions in this matter, please advise.

Very truly yours,



Hal B. Tucker

PBN:jfw

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