NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | N C M G S 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 01 REPORT SOURCE L (6) | 0 5 0 0 0 3 6 9 (7) | 0 8 0 9 8 2 (8) | 0 9 0 8 8 2 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

While in mode 1, during an attempt to isolate a ground by sequentially removing and reenergizing loads, a reactor coolant pump motor was inadvertently deenergized which resulted in a reactor trip by the Reactor Protection System on a sensed loss of flow in reactor coolant loop A. This violates T.S.3.4.1.1 which is reportable per T.S.6.9.1.13(b). No unusual events were experienced during the transient accompanying the trip and trip recovery was commenced. The automatic reactor protection loss of flow trip function operated correctly in this incident. Health and safety of the public were unaffected.

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
C B (11)	B (12)	A (13)	C K T B R K (14)	A (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	0 6 5	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	F (19)	A (20)	C (21)	0 0 1 3	Y (23)	N (24)	L (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The incident was a result of design error, but personnel error contributed since operators failed to recognize the consequence of the loss of control power prior to directing the progression of the ground isolation procedure. A modification will be made to make the 125 VDC Aux. Control Power System (EPK) distribution boards compatible with the ground isolation methodology, and warnings have been posted until the modification is completed. Appropriate personnel will be instructed.

FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
B (28)	0 5 0 (29)	N/A	A (31)	Alarms and reactor trip

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION (39)
0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

NUMBER	DESCRIPTION (41)
0 0 0 (40)	N/A

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE	DESCRIPTION
Z (42)	N/A

PUBLICITY

ISSUED	DESCRIPTION (45)
N (44)	N/A

NRC USE ONLY

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