James A. FitzPatrick Nuclear Power Plant P.O. Box 41 Lycoming, New York 13093 315 342-3840



Harry P. Salmon, Jr. Resident Manager

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May 27, 1994 JAFP-94-0282

United States Nuclear Regulatory Commission Document Control Desk Mail Station P1-137 Washington, D.C. 20555

SUBJECT: DOCKET NO. 50-333 LICENSEE EVENT REPORT: LER-93-019-02:

Potential Design Inadequacies in the Control Room Ventilation System

Dear Sir:

This updated report is submitted in accordance with 10CFR50.73 (a)(2)(ii)(B). This is the final report which summarizes two previously reported conditions and a number of other Control Room Ventilation System adverse conditions, their potential impact on safety, and the corrective actions already taken or planned.

Questions concerning this report may be addressed to Mr. David A. Holliday at (315) 349-6359.

Very truly yours,

HARRY P. SALMON, JR.

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Enclosure

cc: USNRC, Region I USNRC Resident Inspector INPO Records Center

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This LER updates information on two previously reported conditions and describes other adverse conditions related to the Control Room Ventilation (CRV) System found in the past nine months. The plant has been operating at 100 percent power in a steady state condition except during three maintenance outages in September, November and April, 1994. As previously reported, engineering determined on 9/23/93 that an un-quantified amount of unfiltered air would flow into the Control Room (through a normally open manually operated "bypass" damper) with the CRV System in the "isolate" mode of operation with a single failure, specifically, of an isolation motor operated valve (MCV) to close. An update reported that on 10/29/93 engineering identified single failure concerns in the routing of control and power cables for the CRV System fans, dampers, and isolation MOVs. The other CRV related events or conditions generally fell into four categories: single failure concerns; degraded equipment conditions allowing outside air infiltration; inadequately balanced air flows resulting in filter train flow rates that exceeded Tech Spec limits; and inaccurate design and licensing basis documentation. The causes were: inadequate design control and documentation by engineering organizations; inadequate preventive maintenance; inadequate surveillance testing; and no comprehensive CRV design basis document. The system has been returned to an operable status.

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#### DESCRIPTION

In the past nine months, two previously reported conditions and a number of other conditions have been identified related to the Control Room [NA] Ventilation (CRV) System [VI]. This update to the LER describes those conditions, their potential effect on safety, and the corrective actions.

To summarize, LER-93-019-00 identified the first condition. On September 23, 1993, the plant was operating at 100 percent power in a steady state condition. After a series of inspections and evaluations, engineering determined that an un-quantified amount of unfiltered air would flow into the Control Room with the CRV System in the "isolate" mode of operation with a postulated single failure, specifically, of an isolation motor operated valve (MOV) to close. The isolation MOVs are in the air intake duct and in the air exhaust duct. (Refer to the attached simplified ventilation system flow diagram.) The flow would be through a normally open manually operated "bypass" damper installed in parallel with a motor operated damper which is in series with the isolation MOV. The motor operated damper automatically closes when the CRV System is placed in the "isolate" mode of operation to limit unfiltered air entering the Control Room. The intake and exhaust isolation MOVs were closed and sealed to eliminate this single failure concern and to eliminate any leakage past the isolation MOVs.

Revision 01 of LER-93-019 reported an additional single failure concern. On October 29, 1993, engineering determined that some control and power cables for safety-related CRV System fans, dampers, and isolation MOVs were incorrectly identified as non-safety related and were routed in common, non-safety related electrical raceways. The control cables for the fans and dampers were temporarily disconnected and the power supply breakers for the closed MOVs were opened.

The other CRV related conditions described in this update generally fell into four categories: single failure concerns; degraded equipment conditions allowing outside air infiltration; inadequately balanced air flows resulting in filter train flow rates that exceeded Technical Specification limits; and inaccurate design and licensing basis documentation.

An assessment of licensing and design bases for Control Room habitability is nearly complete. Actions have already been taken to correct hardware deficiencies and the CRV System is now operable in all modes of system operation. Additional actions are being taken to correct documentation deficiencies.

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The following describes in more detail the adverse conditions and corrective actions taken to date on the CRV System:

## Single Failure Concerns:

In July of 1993, engineering was evaluating the CRV System after a review of INPO OE 2465, Control Room Environmental Control System Single Failure. The OE identified a potential concern that the non-safety related portion of the CRV System may not swip over completely to a recirculation mode resulting in the inability to maintain positive pressure in the Control Room. A review of standards, regulations, design documents, surveillance tests and hardware by engineering resulted in questions regarding design and testing adequacy, as well as information contained in the 1981 NUREG-0737 (III.D.3.4) submittal.

During this review, a concern was raised over the existence of "bypass" dampers installed in parallel with the intake and exhaust dampers (and their purpose) since they were not described in plant documents. The bypass dampers were shown on an UFSAR figure. The original purchase specifications and vendor test reports for the isolation MOVs and dampers were reviewed to determine system design parameters. Since there was a possibility that the CRV System design would not meet assumptions regarding single failure criterion and since calculations associated with the 1981 NUREG-0737 submittal assumed a leakage rate that has not been verified by testing, the CRV System was placed in the "isolate" mode (the post LOCA lineup) on July 9, 1993. That action placed the CRV System in a condition that ensured it would perform its intended function. An Action Plan was developed to track and ensure resolution of this and other identified CRV related concerns.

In August 1993, the engineering organizations defined and verified the CRV System reference pressure parameters and evaluated the applicability of the single failure criterion. They also reviewed the 1981 NUREG-0737 submittal and re-evaluated compliance. It was during this review that, on September 23, 1993, engineering determined that the un-quantified amount of unfiltered air flowing into the Control Room (through the bypass damper) with the CRV System in the "isolate" mode ci operation with a postulated single failure, specifically, of an isolation MOV to close was reportable. This was reported in LER-93-019-00.

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During subsequent reviews, engineering identified another single failure concern on October 29, 1993. Some control and power cables for safetyrelated CRV System fans, dampers, and isolation MOVs were identified as non-safety related and were routed in common, non-safety related electrical raceways. The components affected were the two Control Room Filter Train Fans and their associated dampers, and the Intake and Exhaust Isolation MOVs (70MOV-107 and 108). The concern was identified following an electrical separation review by engineering of all major CRV System equipment and components. The CRV System was kept in the isolate mode so that actuation of the dampers and MOVs would not be required until a modification was completed which rerouted and relabeled the subject cables. An automatic swap-over feature for the two filter train fans and their associated discharge dampers has been temporarily defeated by a modification until a further evaluation and any required hardware changes could be completed. This was reported in LER-93-019-01. (The isolation MOV cables have since been rerouted.)

## Degraded Equipment Conditions:

On August 4, 1993, engineering discovered that the temperature indicators used to verify the operability of the CRV System as required by Technical Specifications (system capacity test) were not calibrated prior to the test. The temperature indicators were subsequently calibrated (they were within tolerance) and were added to the Preventive Maintenance Program.

On August 27, 1993, engineering personnel were measuring relative pressures between the Control Room and surrounding areas. The Control Room pressure was 0.14 inches of water lower than the Relay Room pressure and was 0.12 inches of water lower than outside atmosphere pressure. The Control Room pressure should have been higher than the other two pressures. Subsequent investigations revealed that the Control Room differential pressure controller measured pressure relative to the CRV equipment chiller room which is outside the CRV envelope. The chiller room pressure was found to be less than atmospheric which caused the controller output to be biased abnormally low. Steps were taken to disconnect the controller output and realign the modulating dampers to the positions which provided maximum Control Room pressure, restoring the proper differential pressure between the Control Room and all surrounding air spaces. Also, leaks were found in various locations on the Relay Room [NA] Ventilation System Filter Housing which is inside the CRV System envelope. The in-leakage was raising the Relay Room pressure. (The leaks have since been repaired.)

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The most recent concern was identified on February 24, 1994. Engineering noted that methods identified in the original CRV System design to limit relative humidity below 70 percent were not implemented during original plant construction. The original design had required a provision to recirculate a fraction of the CRV System charcoal filter train air supply to control humidity to less than 70 percent. According to Regulatory Guide 1.52, if relative humidity exceeds 70 percent, the filter effectiveness of 99 percent cannot be justified. Engineering found a 1974 calculation which assumed 99 percent effectiveness. This potential condition was reported to the NRC.

Subsequent review by engineering determined that the 1974 requirement for 99% effectiveness was based on a proposed configuration of the Main Steam Leakage Collection System (MSLCS) which was different from what was later installed in the plant. The requirement was not relevant and there was no adverse condition.

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The cause of this event appears to be inadequate design control and design documentation (Cause Code B) by engineering organizations, combined with inadequate preventive maintenance on the MOVs and dampers, and inadequate surveillance testing for in-leakage (Cause Code E).

The failure to recognize the deviations during subsequent reviews and the differences in flow characteristics was caused by a lack of a comprehensive CRV System design and licensing basis documentation (Cause Code B).

# EVENT ANALYSIS

The flow through the bypass dampers as a result of a postulated single failure was reportable under the provisions of 10CFR 50.73 (a)(2)(ii)(B) since the condition appeared to be outside of the current plant design basis. The purpose of maintaining a positive pressure in the Control Room during emergency operation is to ensure any leakage is out of (rather than into) the Control Room. The original Control Room Habitability Calculation did not allow for any unfiltered in-leakage.

Although there is insufficient data to quantify the amount of flows through the bypass damper and other leakage paths, engineering judgement is that the current Habitability Calculation (which assumes 2100 cfm) generally bounds the condition, based on flows and pressure drops obtained from recent CRV System capacity testing. The result of these calculations show that a control rod drop accident is the worst case scenario, with a thyroid dose of 6.26 rem. This is less than the regulatory exposure limits (from 10CFR50, Appendix A, GDC 19 and Standard Review Plan, Section 6.4) of 30 rem to the thyroid.

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EVENT ANALYSIS (continued)

To demonstrate that a higher infiltration rate would not significantly degrade habitability, another calculation assuming an infiltration rate of 15,000 cfm yielded a dose of 8.5 rem to the thyroid. (A main steam line break outside containment is not the worst case scenario based on the assumption that reactor coolant activity levels are less than or equal to 0.2 uCi/gm does equivalent I-131. FitzPatrick Technical Specification 3.6.C.1 limits equilibrium reactor coolant activity levels to 3.1 uCi/gm. NRC Standard Technical Specifications limit activity to 0.2 uCi/gm.)

Another evaluation was approved on April 20, 1994, (and revised May 11, 1994) which evaluated the habitability of the Control Room based on the current, repaired configuration and the acceptability of returning the CRV System to the normal mode of operation. Based on these habitability evaluations, the safety significance of this occurrence is low.

#### CORRECTIVE ACTIONS

- 1. The CRV System was placed in the "isolate" mode of operation. This was done as a conservative measure in anticipation that the evaluation might indicate excessive in-leakage. Completed July 9, 1993. Since then, the CRV System has been repaired and has been returned to the normal mode of operation during the April, 1994 maintenance outage.
- 2. A CRV System Action Plan has been developed, revised, and expanded which outlines further evaluations, reviews, and corrective actions to be taken. The plan, initially issued July 30, 1993, has been updated periodically and has been used as a tool in managing the CRV System discrepancies as they are identified and resolved.

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(5-72)	LICENSEE EVE TEXT CO	ESTIMAT THIS IN FORWARD THE IN (MNBB 7 WASHING REDUCTI MANAGEM	ED BURDEN PER IFORMATION COLLE COMMENTS REGA FORMATION AND F 714), U.S. NUCLI TON, DC 20555-0 ON PROJECT ENT AND BUDGET,	RESPONSE ECTION REQUENT RDING BURD RECORDS MA EAR REGULAT 001, AND T (3150-0104) WASHINGTON	TO COMPLY WITH UEST: 50.0 HRS. JEN ESTIMATE TO NAGEMENT BRANCH TORY COMMISSION, O THE PAPERWORK O, OFFICE OF N, DC 20503.				
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Jame: Powe:	s A. FitzPatrick M r Plant	Nuclear	05000333	YEAR 93	SEQUENTIAL NUMBER 019	NUMBER 02	09 OF 10		
TEXT (1	f more space is required, use	additional copies o	f NRC Form 366A) (1	7)					
CORR	ECTIVE ACTIONS (CO	ontinued)							
3.	The isolation MOV Engineering re-pe Room had a higher habitability in t	/s were tempo erformed the c differentia the Control F	prarily seale pressure tes al pressure, Room. Comple	ed and its an re-es ited S	verified d verifie tablishin eptember	leak- d the g the 25, 19	tight. Control 93.		
4.	4. In order to temporarily resolve the cable separation concern, the two isolation MOV power supply breakers were opened and cables associated with the filter train fans and dampers were disconnected at both ends. Since then, the MOV cable separation issues were resolved through a modification during the April, 1994 maintenance outage.								
5.	5. The repair of the three leaking isolation MOVs (as well as the fourth MOV) was completed during the April, 1994 maintenance outage. The seals were tested by the direct measurement and pressure decay method.								
6.	A ventilation exp discrepancies and balanced and a su Control Room rela	pert was hire assist with arveillance t ative to all	ed to identif balancing a cest confirme surrou.ding	y and irflo ed a p areas	resolve ws. The positive p	additi system ressur	onal was e in the		
7.	The Preventive Ma changed to incorr dampers. Estimat	aintenance Pr porate the CF ted completio	ogram evalua RV MOVs and, on date: Sep	if ne tembe	will be eded, the r 30, 199	review motor 4.	ed and operated		
8.	The CRV design ba Estimated complet	asis document tion date: S	has been re September 30,	viewe 1994	d and is	being	revised.		
9.	A revised NUREG-( completion date:	)737 submitta September 3	11 will be su 10, 1994.	ıbmitt	ed. Esti	mated			
ADDI	TIONAL INFORMATION	4							
Fail	ed Components:	None							
Prev	ious Similar Event	None							
Rela	ted Industry Expe	cience: INPC Cont	OE 2465 - C Trol System S	Contro Single	l Room En Failure,	vironm 3/16/	ental 88.		
		NRCM	86-76 - Pro gency Ventil	blems	Noted in Systems,	Contr 8/28/	ol Room 86.		

