SOUTH CAROLINA ELECTRIC & GAS COMPANY
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O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

March 10, 1983

Mr. James P. O'Reilly,
Regional Administrator
U.S. Nuclear Regulatory Commission
Region II, Suite 2900
101 Marietta Street, N.W.
Atlanta, Georgia 30303

SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Thirty Day Written Report
LER 83-012

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #83-012 for Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(b) as a result of entry into Action Statement 6 of Technical Specification 3.3.1, "Reactor Trip Instrumentation," Table 3.3-1, Item 14, on February 10, 1983.

Should there be any questions, please call us at your convenience.

Very truly yours,

O. W. Dixon, Jr.

CJM:OWD:dwf/fjc
Attachment

cc: V. C. Summer
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DETAILED DESCRIPTION OF EVENT

On February 10, 1983, at 2030 hours, with the Plant in Mode 1, the flow indicator for Steam Generator "A" Feedwater Flow (FI-477) failed low as compared to redundant channels. The channel was declared inoperable, and the associated bistables were placed in the tripped condition at 2125 hours in accordance with Action Statement 6 of Technical Specification 3.3.1, Table 3.3-1, Item 14.

PROBABLE CONSEQUENCES

There were no adverse consequences as a result of this event since the downscale failure of the instrument channel was in the safe direction. A reactor trip would still have occurred with a Steam Generator Water Level Low in coincidence with a Steam Flow/ Feedwater Flow mismatch.

CAUSE(S) OF THE OCCURRENCE

The cause of this occurrence is attributed to the failure of the power supply circuit board FQY-477. An electronic component failure on this circuit board caused the output to slowly decrease with a constant input.

IMMEDIATE CORRECTIVE ACTIONS TAKEN

The instrumentation channel for Steam Generator "A" Feedwater Flow was placed in the tripped condition within one (1) hour in compliance with Action Statement 6 of Technical Specification 3.3.1. The output of power supply circuit board (FQY-477) was found to slowly decrease with a constant input. The circuit board was replaced and the channel returned to operable status on February 11, 1983, upon the satisfactory completion of the appropriate surveillance test procedure.

ACTION TAKEN TO PREVENT RECURRENCE

The licensee plans no additional action in regard to this event other than the normal surveillance testing.