

. 17

Northeast Nuclear Energy

Rope Ferry Rd. (Route 156), Waterford, CT 06385

Millstone Nuclear Power Station Northeast Nuclear Energy Company P.O. Box 128 Waterford, CT 06385-0128 (203) 444-4300 Fax (203) 444-4277

The Northeast Utilities System

Donald B. Miller Jr., Senior Vice President – Millstone

Re: 10CFR50.73(a)(2)(4) 10CFR50.73(a)(2)(i)(B)

May 23, 1994 MP-94-360

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65 Docket No. 50-336 Licensee Event Report 94-009-00

Gentlemen:

This letter forwards Licensee Event Report 94-009-00 required to be submitted within thirty (30) days pursuant to 10CFR50.73(a)(2)(iv) and 10CFR50.73(a)(2)(i)(B).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Dohald B. Miller, Jr.

Senior Vice President – Millstone Station

DBM/RAB:dlr

310087

Attachment: LER 94-009-00

CC: T. T. Martin, Region I Administrator
 P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2 and 3
 G. S. Vissing, NRC Project Manager, Millstone Unit No. 2

FZK

9406010174 940523 PDR ADDCK 05000336 S PDR

NAME (1) DOCKET HUMBER (2) DOCKET HUMBER (2) <th< th=""><th></th><th>.1</th><th>LICEI</th><th>NSEE</th><th>U.S. NU EVENT RI ed number of digits/c</th><th>JOLEAF EPOF</th><th>R RE</th><th>(LER)</th><th>)</th><th>MMISS</th><th>ON ECBB SP 7</th><th>STIMATE OLLECT URDEN RANCH /ASHING /ASHING</th><th>APPI D BURDER ON REOL ESTIMATE (MNBB 7 TON, DC (3150-0 TON, DC 2</th><th>ROVED E EX I PER RES EST 50.0 TO THE II 714), U.S 20555-000 104), OFI 0503</th><th>PONSE TO HRS FO NFORMATI NUCLEJ MAND 1 FICE OF</th><th>NO. 3 5/31/9 ORWARE ON AND AR RE TO THE MANU</th><th>150-010 95 LY WITH THI D COMMEN D RECORDS GULATORY PAPERWO AGEMENT</th><th>IA S INFORM TS REGA MANAG COMMI RK REDU AND BI</th><th>AATIO RDIN EMEN SSION ICTIO JDGE</th></th<>		.1	LICEI	NSEE	U.S. NU EVENT RI ed number of digits/c	JOLEAF EPOF	R RE	(LER))	MMISS	ON ECBB SP 7	STIMATE OLLECT URDEN RANCH /ASHING /ASHING	APPI D BURDER ON REOL ESTIMATE (MNBB 7 TON, DC (3150-0 TON, DC 2	ROVED E EX I PER RES EST 50.0 TO THE II 714), U.S 20555-000 104), OFI 0503	PONSE TO HRS FO NFORMATI NUCLEJ MAND 1 FICE OF	NO. 3 5/31/9 ORWARE ON AND AR RE TO THE MANU	150-010 95 LY WITH THI D COMMEN D RECORDS GULATORY PAPERWO AGEMENT	IA S INFORM TS REGA MANAG COMMI RK REDU AND BI	AATIO RDIN EMEN SSION ICTIO JDGE		
TTM 16 Manual Reactor Trip and Technical Specification Non – Compliance US000030 1 0 4 4 Warual Reactor Trip and Technical Specification Non – Compliance Context (a) Dooler TAMER Dooler TAMER Werth Dev YEAR 10 EER NUMBER (b) REPORT DATE (f) OTHER PACILITIES INVOLVED (g) Marual Dev YEAR 10 Dooler TAMER Dooler TAMER Dooler TAMER Month Dev YEAR 10 Dooler TAMER Dooler TAMER Dooler TAMER Month Dev YEAR 10 Dooler TAMER Dooler TAMER Dooler TAMER Month Dev YEAR 10 Dooler TAMER Dooler TAMER Dooler TAMER Month Dev YEAR 10 Dooler TAMER Dooler TAMER Dooler TAMER Month Dev THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 3: (Check one or more) (TI TEXPECTED Dooler TAMER TEXPECTED TEXPECTED TEXPECTED TEXPECTED Dooler TAMER TEXPECTED Dooler TAMER TEXPECTED <t< th=""><th>FACILIT</th><th>Y NAME</th><th>(1)</th><th>Millet</th><th>ano Nuclear I</th><th>Power</th><th>C+-</th><th>ation 11</th><th>oit 0</th><th></th><th></th><th></th><th></th><th>DOCKET</th><th>NUMBER</th><th>1 (2)</th><th></th><th>PAGE (3</th><th></th></t<>	FACILIT	Y NAME	(1)	Millet	ano Nuclear I	Power	C+-	ation 11	oit 0					DOCKET	NUMBER	1 (2)		PAGE (3			
Description Description Description Description MORE TO BE TO BELLER NUMBER (0) PEOPT DATE (7) OTHER FACILITIES INVOLVED (8) MORE TO BE TO BELLER NUMBER (0) PEOPT DATE (7) OTHER FACILITIES INVOLVED (8) MORE TO BE TO BELLER NUMBER (0) PEOPT DATE (7) PEOPT DATE (7) DOCKET MUMBER (10) MORE TO BE TO BELLER NUMBER (0) PEOPT DATE (7) PEOPT DATE (7) DOCKET MUMBER (10) MORE TO BE TO BELLER NUMBER (10) PEOPT DATE (7) PEOPT DATE (7) DOCKET MUMBER (10) MORE TO BE TO BELLER NUMBER (10) PEOPT DATE (7) PEOPT DATE (7) PEOPT DATE (7) MORE TO BE TO BE TO BELLER NUMBER (10) PEOPT DATE (7) PEOPT DATE (7) PEOPT DATE (7) MORE TO BE TO B	TITLE (4	M	anual	Reacto	or Trip and Te	chnics	als	necific:	ation t	Von-I	Comp	lance			05000.	330		1 OF	4		
Description Description Description Description Description Description 1000000000000000000000000000000000000	EVE	NT DA	TE (5)	I		D (6)		PEDO	DET DA	TE (7)	T	neu ie c	OTH	ED EAC	ILITIES	INNO	VED (0)				
Od 23 94 94 0.09 00 05 23 94 PACUTY NAME DOOMET MUMBER 05000 MODE (R) 2 THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFB (s): (Check one or more) (11, 20.4020) 12.71(0) <td< td=""><td>MONTH</td><td>DAY</td><td>YEAR</td><td>YEAR</td><td>SEQUENTIAL NUMBER</td><td>REVISI</td><td>ON</td><td>MONTH</td><td>DAY</td><td>YEAR</td><td>FACILI</td><td colspan="5">DILITY NAME</td><td colspan="5">DOCKET NUMBER</td></td<>	MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISI	ON	MONTH	DAY	YEAR	FACILI	DILITY NAME					DOCKET NUMBER				
Operating 2 THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11	04	23	94	94	- 009 -	00		05	23	94	FACILI	CILITY NAME				DOCKET NUMBER					
EXPERT 2 <td>OPERATI</td> <td>NO</td> <td></td> <td>THIS</td> <td>REPORT IS BEI</td> <td>NG SU</td> <td>BM</td> <td></td> <td>IRSUA</td> <td>NT TO</td> <td></td> <td>FOLIER</td> <td>EMENT</td> <td>S OF 10</td> <td>CED A.</td> <td>Chac</td> <td>05000</td> <td>moral</td> <td>(4.4.)</td>	OPERATI	NO		THIS	REPORT IS BEI	NG SU	BM		IRSUA	NT TO		FOLIER	EMENT	S OF 10	CED A.	Chac	05000	moral	(4.4.)		
EVER OO 20-4064(110) Model(P) Strate(2)M 73 T(B) LEVEL (19) 0.00 20-4064(110) Model(P) Model(P) Strate(2)M Strate(2)M <td>MODE</td> <td>(9)</td> <td>2</td> <td>20</td> <td>402(b)</td> <td>140 00</td> <td>C2 1141</td> <td>20.405(c)</td> <td>UNDUM</td> <td>10</td> <td></td> <td>(50.7</td> <td>3(a)(2)()v)</td> <td>a OF TO</td> <td>orns.</td> <td>TT</td> <td>73.71(b)</td> <td>more)</td> <td>(11)</td>	MODE	(9)	2	20	402(b)	140 00	C2 1141	20.405(c)	UNDUM	10		(50.7	3(a)(2)()v)	a OF TO	orns.	TT	73.71(b)	more)	(11)		
Level: 119 000 20:405(a)(11)//a 80:38(c)(2) 60:73(a)(2)//a 00:73(a)(2)//a 00:73(a)(2)//a 00:73(a)(2)//a Image: State in the st	POWE	R		20	405(a)(1)(l)			50.36(c)/	5).			50.7	3(a)(2)(v)				73.71(c)				
Image: Supplemental REPORT EXPECTED (14) EXPECTED Submission Date Model and control of the control	LEVEL (10)	000	20	405(a)(1)(ii)			50.36(c)()	2)			50.7	3(a)(2)(vii)				OTHER				
Description Strate (2)(1) Strate (2)(1) Strate (2)(1) Strate (2)(1) Strate (2)(1) LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER (Incluse And Colspan="2">NUME DISTING (2)(2)(2)(2)(2)(2)(2)(2)(2)(2)(2)(2)(2)(20	405(a)(1)(fil)		X	50.73(a)(2)(1)			60.7	3(a)(2)(VIF)	(A)		(Spec	(Specity in Abstract				
International and the reactor in Mode 1 at 99.8% power, a planned plant shutdown was commoniced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that the Technical Specification action statement. Mover and Cech and provide Area Cooland Cech Art -65 immovable and logged into the applicable Technical Specification action statement. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 210 statement. At approximately 0035 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 210 statement. At approximately 0230 hours, it was identified that the Technical Specification action statement. At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA 7 -65 immovable and logged into the applicable Technical Specification action statement.				20.4	405(a)(1)(iv)			50.73(a)0	2)(II)			50.7	3(a)(2)(viil)	(B)		Form	and in Text, 366A)	NHC			
LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER REQUEST FOR THIS LER (12) TELEPHONE NUMBER REQUEST FOR THIS LER (12) TELEPHONE NUMBER REQUEST FOR THIS LER (12) COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) COMPONENT MANUFACTURER REPORTANCE X AAA JSUPPLEMENTAL REPORT EXPECTED (14) SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE X M® SUBMISSION DATE X M® SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE X M® SUBMISSION DATE X M® SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE X M® SUBMISSION DATE X M® SUBMISSION X MSEM MOMTH MANUFACTURER MOMTH MANUFACTURER MOMTH MANUFACTURER MOMTH				20	405(a)(1)(V)			50.73(a)(2) (6f)			50.7	3(a)(2)(x)			1					
TELEPHONE NUMBER INCLUSE AVE COMPONENT Philip J. Lutzi, Site Licensing COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) COMPONENT MANUFACTURER REPORTABLE SUPPLEMENTAL REPORT COMPONENT FAILURE DESCRIBED IN THIS REPORT MANUFACTURER X AA JS C490 Y MONTH DAY TE SUPPLEMENTAL REPORT EXPECTED (14) SUPPLEMENTAL REPORT EXPECTED (14) X YES MONTH DAY TE ABSTRACT (Umt to 1000 appease, Le approximately 19 angle-appead type=traine fine() (16) On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99 8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 – 65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 – 65 was at 21.43 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 0250 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. <td></td> <td>-</td> <td>and the second second second</td> <td></td> <td>LIC</td> <td>CENSEL</td> <td>EC</td> <td>ONTACT</td> <td>FORT</td> <td>HIS LE</td> <td>R (12)</td> <td></td> <td></td> <td></td> <td></td> <td></td> <td></td> <td></td> <td></td>		-	and the second second second		LIC	CENSEL	EC	ONTACT	FORT	HIS LE	R (12)										
Philip J. Lutzi, Site Licensing (203) 447–1791 Ext. 6585 COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) CAUSE X AA JS CAUSE SYSTEM COMPONENT MANUFACTURER TO NERDO X AA JS COMPONENT MANUFACTURER TO NERDO SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUPPLEMENTAL REPORT EXPECTED (14) X MS SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED X MS SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE) NO ABSTRACT COMPONENT MANUFACTURER SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED MUMONTH DAY X MS	NAME.				-										TELEPH	IONE NI	UMBER (Incl.	Ide Area I	ode)		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) CAUBE SYSTEM COMPONENT MANUFACTURER REPORTABLE CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE X AA JS C490 Y Image: System COMPONENT MANUFACTURER TO NPROD X MA JS C490 Y Image: System COMPONENT MANUFACTURER TO NPROD X ME SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED MONTH Day YE X MES SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE 06 30 S ABSTRACT LIDE4 NO Date (15) 06 30 S Control 120 SubMission DATE NO SubMission DATE 06 30 S ABSTRACT LIDE4 Reproduct as not inserting. The Shift Supervisor had the indication investigated and declared the read switch CEA position indicator channel for CEA 7 ~65 inoperable at 2143 ho		F	nilip J	. Lutzi,	Site Licensir	ng									(203)	447	-1791	Ext. 68	585		
CAUSE SYSTEM COMPONENT MANUFACTURER PEPORTABLE TO NPROF CAUSE SYSTEM COMPONENT MANUFACTURER PEPORTABLE TO NPROF X AA JS C490 Y Image: System COMPONENT MANUFACTURER REPORTABLE TO NPROF X AA JS C490 Y Image: System COMPONENT MANUFACTURER REPORTABLE TO NPROF X Max SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE Image: System MONTH DATE Image: System OG SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE Image: System Image: System DATE Image: System OG SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE Image: System Image: System OG System OG SUP SUP Image: System OG OG SUP Image: System OG SUP System Image: Sy			CON	MPLETE	ONE LINE FOR	REACH	CC	MPONE	NT FAI	LURE	ESCR	IBED I	N THIS	REPORT	(13)						
X AA JS C490 Y SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED MONTH DAY YE X YES NO DATE (15) 06 30 SC ABSTRACT (Unit to 1400 appose, Le. approximately 16 angle- approximately 16 angle- approximately (16) NO DATE (15) 06 30 SC ABSTRACT (Unit to 1400 appose, Le. approximately 16 angle- approximately 10 NO DATE (15) 06 30 SC ABSTRACT (Unit to 1400 appose, Le. approximately 16 angle- approximately (16) On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 - 65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 - 65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 10 steps, the operators began to suspect that CEA 7 - 65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7 - 65 immovable and logged into the applicable Technical Specification action statement. At approximately 0230	CAUSE	SYSTE		PONENT	MANUFACTURE	R T	EPOR	PRDS		c	AUSE	SYSTE	м соя	PONENT	MA	ANUFACTURER		REPOR TO N	RDS		
SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED X YES (If yes, complete EXPECTED SUBMISSION DATE) No DATE (15) D6 30 SC ABSTRACT (Unit to 1400 space, Le, approximately 16 aring=-spaced typewritten lines) (16) No DATE (15) D6 30 SC ABSTRACT (Unit to 1400 space, Le, approximately 16 aring=-spaced typewritten lines) (16) On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 –65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 –65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 110 steps, the operators began to suspect that CEA 7 –65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7 –65 immovable and logged into the applicable Technical Specification action statement. At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At 0250 hours with the reactor in Mode 2 at approximately 10 ⁻⁵ % power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7 –65), pow	X	AA		JS	C490		Y														
SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE X Yes of yes, complete EXPECTED SUBMISSION DATE) No Date (15) 06 30 s ABSTRACT (Limit to 1400 epices, Le, approximately 15 angle = spaced typewritten lines) (16) On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 – 65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 – 65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 110 steps, the operators began to suspect that CEA 7 – 65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7 – 65 immovable and logged into the applicable Technical Specification action statement. At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At 0250 hours with the reactor in Mode 2 at approximately 10 ^{-5%} power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7 – 65), power was removed to the Control Element Drive Mechanism (CEDM) for CEA 7 – 65 and the CEA fully inserted into the core. At 0251 hours the reactor was manually tr							ant by same		10-10-2011-000-10					an minute and and a							
YES NO SUBMISSION DATE (15) 06 30 S ABSTRACT (Umit to 1400 spaces; Le. approximately 16 single-spaced typewritten lines) (16) 0n April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 – 65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 – 65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 110 steps, the operators began to suspect that CEA 7 – 65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7 – 65 immovable and logged into the applicable Technical Specification action statement. At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At 0250 hours with the reactor in Mode 2 at approximately 10 ⁻⁵ % power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7 – 65), power was removed to the Control Element Drive Mechanism (CEDM) for CEA 7 – 65 and the CEA fully inserted into the core. At 0251 hours the reactor was manually tripped.					SUPPLEMENT	TAL REI	POF	RTEXPE	CTED ((14)	areas and a second			F	XPECTE	=D	MONTH	DAY	YEA		
ABSTRACT (Umit to 1400 spaces, i.e. approximately 15 single-spaced typewritten lines) (16) On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7–65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7–65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 110 steps, the operators began to suspect that CEA 7–65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7–65 immovable and logged into the applicable Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time.	XYE	IS	niete EVD				1		NO					SU	BMISS	ON	06	30	94		
 On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, it was identified that Control Element Assembly (CEA) 7 – 65 was indicated as not inserting. The Shift Supervisor had the indication investigated and declared the reed switch CEA position indicator channel for CEA 7 – 65 inoperable at 2143 hours and logged into the applicable Technical Specification action statement. The downpower was recommenced at 2207 hours. At approximately 0055 hours on April 23, 1994 with reactor power at approximately 25% and CEA Group 7 at approximately 110 steps, the operators began to suspect that CEA 7 – 65 was at the fully withdrawn position. At 0115 hours the Shift Supervisor declared CEA 7 – 65 immovable and logged into the applicable Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At 0250 hours with the reactor in Mode 2 at approximately 10⁻⁵% power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7 – 65), power was removed to the Control Element Drive Mechanism (CEDM) for CEA 7 – 65 and the CEA fully inserted into the core. At 0251 hours the reactor was manually tripped. 		YMM, LOOPTI	DIEIR EAPI	CONED SO	SMISSION DALE)			-	_						SWIE (11	21					
At approximately 0230 hours, it was identified that the Technical Specification action statement requirements for an immovable CEA had not been performed within the specified time. At 0250 hours with the reactor in Mode 2 at approximately 10 ⁻⁵ % power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7–65), power was removed to the Control Element Drive Mechanism (CEDM) for CEA 7–65 and the CEA fully inserted into the core. At 0251 hours the reactor was manually tripped.	ABSTR	April	(Limit to 1) 22, 19	400 spaces 94, at 2	2100 hours, v	5 single-	e p	ed typewritt lant in 1	en lines) (Mode	16) 1 at 9	9.8%	oowe	r, a pla	nned p	lant sh	utdo	wn was				
At 0250 hours with the reactor in Mode 2 at approximately 10^{-5} % power and Group 7 CEAs at approximately 90 steps (with the exception of CEA 7-65), power was removed to the Control Element Drive Mechanism (CEDM) for CEA 7-65 and the CEA fully inserted into the core. At 0251 hours the reactor was manually tripped.	ABSTF On cor inve and 220 At a app 011 Spe	April April April April April Appro Appro Shou Shou Scifica	(Limit to 1/ 22, 19 ced to Element ted an red into urs. ximate nately urs the ation ad	940 spaces 94, at 2 repair at Asse d decl o the a hly 005 110 ste Shift S ction si	a degraded mbly (CEA) 7 ared the reec pplicable Tec bours on Ap ps, the oper supervisor de tatement.	single- vith the Reacto 7-65 v d switc chnical pril 23 ators b clarec	e p or (was h C Sp g g g g g	ed typewritt lant in 1 Coolant s indica DEA po: Decifica 094 with jan to s EA 7-6	en lines) (Mode t Pump ted as sition i tion a tion a n reac uspec 55 imm	1 at 9 p seal s not in indica ction s tor po t that novab	9.8% j At aj hsertir tor ch tatem wer at CEA 1 le and	powe pprox ng. Ti annel ent. appr 765 1 logg	r, a pla imately for CE The do oximat was at led into	nned p / 2113 t Super A 7 – 6 wnpow ely 25% the ful the ap	lant sh hours, visor h 5 inope ver was 6 and (ly with oplicab	utdo it wa bad th erable s reco CEA draw le Te	wn was is identi e at 214 ommeni Group 7 n positio chnical	fied th ation 3 hou ced at dat on. At	at rs		
	ABSTF On Cor Cor inve anc 220 At a app 011 Spe At a an i	April April April April Bastiga Logg 7 hou proxim 5 hou proxim 5 hou appro immo	(Limit to 1/ 22, 19 ced to Elemer ted ant ed into urs. ximate hately urs the ation ac vable (9400 spaces 94, at 2 repair nt Asse d decl o the a hly 0055 110 ste Shift S ction si hly 0236 CEA ha	2100 hours, v a degraded mbly (CEA) 7 ared the reed pplicable Tec 5 hours on Ap aps, the open Supervisor de tatement. 0 hours, it wa ad not been p	single- vith the Reacto 7-65 v d switc shnical pril 23, ators b oclared sciared	e p or (was h C I Sp g d C I tific	ed typewritt lant in 1 Coolant s indica Decifica 294 with jan to s EA 7 – 6 ed that d within	en lines) (Mode t Pump ted as sition i tion at tion ac uspec 55 imm the Te o the s	1 at 9 p seal s not in indica ction s tor po t that novab echnici pecifie	9.8% j At aj hsertir tor ch tatem wer at CEA 1 le and al Spe ad tim	powe pprox ng. Ti annel ent. appr 765 1 logg ecifica e.	r, a pla imately he Shif for CE The do oximat was at ed into tion ac	nned p / 2113 t Super A 7 – 6 wnpow ely 259 the ful the ap	lant sh hours, visor h 5 inope ver was 6 and (ly with oplicab	utdo it wa had th erablis reco CEA drawi le Tei nt req	wn was is identi e at 214 ommeni Group 7 n positio chnical uiremer	fied th ation 3 hou ced at 2 at on. At	at rs		
	ABSTF On cor Cor inve anc 220 At a app 011 Spe At a an i At a for	April April April Inmen Introl E estiga I logg 7 hou proxim 5 hou ecifica appro immo 250 h steps CEA	Lumit to 1/ 22, 19 ced to Element ted an used into urs ximate values the value value value (with t 7-65 a	^{400 spaces} 94, at 2 repair at Asse d decl o the a ly 005 110 ste Shift S ction si ly 023 CEA ha with the exc and the	a degraded mbly (CEA) 7 ared the reed pplicable Tec bours on Ap ps, the open supervisor de tatement. D hours, it wa ad not been p e reactor in N eption of CE b CEA fully in	single- vith the Reactor 7-65 v d switc chnical pril 23, ators t polarec solarec solarec A 7-6 serted	e p or (was h C I Sp d C i sp d C i sp d C i sp i nt	ed typewritt lant in 1 Coolant s indica DEA poi becifica 294 with an to s EA 7 - 6 ed that d within approx power to the c	en lines) (Mode t Pump ted as sition i tion a uspec 55 imm the Te the s the s was n ore. A	1 at 9 p seal s not in indica ction s tor po t that novab cchnic pecific ly 10 emove	9.8% j At a hsertir tor ch tatem wer at CEA 1 le and al Spe d to t 1 hour	powe pprox ng. Ti annel ent. appr 7-65 l logg cifica e. ower i he Co rs the	r, a pla imately he Shif for CE The do oximat was at ed into tion ac and Gr pontrol E reacto	nned p / 2113 t Super A 7 – 6 wnpow ely 259 the ful the ap tion sta	lant sh hours, visor h 5 inope ver was 6 and 0 ly with oplicab atemen DEAs a t Drive nanual	utdo it wa had th erables reco CEA draw le Te t req t app Meci lly trip	wn was is identi e at 214 ommeno Group 7 n positio chnical uiremen proxima hanism	fied th ation 3 hou ced at 7 at on. At nts for tely (CED)	at rs VI)		
	ABSTF On cor Coi inve anc 220 At a app 011 Spe At a an i At 0 90 for	April April April April I logg 1 logg	Lumit to 1/ 22, 19 ced to Element ted an red into urs the ation at value value tion at value value (with t 7-65 a	94, at 2 repair at Asse d decl o the a ly 005! 110 ste Shift S ction si ly 023! CEA ha with the the exc and the	a degraded mbly (CEA) 7 ared the reed pplicable Tec bours on Apps, the open supervisor de tatement. Thours, it wa ad not been p e reactor in M eption of CE of CEA fully in	single- vith the Reacter 7-65 v I switc chnical pril 23, actors b polarec sclarec volarec volarec sclarec sclarec	e p or (was h C Sp c g d C tific mec tific (5), int	ed typewritt lant in 1 Coolant s indica Decifica 994 with an to s EA 7 – 6 ed that d within approx power to the c	en lines) (Mode t Pump ted as sition i tion a uspec 55 imm the Te o the s imate was n ore. A	1 at 9 p seal s not in indica ction s tor poo t that novab echnic pecific ly 10 emove At 025	9.8% j At a hsertin tor ch tatem wer at CEA 7 le and al Spe ed tim 5% po d to t 1 hour	powe pprox ng. Ti annel ent. appr 7-65 1 logg cifica e. be Co rs the	r, a pla imately he Shif for CE The do oximat was at ed into tion ac and Gr potrol E reacto	nned p / 2113 t Super A 7 – 6 wnpow ely 25% the ful the ap tion sta	lant sh hours, visor h 5 inope ver was 6 and (ly with oplicab atemen CEAs a t Drive nanual	utdo it wa had the rables reco CEA drawi le Tei treq treq treq tapp Meci lly trip	wn was is identi e at 214 ommeno Group 7 n positio chnical uiremen proximat hanism oped.	fied th ation 3 hou ced at at on. At hts for (CED)	at rs VI)		
	ABSTF On cor Col inve and 220 At a app 011 Spe At a an i At a for	April April April April 1 ogg 7 hou 2007 hou 200	Lumit to 1/ 22, 19 ced to Element ted ant urs the tion action vable tours to (with to 7 - 65 a	94, at 2 repair at Asse d decl o the a ly 005 110 ste Shift S ction s ly 023 CEA ha with the she exc and the	2100 hours, v a degraded mbly (CEA) 7 ared the reed pplicable Teo 5 hours on Ap ps, the open Supervisor de tatement. 0 hours, it wa ad not been p e reactor in M eption of CE e CEA fully in	^{5 single-} vith thi Reacto 7 - 65 v I switc shnical oril 23, ators b oclared sciared	e p or (was h C i Sp g g d C i tifie med tat ;5), int	ed typewritt lant in 1 Coolant s indica Decifica 294 with jan to s EA 7 – 6 ed that d within approx power to the c	en lines) (Mode t Pump ited as sition i tion au n react uspec 55 imm the Te the s timate was n ore. A	1 at 9 p seal s not in indica ction s tor po t that novab echnica pecific emove At 025	9.8% j At a hsertir tor ch tatem wer at CEA 1 le and al Spe ed tim 5% po d to t 1 hour	powe pprox ng. Ti annel pent. appr 7-65 l logg cifica e. ower a he Co rs the	r, a pla imately he Shif for CE The do oximat was at ed into tion ac and Gr pontrol E reacto	nned p / 2113 t Super A 7 – 6 wnpow ely 259 the ful the ful the ap tion sta	lant sh hours, visor h 5 inope ver was 6 and (ly with oplicab atemen DEAs a t Drive nanual	utdo it wa had th erablis reco CEA drawi le Tei t req t app Meci lly trip	wn was is identi- ne indici e at 214 ommeno Group 7 n positio chnical uiremer proxima hanism oped.	fied th ation 3 hou ced at at on. At nts for tely (CEDI	at rs VI)		

NRC For (5-92)	n 366A U.S. NUCLEAR REGULAT LICENSEE EVENT REPORT (I TEXT CONTINUATION	TORY COMMISSION	ESTIMAT COLLECT BURDEN BRANCH WASHING PROJECT WASHING	APP ED BURD NON REC ESTIMAT (MNBB STON DX (3150- STON, DC	ROV EN PE DUEST E TO 7714) 0 205 -0104 : 2050	ED BY EXPI ER RESPO T 50.0 H THE INF . U.S 555-0001 A) OFFI 3.	OMB RES: DNSE T IRS I ORMA NUCU AND CE O	NO. 3150 5/31/95 0 COMPLY W FORWARD CO TION AND RE EAR REGUL TO THE PAR F MANAGEN	-0104	INFORMA REGAR MANAGEN COMMISI (REDUC ND BUE	ITION DING MENT SION TTION DGET,		
FACILITY NA	ME (1)	DOCKET NUMBER (2)			LE		R (6)	DEVACION	PAC	RE (3)			
1	Aillstone Nuclear Power Station Jnit 2	05000336		YEAR 94		NUMBER	_	NUMBER 00	02	OF	04		
TEXT (if m I.	ore space is required, use additional copies of NRC Form 366A) (17) Description of Event												
	On April 22, 1994, at 2100 hours, with the plant in Mode 1 at 99.8% power, a planned plant shutdown was commenced to repair a degraded Reactor Coolant Pump seal. At approximately 2113 hours, when control rods were being inserted for core power distribution control, it was identified that Control Element Assembly (CEA) 7–65 was indicating that it was not inserting.												
	The downpower was stopped and troubl activity indicated that CEA 7–65 was mo indicator channel had become "magneti that the reed switch channel was inopera indicator channel for CEA 7–65 inoperal Specification action statement. The dow	Inpower was stopped and troubleshooting of CEA 7-65 was performed. This troubleshooting indicated that CEA 7-65 was movable and it was believed that the reed switch CEA position richannel had become "magnetized" during the long (greater than 150 days) operating run, and reed switch channel was inoperable. The Shift Supervisor declared the reed switch CEA position richannel for CEA 7-65 inoperable at 2143 hours and logged into the applicable Technical ation action statement. The downpower was recommenced at 2207 hours using boration.											
	At approximately 2310 hours, reactor power was less than 70% power as required by the Technical Specification action statement for an inoperable reed switch CEA position indicator channel. The operators began inserting control rods for power distribution control, believing that CEA 7–65 was actually inserting and that the problem was an inoperable reed switch indicator channel.												
	At approximately 0055 hours on April 23, 1994, with reactor power at approximately 25 % and CEA Group 7 at approximately 110 steps (fully withdrawn position is 176 steps), the operators began to suspect that CEA 7–65 was at the fully withdrawn position. This was based on observed deviations between the four Reactor Protection System (RPS) channels for Nuclear Instrument (NI) power and Axial Shape index (ASI) indications. The approximately 35.												
	At 0115 hours the Shif. Supervisor declared CEA 7–65 immovable and logged into the applicable Technical Specification action statement. At approximately 0150 hours, the Reactor Engineer had confirmed that CEA 7–65 was fully withdrawn based on information from the fixed incore detector monitoring system.												
	Technical Specification Action Statement	t (TSAS) 3.1.3.1.a	states:										
	"With one or more full length CEAs inope mechanical interference or known to be of Specification 3.1.1.1 is satisfied within	erable due to bein untrippable, deter 1 hour and be in	g immo mine th at least	ovable lat the HOT	as SH STA	a resu IUTDC ANDBY	ilt of WN (with	excessiv MARGIN hin 6 hou	e fricti requi rs."	on or remer	nt		
	Contrary to the above requirements, no is requirements were satisfied within one higuidance to allow the operators to perfor Shift Supervisor at approximately 0230 his shutdown margin calculation. Technical	actions had been iour of declaring C rm this determinat iours that he had i Specification 3.0.	perform EA 7 – I tion, an nsuffici 3 was r	ned to 65 imr d the f ent inf not ent	ens nov Rea form tere	sure th able. Ictor E nation id.	hat th Thei ngin avai	ne shutdo re was no eer had i lable to p	own m o proc nform perform	argin edura ed the n the	.i		
	At 0250 hours with the reactor in Mode 2 approximately 90 steps (with the excepti Drive Mechanism (CEDM) for CEA 7–65 reactor was manually tripped, and opera "Standard Post Trip Actions". All safety in a stable condition.	2 at approximately ion of CEA 7–65), 5 and the CEA fully ators then perform related equipment	10 ⁻⁵ % power inserte ed Eme respor	powe was ro ed into ergeno nded a	er an emo the cy C as e	nd Gro oved to e core)perati xpecto	o the . At ing P ad at	7 CEAs a Control 0251 hou Procedure nd the un	t Elerne irs the 2525 it was	ent i, place	эd		

NRC (5-9	NRC Form 366A (5-92)				APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION									
	LICENSEE EVENT REPORT (TEXT CONTINUATION	LER)	COLLECT BURDEN BRANCH WASHING PROJECT WASHING	ED BORD RON REC ESTIMAT (MNBB 3TON, DC (3150- 3TON, DC	DUEST 50.0 F E TO THE INI 7714). U.S 20555-0001 -0104). OFFI 20503.	ARS FORMA NUCL AND CE C	FORWARD CO TION AND RE EAR REGUL TO THE PAI OF MANAGEN	CORDS CORDS ATORY PERWORI MENT A	N REGAR MANAGE COMMIS K REDUC ND BU	RDING MENT SION CTION IDRET				
FACILIT	YNAME (1)	DOCKET SUMBER (2)			LER NUMBE	ER (6)		PA	GE (3)					
1.				YEAR	SEQUENTI	AL.	REVISION NUMBER							
	Millstone Nuclear Power Station Unit 2	05000326		94	- 009	-	00	03	OF	04				
TEXT	(If more space is required, use additional copies of NRC Form 366A) (17))												
11.	Cause of Event													
	The cause of the failure of CEA 7-65 to intermittent failure of one (of three) silico switch. This failure caused the upper gr therefore did not permit CEA 7-65 to be	insert upon dema on controlled rectifi ipper coil to remai e inserted.	nd is po ers (SC n energ	ostulat Rs) ir jized t	ed to hav the uppe hrc ughou	ve be er gri ut the	en cause ipper coll e insert se	ed by powe guen	an Ir ce an	id				
	When the Coil Power Programmer (CPP April 23, 1994, the upper gripper coil de subsequently cooled down and the CED activities performed on April 23, 1994.) circuit breaker fo —energized, and r)M for CEA 7—65 f	r CEA 7 esulted unction	7-65 i in CE ied pro	was open A insertic operly du	ie dia on แต ring	at 0250 or to the cor troublest	n re. Th nootin	e SCI g	R				
	The decision to initiate a manual reactor trip following the opening of the CPP circuit breaker tor CEA 7–65 was made by the Shift Supervisor, the Duty Officer, Reactor Engineer and Operations Manager. The purpose of the manual reactor trip was to place the reactor in a safe, stable condition and to ensure that shutdown margin requirements were satisfied.													
	The reason for the operators not recogn the reed switch CEA position indication a highly experienced instrument and Co reed switches have historically been obs contributing cause to this event was inac	izing that CEA 7— channel was inope introls Specialist h served to "magnet dequate troublesh	65 was erable. ad perf ize" or ooting a	immo This b ormec "hang and la	vable was belief was the trout up" after ck of a fo	s du bas blest a lo rmal	e to their ed upon nooting, a ng opera troubles!	belief the fai and thi ting ru hootin	that ct that at the in. A ig pla	t n.				
	The contributing and root causes of the an immovable CEA are currently being i cause analysis. The results of this invest	failure to fully com nvestigated by a d tigation will be pro	ply with ledicate wided in	n the a ed tear n a su	ction stat n of pers pplement	terne onne tal re	ent require el trained eport.	ement I in roc	s for					
Ш.	Analysis of Event													
	The manual reactor trip is being reporter "Reporting any event or condition that re Feature System".	The manual reactor trip is being reported pursuant to the requirements of Paragraph 50.73(a)(2)(iv), "Reporting any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature System".												
	There was no safety consequence from the manual reactor trip event. All safety related equipment responded as expected and plant operators executed applicable Emergency Operating Procedures accordingly.													
	The failure to fully comply with action sta requirements of Paragraph 50.73(a)(2)(i Specifications."	atement requireme)(B), "Any operatic	ents is b in or co	eing r nditio	eported p n prohibit	ours ed b	uant to th by the pla	e nt's Te	chnical	al				
	There was no safety consequence as a requirement for an immovable CEA. Th	result of the failure is conclusion is ba	to fully used on	comp the fo	ly with th llowing:	ie ac	tion state	ment						
	 (a) The reactor shutdown was completed Specification LCO 3.0.3 (though not period of time that CEA 7-65 was 	ted in a timely fash at logged into by th declared immovat	nion and ne oper ple.	d the r ators)	equireme were bei	nts i ng si	of Technic atisfied d	cal uring t	the					
	(b) CEA 7-65 was ultimately determine	ed to be trippable	by rem	oving	power fro	om it	s CEDM.							
IV,	Corrective Action													
	The upper gripper power switch module to the vendor for failure analysis.	e has been replace	ed for C	EA 7-	65. The	pow	ver switch	will b	e sen	t				

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION					APPROVED BY OMB NO. 3150-0104 EXPIRES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATIO COLLECTION REQUEST 50.0 HRS FORWARD COMMENTS REGARDIN BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMEN BRANCH (MNB8 7714), U.S. NUCLEAR REGULATORY COMMISSIO WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTIO PROJECT (\$150-0104), OFFICE OF MANAGEMENT AND BUDGE WASHINGTON, DC 20503.								
FACILITY	NAME (1)		T	DOCKET NUMBER (2)			LER NUMBER	(6)	PA	GE (3)			
						YEAR	SEQUENTIAL NUMBER	REVISION NUMBER					
	Millstone Nuclea Unit 2	r Power Station		05000336		94	- 009	- 00	04	OF	04		
	Procedure AOI Specification a Training on the corrective actio investigation.	alfunctions" requirement lure will be c lemented ba	is being revised ts for CEA posit ompleted prior sed upon the fil	d to prov ion and to perfor ndings a	ide gu CEA p ming nd rea	uidance for position inc reactor sta commenda	r each of th dicator char artup. Addi ations of the	e Tech nnels. tional e root	caus	θ			
V.	Additional Info	rmation											
	Similar LERS:												
	Manual Reactor Trips: 91-012, 91-004, 90-006, 84-012												
	Action Statement non-compliance: 93-002, 92-009, 90-002, 89-009. All of these events involved a failure to log into Technical Specification action statements. The corrective actions in each of these events were unique to the event												
	EIIS Codes for Control Elemen Reed Switch C CEA Motion Ini Reactor Protect Nuclear Instrum Incore Detecto Control Elemen Reactor Coolan Silicon Control Coil Power Pro	referenced com nt Assembly: EA position indi hibit circuit: ction System: mentation: r Monitoring System nt Drive Mechan nt System: led Rectifier: ogrammer:	AA-ROD AA-DCC JC-C490 IG-C490 IG-C490 stem: IG hism: AV AB-C490 AA-SCR- AA-STC-	- C490 A - EIS - C490 - C490 A - 75 - C490 - C490 - C490									
	The following component failed during this event:												
	CEA 7 - 65 Upper gripper power switch: AA-JS-C490												
		A											
	Manufacturer:	Combustion E	ingineering										
	Manufacturer: Model No.:	N 9018, Rev. 1	ngineering										