VESSEL HEAD LIFT RIG, REACTOR VESSEL INTERNALS
LIFT RIG, LOAD CELL, LOAD CELL LINKAGE AND
REACTOR COOLANT PUMP MOTOR LIFT SLING
TO THE REQUIREMENTS OF NUREG 0612
for
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNITS 3 AND 4

DECEMBER, 1982

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ABSTRACT

An evaluation of the Turkey Point Units 3 and 4 reactor vessel head and internal lift rigs, load cell, load cell linkage and reactor coolant pump motor lift sling was performed to determine the acceptability of these devices to meet the requirements of NUREG 0612. The evaluation consists of: (1) a comparison report of the ANSI N14.6 requirements and the requirements used in the design and manufacture of these devices; (2) a stress report in accordance with the design criteria of ANSI N14.6; and (3) a list of recommendations to enable these devices to demonstrate compliance with the intent of NUREG 0612 and ANSI N14.6.

TABLE OF CONTENTS

Section	Title	Page
	ABSTRACT	111
1.	INTRODUCTION	1-1
	1.1 Background	1-2
2.	CUMPONENT DESCRIPTION	2-1
	2.1 Reactor Vessel Head Lift Rig	2-1
	2.2 Reactor Vessel Internals Lift Rig	2-1
	2.3 Load Cell and Load Cell Linkage	2-2
	2.4 Reactor Coolant Pump Motor Lift Sling	2-2
3.	SCOPE OF EVALUATION *	3-1
	3.1 Study of ANSI N14.6-1978	3-1
	3.2 Stress Report	3-1
	3.3 Recommendations	3-2
4.	DISCUSSION OF EVALUATIONS	4-1
	4.1 Study of ANSI N14.6-1978	4-1
	4.2 Stress Report	4-1
	4.3 Recommendations	4-2
5.	CONCLUSIONS	5-1
6	RECOMMENDATIONS	6-1

LIST OF ILLUSTRATIONS

Figure	Title	Page
2-1	Reactor Vessel Head Lifting Rig	2-3
2-2	Reactor Vessel Internals Lifting Rig	2-4
2-3	Reactor Coolant Pump Motor Lift Sling	2-5

ATTACHMENTS

- A. Comparison of ANSI N14.6-1978 Requirements for Special Lifting
 Devices and the Requirements for the Reactor Vessel Head Lift Rig,
 Reactor Vessel Internals Lift Rig, Load Cell, Load Cell Linkage and
 Reactor Coolant Pump Motor Lift Sling for Florida Power and Light
 Company, Turkey Point Units 3 and 4.
- B. Stress Report Reactor Vessel Head Lift Rig, Reactor Vessel Internals Lift Rig, Load Cell, Load Cell Linkage and Reactor Coolant Pump Motor Lift Sling for Florida Power and Light Company, Turkey Point Units 3 and 4.

REFERENCES

- George, H., Control of Heavy Loads at Nuclear Power Plants <u>Resolution of Generic Technical Activity A-36</u>, NUREG 0612, July, 1980.
- ANSI N14.6-1978 Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Material
- ANSI B 30.9-1971. Slings, American National Standards Institute, New York, 1971
- 4. Westinghouse Drawing 685J249, Head Lifting Rig General Assembly
- Westinghouse Drawing 685J291, Turkey Point Units 3 and 4 Internals Lifting Rig General Assembly
- 6. Westinghouse Drawing AED-SK-618J644TXK, Handling Sling Spreader for Motor SV-4M-Al Pump General Assembly

COMPARISON OF ANSI N14.6-1978 REQUIREMENTS FOR SPECIAL LIFTING DEVICES AND THE REQUIREMENTS FOR THE REACTOR VESSEL HEAD LIFT RIG, REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL, LOAD CELL LINKAGE AND REACTOR COOLANT PUMP MOTOR LIFT SLING

FOR

FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNITS 3 AND 4

December 1982

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ABSTRACT

The requirements used in the original design, fabrication, testing, maintenance and quality assurance were compared to the ANSI N14.6-1978 requirements for the Turkey Point Units 3 and 4 reactor vessel head and internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling. A critical items list per section 3.1.2 has been prepared and a tabulation of ANSI N14.6 requirements that are, at present, incompatible with the Turkey Point Units 3 and 4 lifting devices has been prepared.

TABLE OF CONTENTS

Section	Title	Page
	ABSTRACT	iii
1	PURPOSE	1-1
2	INTRODUCTION	2-1
	2.1 Background	2-1
	2.2 Component Description	2-2
	2.2.1 Reactor Vessel Head Lift Rig	2-2
	2.2.2 Reactor Vessel Internals Lift Rig	2-2
	2.2.3 Load Cell and Load Cell Linkage	2-3
	2.2.4 Reactor Coolant Pump Motor Lift Sling	2-3
3	DISCUSSION	3-1
4	CONCLUSION	4-1.
APPENDIX A	CRITICAL ITEMS LIST PER ANSI N14.6-1978	A-1
APPENDIX B -	TABULATION OF ANSI N14.6-1978 REQUIREMENTS INCOMPATIBLE WITH THE TURKEY POINT SPECIAL	B-1

LIST OF ILLUSTRATIONS

Figure	Title	Page
A-1	Reactor Vessel Head Lift Rig	A-5
A-2	Reactor Vessel Internals Lift Rig, Load Cell and Linkage	A-10
A-3	Reactor Coolant Pump Motor Lift Sling	A-12

LIST OF TABLES

Table	Title	Page
2-1	Comparison of the Requirements of ANSI N14.6 and Turkey Point Special Lifting Devices	2-4
A-1	Reactor Vessel Head Lift Rig, Critical Items List of Parts per ANSI N14.6-1978	A-3
A-2	Reactor Vessel Head Lift Rig, Critical Items List of Welds per ANSI N14.6-1978	A-4
A-3	Reactor Vessel Internals Lift Rig, Load Cell and Load Cell Linkage, Critical Items List of Parts per ANSI N14.6-1978	A-6
A-4	Reactor Vessel Internals Lift Rig, Load Cell and Load Cell Linkage, Criticals Items List of Welds per ANSI N14.6-1978	A-9
A-5	Reactor Coolant Pump Motor Lift Rig, Critical Items List of Parts and Welds per ANSI N14.6-1978	A-11
B-1	Bill of Material for the Non-Designed Items of the Reactor Coolant Pump Motor Lift Sling	B-7

REFERENCES

- 1. Westinghouse Drawing 685J249, Head Lifting Rig General Assembly
- Westinghouse Drawing 685J291, Turkey Point Units 3 and 4 Internals Lifting Rig General Assembly
- Westinghouse Drawing AED-SK-618J644TXK Handling Sling Spreader for Motor SV-4M-Al Pump General Assembly

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SECTION 1 PURPOSE

The purpose of this report is to compare the requirements of the special lifting rigs used to lift the reactor vessel head, reactor vessel upper and lower internals, and the reactor coolant pump motor with the requirements contained in ANSI N14.6 for special lifting devices.

SECTION 1 INTRODUCTION

The Nuclear Regulatory Commission (NRC) issued NUREG 0612 "Control of Heavy Load at Nuclear Power Plants"[1] in 1980 to address the control of heavy loads to prevent and mitigate the consequences of postulated accidental load drops. NUREG 0612 imposes various training, design, inspection and procedural requirements for assuring safe and reliable operation for the handling of heavy loads. In the containment building. NUREG 0612 Section 5.1.1(4) requires special lifting devices to meet the requirements of ANSI N14.6-1978-"American National Standard for Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Materials" [2]. In general, ANSI N14.6 contains detailed requirements for the design, fabrication, testing, maintenance, and quality assurance of special lifting devices. In addition, ANSI N14.6 required that when wire rope or chain is used in the design of a special lifting device, the wire rope or chain shall be in conformance with ANSI B30.9-1971 "American National Standard Safety Standard for Slings. "[3] The Turkey Point lifting devices which can be categorized as special lifting devices and which are contained in the scope of this report are:

- 1. Reactor vessel head lift rig
- 2. Reactor vessel internals lift rig
- 3. Load cell
- 4. Load cell linkage
- Reactor coolant pump motor lift sling.

This report contains the evaluation performed on these lifting devices to determine the acceptability of these devices to meet the above requirements.

1.1 BACKGROUND

The reactor vessel head lift rig, the reactor vessel internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling were designed and built for the Turkey Point Units 3 and 4 circa 1967-68. The actual design criteria is unknown for the lifting devices. It appears that for the head lift rig, internals lift rig, load cell, load cell linkage and the reactor coolant pump motor lift sling that in most cases the design criteria used was that the resulting stress in the load carrying members, when subjected to the total combined lifting weight, does not exceed one fifth (1/5) of the ultimate strength of the material. These items were not classified as nuclear safety components and requirements for formal documentation of design requirements and stress reports were not applicable. Thus, stress reports and design specifications were not formally documented. Westinghouse defined the design, fabrication and quality assurance requirements on detailed manufacturing drawings and purchase order documents. Westinghouse also issued field assembly and operating instructions, where applicable.

SECTION 2 COMPONENT DESCRIPTION

2.1 REACTOR VESSEL HEAD LIFT RIG

The reactor vessel head lift rig^[4] (Figure 2-1) is a three legged carbon steel structure, approximately 35 feet high and 14 feet in diameter, weighing approximately 25,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and Control Rod Drive Mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operation, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2 REACTOR VESSEL INTERNALS LIFT RIG

The internals lifting rig^[5] (Figure 2-2) is a three-legged carbon and stainless steel structure, approximately 29 feet high and 12 feet in diameter weighing approximately 13,000 pounds. It is used to handle the upper and lower reactor vessel internals packages. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The rig attaches to the internals packages by means of three engaging screws which are screwed into tapped holes in the internals flanges. These screws are manually operated from the spreader assembly using the integral tools. The screws are normally spring retracted upward and are depressed to engage the tapped holes.

2.3 LOAD CELL AND LOAD CELL LINKAGE

The load cell is used to monitor the load during lifting and lowering the reactor vessel internals to ensure no excessive loadings are occurring. It is installed between the load cell linkage and the lifting device. The load cell is a strain gage (tension) type, rated at 300,000 pounds and built by W. C. Dillon and Co. The load cell linkage is an assembly of pins, plates and bolts which connect the polar crane main hook to the load cell.

2.4 REACTOR COOLANT PUMP MOTOR LIFT SLING

The reactor coolant pump motor lift sling ^[6] (Figure 2-3) consists of a six (6) foot triangular carbon steel spreader assembly weighing approximately one thousand (1,000) pounds. It has wire rope slings, shackles and turnbuckles attached which form a three point lift assembly. It is used to handle the reactor coolant pump motor.

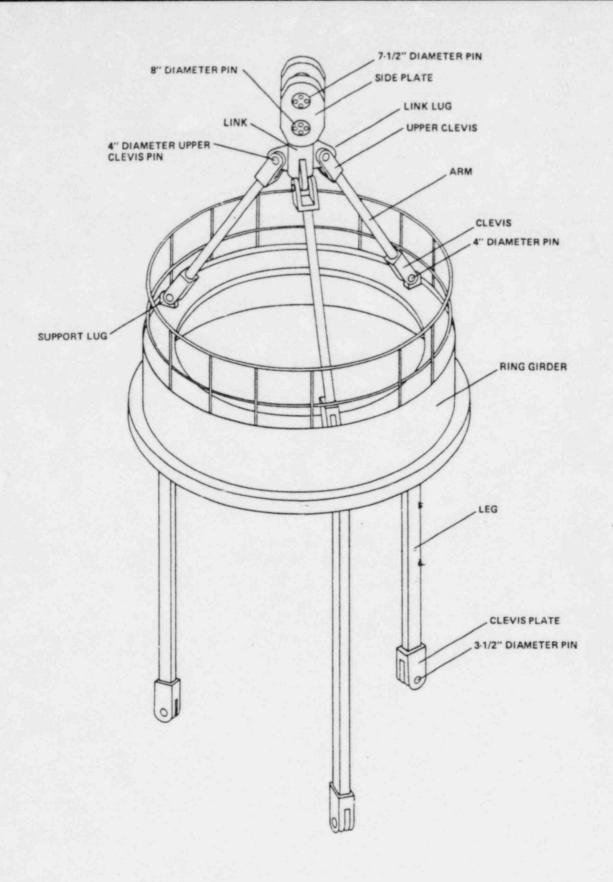


Figure 2-1. Reactor Vessel Head Lifting Rig

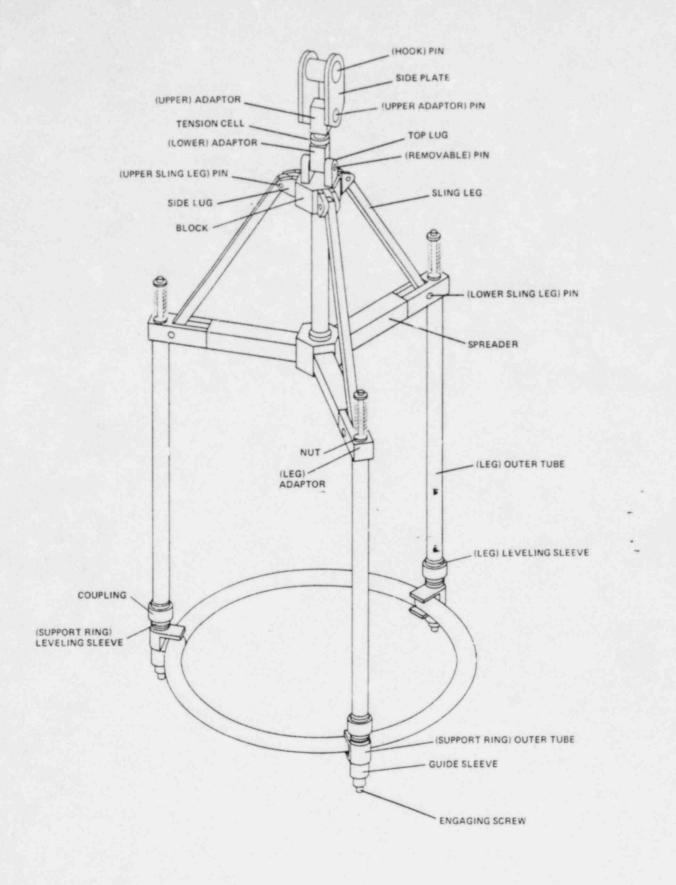


Figure 2-2. Reactor Vessel Internals Lifting Rig

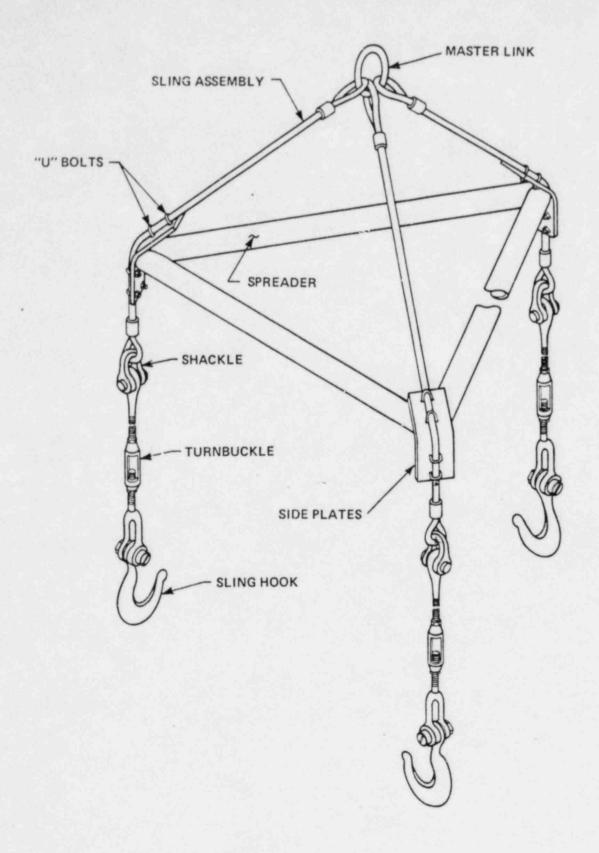


Figure 2-3. Reactor Coolant Pump Motor Lift Sling

SECTION 3 SCOPE OF EVALUATION

The evaluation of these lifting devices consists mainly of three parts:

- 1. A detailed review of the ANSI N14.6 requirements
- 2. Preparation of a stress report
- Recommendations to demonstrate compliance with NUREG 0612, Section 5.1.1(4).

Discussion of these items follows.

3.1 REVIEW OF ANSI N14.6-1978

A detailed comparison was made of the information contained in ANSI N14.6 with the information that was used to design, manufacture, inspect and test these special lifting devices.* The detailed comparison is provided in three parts:

- 1. Overall item by item comparison of requirements
- Preparation of a critical item list per ANSI N14.6 Section 3.1.2, and
- 3. Preparation of a list of nonconforming items.

This detailed analysis is contained in Attachment A to this report.

3.2 PREPARATION OF A STRESS REPORT

Section 3.1.3 of ANSI N14.6 and NUREG 0612 Section 5.1.1(4) require a stress report to be prepared. Special loads and allowable stress criteria are specified for this analysis. The stress report is Attachment B to this report.

3.3 RECOMMENDED ACTIONS

An obvious result from the previous evaluations is a list of items that can be performed to demonstrate to the NRC that these special lifting devices are in compliance with the guidelines of ANSI N14.6 and NUREG 0612 Section 5.1.1(4). These recommendations are identified in Section 6.

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SECTION 4 DISCUSSION OF EVALUATIONS

4.1 STUDY OF ANSI N14.6-1978

A review of ANSI N14.6 identifies certain analyses to be performed and certain identifications that are required to be made to demonstrate compliance with this document. These are a preparation of a stress report in accordance with Section 3.2 and a preparation of a critical items list in accordance with Section 3.1.2. The stress report is Attachment B to this report. The critical items list has been prepared per Section 3.1.2 and is contained in Appendix A to Attachment A. This list identifies the critical load path parts and welds, the materials of these items, and the applied non-destructive volumetric and surface inspections that were performed. (Details of these non-destructive processes and acceptance standards are available at Westinghouse should they be needed.)

A detailed item by item comparison of all the requirements of ANSI N14.6 and those used for the design, manufacture and inspection of these lifting devices is contained as Table 2-1 of Attachment A. The comparison shows that these devices meet the intent of the ANSI document for design, fabrication and quality control. However, they do not meet the requirements of ANSI N14.6 for periodic maintenance, proof and functional testing. Thus, a tabulation of those ANSI N14.6 requirements that are incompatible with these lifting devices was prepared and is Appendix B to Attachment A. Included in Appendix B to Attachment A are recommended actions that may be used to demonstrate acceptability to the NRC.

4.2 STRESS REPORT

As part of the invoking of the ANSI N14.6 document, the NRC requested utilities to demonstrate their compliance with the stress criteria with some qualifying conditions. Attachment B is the stress report for these devices performed in accordance with the criteria of ANSI N14.6. A

discussion is included which responds to the NRC qualifying conditions of NUREG 0612. All of the tensile and shear stresses with the exception of the tensile stress at the minimum section of the internals engaging screw, item 23, meet the design criteria of Section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with the accompanying allowable stress limits of yield and ultimate strength, respectively. In the engaging screw, the tensile stress slightly exceeds the criteria that three times the calculated stress must be less than the yield stress. However, the conservative criterion that five times the calculated stress must be less than the ultimate is met.

Application of Section 3.2.1.1 of ANSI N14.6 criteria to pins subject to bending, structural members subject to buckling and bending loads, and various parts subject to bearing loads, result in some stresses exceeding this criteria. However, when using more appropriate criteria, the resulting stresses are acceptable.

4.3 RECOMMENDATIONS

The recommendations identified in Section 6 require a review of the existing Turkey Point maintenance and operating instructions to ensure that they contain information relative to the identification, maintenance and periodic testing required by ANSI N14.6. The extent of the periodic testing is also addressed and the recommendations identify procedures which are intended to fully meet the intent of NUREG U612 and ANSI N14.6 with the least amount of perturbation to the refueling sequence.

SECTION 5 CONCLUSIONS

The following conclusions are apparent as a result of this evaluation:

- The ANSI N14.6 requirements for design, fabrication and quality assurance are generally in agreement with those used for these special lift devices.
- The ANSI N14.6 criteria for stress limits associated with certain stress design factors for tensile and shear stresses are adequately satisfied with the exception noted in Section 4.2.
- 3. These devices are not in strict compliance only with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance. Recommendations are included to identify actions that should enable these devices to be considered in compliance with the intent of ANSI N14.6.
- 4. The application of the ANSI N14.6 criteria for stress design factor of 3 and 5 are only for shear and tensile loading conditions. Other loading conditions are to be analyzed to other appropriate criteria.

SECTION 6 RECOMMENDATIONS

The following recommendations address the areas of ANSI N14.6 which are incompatible with the present lifting devices and which are considered most important in demonstrating the continued reliability of these devices. They consist of suggestions and proposed responses to identify compliance to the NRC and future considerations.

- 6.1 Review the existing operating procedures to include consideration of ANSI N14.6 Sections 5.1.3 through 5.1.8. These sections include requirements for: scheduled periodic testing; special identification and marking; maintenance, repair, testing and use. Westinghouse remarks on addressing these sections are listed in Attachment A, Appendix B, Items 7, 8, and 9.
- 6.2 A proposed response to the requirement of ANSI N14.6 Section 5.2.1 requiring an initial acceptance load test prior to use equal to 150 percent of the maximum load is as follows:
 - The head lift rig was load tested and inspected at assembly to approximately 100 percent of the load. The internals lift rig and the R.C. pump motor lift sling were not required to be load tested.
- 6.3 A proposed response to ANSI N14.6 Section 5.3 which requires, annually, either a 150 percent maximum load test or dimensional, visual and non-destructive testing of major load carrying welds and critical areas follows. (Since the 150 percent load test is very impractical, the approach identified in the following recommendation is to perform a minimum of non-destructive testing.)

a. Reactor Vessel Head Lift Rig:

Prior to use and after reassembly of the head lifting rig sling assembly to the platform assembly, visually check the clevis to leg fillet welds at the bottom end of the legs and the support lug to ring girder fillet welds on the platform. Raise the vessel head slightly above its support and hold for 10 minutes. During this time, continue to visually inspect these welds. If no problems are apparent, continue to lift.

b. Reactor Vessel Internals Lift Rig

Prior to use, visually inspect the rig components and welds while on the storage stand for signs of cracks or deformation. Check all boiled joints to ensure that they are tight and secure. After connection to the upper or lower internals, raise the assembly slightly off its support and hold for 10 minutes. If no problems are apparent, continue to lift, monitoring the load cell readout at all times.

c. Reactor Coolant Pump Motor Lift Sling

Prior to use, visually inspect the rig components and welds for signs of cracks, deformation, kinks, or frayed ends. Check all bolted joints to ensure that they are tight and secure. After connection to the pump motor, raise the assembly slightly off its support and hold for 10 minutes. During this time visually check the spreader welds. If no problems are apparent, continue to lift.

The above actions do not include a non-destructive test of these welds because:

a. Access to the welds for surface examination is difficult. These rigs are in containment and some contamination is present.

- b. All tensile and shear stresses in the welds are well within the allowable stress with the exception of the internals engaging screw.
- c. The items that are welded remain assembled and cannot be misused for any other lift other than their intended function.
- d. To perform non-destructive tests would require:
 - Removal of paint around the area to be examined which is contaminated.
 - (2) Performance of either magnetic particle inspection or liquid penetrant inspection and
 - (3) Repainting after testing is completed.
 - (4) Cleanup of contaminated items.

Performing non-destructive tests on these welds every refueling would increase the critical path refueling time.

Dimensional checking is not included since these structures are large (about 14 ft. dia. by 35 ft. high) and the results of dimensional checking would always be questionable. Other checks on critical load path parts such as pins, are also not included since an examination of these items would require disassembly of the special lift devices.

6.4 Recommend that a periodic non-destructive surface examination of critical welds and/or parts be performed once every ten years as part of an inservice inspection outage.

- 6.5 Recommend that a modification be made to the load cell to be adaptable to both the head and internals lift rig. Thus, monitoring of these loads during movement can be accomplished at all times.
- 6.6 Recommend that no changes be made to the reactor vessel internals lift rig should the stresses, discussed in Attachment B, be considered excessive by others because:
 - a. The design weight used in the stress calculations is based on the weight of the lower internals. The lower internals are only removed when a periodic inservice inspection of the vessel is required (once/10 years).
 - b. Prior to removal of the lower internals, all fuel is removed. Thus the concern for handling over fuel is non-existant in this particular case.
 - c. Normal use of the rig is for moving the upper internals, which weigh less than 1/2 of the lower internals. The design weight is based on lifting the lower internals. Thus all the stresses could be reduced by approximately 50 percent and considered well within the ANSI N14.6 criteria for stress design factors.
- 6.7 Recommended that the internals lift rig sling block be changed to a forged block with welded side lugs to reduce the number of welds, should the NRC require yearly surface inspection of welds.
- 6.8 Recommended that plates be added to the sides of the spreader (item 14) to reinforce it. Plates should be of sufficient size to satisfy the emperical equations for structures loaded in combined compression and bending, as described in the ASME Boiler and Pressure Vessel Code Section III, Appendix XVII.

SECTION 2 INTRODUCTION

ANSI N14.6-1978-"American National Standard for Special Lifting Devices for Shipping Containers Meighing 10,000 Pounds or More for Nuclear Materials" contains detailed requirements for the design, fabrication, testing, maintenance and quality assurance of special lifting devices. NUREG 0612 "Control of Heavy Load at Nuclear Power Plants", paragraph 5.1.1(4), specifies that special lifting devices should satisfy the guidelines of ANSI N14.6-1978. Subsequently the Nuclear Regulatory Commission (NRC) has requested operating plants to demonstrate compliance with NUREG 0612. To demonstrate compliance with this document, a detailed comparison of the original design, fabrication, testing, maintenance and quality assurance requirements with those of ANSI N14.6 is necessary.

Thus, the ANSI N14.6 document has been reviewed in detail and compared to the requirements used to design and manufacture the reactor vessel head lift rig, the reactor vessel internals lift rig, load cell, load cell linkage and the reactor coolant pump motor lift sling. This comparison is listed in Table 2-1.

2.1 BACKGROUND

The reactor vessel head lift rig, the reactor vessel internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling were designed and built for the Turkey Point Units 3 and 4, circa 1967-63. The actual design criteria is unknown for the lifting devices. It appears that for the head lift rig, internals lift rig, load cell, load cell linkage and the reactor coolant pump motor lift sling that in most cases the design criteria used was that the resulting stress in the load carrying members, when subjected to the total combined lifting weight, does not exceed one fifth (1/5) of the ultimate strength of the material. Westinghouse also required non-destructive tests and inspections on most critical load path parts and welds both as

raw material and as finished items. These requirements of design, manufacturing and quality assurance were identified on detailed manufacturing drawing and purchasing documents.

Westinghouse also issued field assembly and operating instructions, where applicable.

2.2 COMPONENT DESCRIPTION

2.2.1 Reactor Vessel Head Lift Rig

The reactor vessel head lift rig^[1] is a three legged carbon steel structure, approximately 35 feet high and 14 feet in diameter, weighing approximately 25,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and control rod drive mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod sling assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operations, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2.2 Reactor Vessel Internals Lift Rig

The reactor vessel internals lift rig^[2] is a three-legged carbon and stainless steel or ture, approximately 29 feet high and 12 feet in diameter we was a roximately 13,000 pounds. It is used to handle the upper and the roximately 13,000 pounds. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The rig attaches to the internals packages by means of three engaging screws which are screwed into tapped holes in the internals flanges. These screws are manually operated from the spreader assembly using the integral tools. The screws are normally spring retracted upward and are depressed to engage the tapped holes.

2.2.3 Load Cell and Load Cell Linkage

The load cell is used to monitor the load during lifting and lowering the reactor vessel internals to ensure no excessive loadings are occurring. It is installed between the load cell linkage and the lifting device. The load cell is a strain gage (tension) type, rated at 300,000 pounds, built by W. C. Dillon and Co. The load cell linkage is an assembly of pins, plates and bolts which connect the polar crane main hook to the load cell.

2.2.4 Reactor Coolant Pump Motor Lift Sling

The reactor coolant pump motor lift sling^[3] consists of a six (6) — foot triangular carbon steel spreader assembly weighing approximately one thousand (1,000) pounds. It wire rope slings, shackles and turnbuckles attached which form a three point lift assembly. It is used to handle the reactor coolant pump motor.

TABLE 2-1

COMPARISON OF THE REQUIREMENTS OF ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
1 1.1 to 1.3 2	Scope and Definitions - These sections define the scope of the document and include pertinent definitions of specific items	These sections are definitive, and not requirements.
3 3.1 3.1.1 to 3.1.4	Designer's Responsibilities - This section contains requirements for preparing a design specification and its' contents, stress reports; repair procedures; limitations on use with respect to environmental conditions; marking and nameplate information; and critical items list.	A. No design specification was written concerning these specific requirements. However, assembly and detailed manufacturing drawings and purchasing documents contain the following requirements: (1) Material specification for all of the critical load path items to ASTM, ASME specifications or special requirements listed. (2) All welding, weld procedures and welds to be in accordance with ASME Boiler and Pressure Vessel Code - Section IX or Westinghouse specifications (3) Special non-destructive testing for specific critical load path items to be performed to written and approved procedures in accordance with ASTM or specified requirements

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
		(4) All coatings to be performed to strict compliance with specified requirements.
		(5) Letters of compliance for materials and specifications were required for verification with original specifi- cations.
		B. A stress report was not originally required but has been prepared and is Attachment B.
		C. Repair procedures were not identified.
	• • •	D. No limitations were identified as to the use of these devices under adverse environments.
		E. Markings and nameplate information was not addressed except for the reactor coolant pump motor lifting which identifies MAXCAPACITY & TOOL (Drawing) Number.
		F. Critical item lists have been prepared for each device and are identified as Appendix A to this Attachment A.

TABLE 2-1 (cont)

CUMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.2 3.2.1 to 3.2.6	Design Criteria Stress Design Factors - These sections contain requirements for the use of stress design factors of 3 and 5 for allowable stresses of yield and ultimate respec- tively for maximum shear and tensile stresses; high strength material stress design factors; special pins; wire rope and slings to meet ANSI B30.9-1971; and drop-weight tests and Charpy impact test requirements	1. The actual design criteria is unknown for the lifting devices. It appears that for the head lifting rig, internals lift rig, load cell, and the reactor coolant pump motor lift sling that in most cases the design criteria used was that the resulting stress in the load carrying members, when subjected to the total combined lifting weight, does not exceed one fifth of the ultimate strength of the material. A stress report (Attachment B) has been generated which addresses the capability of these rigs to meet the ANSI design stress factors. 2. High strength materials are used in some of these devices (mostly for pins). Although the fracture toughness was not determined, the material was selected based on it's fracture toughness characteristics. However, the stress design factors of ANSI NI4.6 Section 3.2.1 of 3 and 5 were used in the analysis and the resulting stresses are acceptable. 3. Where necessary, the weight of pins was considered for handling.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.3 3.3.1 to 3.3.8	Design Considerations - These sections contain considerations for; materials of construction, lamellar tearing; decontamination effects; remote engagement provisions; equal load distribution; lock devices; position indication of remote actuators; retrieval of device if disengaged; and nameplates.	4. Wire rope is used only in the design of the R.C. pump motor lift sling. However, the wire rope is of a special design and thus this device is considered a special lift device. 5. Drop weight and Charpy impact tests were not required nor performed. Decontamination was not specifically addressed. Lamellar tearing was not considered but the designs of these devices are not susceptible to this type of joint deterioration. Even distribution of the load is evident from these designs. Locking plates, pins, etc. are used throughout these special lifting devices. Remote actuation is only used when engaging the internals lift rig with the internals, however, no position indication of engagement was provided. However, all these items were considered and the designs reflect these requirements.
3.4 3.4.1 to 3.4.6	Design Considerations to Minimize Decontam- ination Efforts in Special Lifting Device Use - These sections contain fabrication, welding, finishes, joint and machining requirements to permit ease in decontamination.	Decontamination was not specifically addressed. However, the design and manufacture included many of these items, i.e. lock devices, pins, etc.

TABLE 2-1 (cont)
COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.5 3.5.1 to 3.5.10	Coatings - These sections contain provisions for ensuring proper methods are used in coating carbon steel surfaces and for ensuring non-contamination of stainless steel items.	The requirements for coating carbon steel surfaces are contained in a Westinghouse process specification referenced on the assembly and detail drawings except for the reactor coolant pump motor lift sling. (The coating requirements for the reactor coolant pump motor lift sling are contained in purchasing documents and require proper preparation and application of an epoxy paint.) This specification requires a proven procedure, proper cleaning, preparation, application and final inspection of the coating. These requirements meet the intent of 3.5.1 through 3.5.8. No provisions were included in these designs for consideration of decontamination materials or the use of non-contaminating contact materials for use in stainless steel parts.
3.6 3.6.1 to 3.6.3	Lubricants - These sections contain requirements for special lubricants to minimize contamination and degradation of the lubricant and contacted surfaces or water pools	No specific lubrication requirements have been identified. However, neolube is recommended for use with the engaging screws in the internals lift device which are under water and silicone grease for the load cell pins which are out of water.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
4 4.1 4.1.1 to 4.1.12	Fabrication Fabricators Responsibilities -These sections contain specific requirements for proper quality assurance, document control, deviation control, procedure control, material identification and certificate of compliance.	A formal quality assurance program for the manufacturer was not required. However, all the manufacturers welding procedures and non-destructive testing procedures were reviewed by Westinghouse prior to use. All critical load carrying members require letters of compliance for material requirements. Westinghouse performed certain checks and inspections during various steps of manufacturing. Final Westinghouse review includes visual, dimensional, procedural, cleanliness, personnel qualification, etc. and issuance of a quality release* to ensure conformance with drawing requirements.
4.2 4.2.1 to 4.2.5	Inspectors Responsibilities -These sections contain requirements for a non-supplier inspector.	Westinghouse Quality Assurance personnel performed some inprocess and final inspections similar to those identified in these sections. (Also see comments to Section 4.1 above)
4.3 4.3.1 to 4.3.3	Fabrication Considerations -These sections contain special requirements for ease in decontamination or control of corrosion.	General good manufacturing processes were followed in the manufacture of these devices. However, the information defined in these sections was not specifically addressed.

^{*} These remarks are not applicable to the Reactor Coolant Pump Motor Lift Sling

TABLE 2-1 (cont)

COMPARISON OF REQUIREMENT OF THE ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
5.1 5.1.1 to 5.1.8	Acceptance Testing Maintenance, and Assurance of Continued Compliance Owner's Responsibilities - Sections 5.1.1 and 5.1.2 require the owner to verify that the special lifting devices meet the performance criteria of the design specification by reviewing records and witness of testing.	There wasn't any design specification for these rigs and functional load testing was not originally required, or performed. *However, the Westinghouse Quality Release may be considered an acceptable alternate to verify that the criteria for letters of compliance for materials and specifications required by the Westinghouse drawings and purchasing documents was satisfied. *Although proof and functional testing was not required, the site assembly instructions require, after initial assembly on site, the following for the reactor vessel head lifting: Raising the rig, assembled to it's respective attachment, slightly above the supporting surface to be free hanging for one-half hour. Lowering the rig to its support and performing visual inspection and the appropriate nonde-structive testing.

^{*} These remarks are not applicable to the Reactor Coolant Pump Motor Lift Sling

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
	Section 5.1.3 requires periodic functional testing	Since maintenance and inspection procedures are written, if these procedures do not contain this requirement, then these procedures should be revised to include a visual check of critical welds and parts during lifting to comply with this requirement for functional testing.
	Section 5.1.4 requires operating procedure	Operating instructions for the reactor vessel internals lift rig were furnished to the utility and operating procedures were prepared and are used.
	Sections 5.1.5, 5.1.5.1 and 5.1.5.2 require special identification and marking to prevent misuse.	It is obvious from their designs that these rigs are special lifting devices and can only be used for their intended purpose. Specific identification of the rig can be made by marking, with stencils, the rig name and rated capacity, preferably on the spreader assembly.
	Sections 5.1.6, 5.1.7 and 5.1.8 require the owner to provide written documentation on the maintenance, repair, testing and use of these rigs.	Since operating instructions and maintenance instructions have been written by the owner, these should be reviewed to assure that they contain the requirements to address maintenance logs, repair and testing history, damage incidents etc.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
5.2 and 5.3 5.2.1 to 5.2.3 and 5.3.1 to 5.3.8	Acceptance lesting and Testing to Verify Continuing Compliance - These paragraphs require the rigs to be initially tested at 150 percent maximum load followed by non-destructive testing of critical load bearing parts and welds and also annual 150 percent load tests or annual non- destructive tests and examinations; qualification of replacement parts.	The head lift rig was load tested at field assembly. It is suggested that a check of critical welds and parts be included in the maintenance procedures for all three devices. Preferably, since these devices have been in use at least once per year for over ten years, that a visual check during initial lift when possible should be acceptable. Further note that with the use of the load cell for the internals, lifting and lowering is monitored at all times. However, the load cell, which is used to monitor the internal lift rig loads at all times, cannot exceed the rated load by 20 percent without being inaccurate. This would preclude monitoring of a load test with the present equipment. Replacement parts should be in accordance with the original or equivalent requirements.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND

TURKEY POINT SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
5.4 5.4.1 to 5.4.2	Maintenance and Repair - This section requires any maintenance and repair to be performed in accordance with original requirements and no repairs are permitted for bolts, study and nuts.	Maintenance and repair procedure should contain, as much as possible, requirements that were used in the original fabrication. The critical items list of Appendix A contains the original type of non-destructive testing. The procedure should also define bolts, studs and nuts as non-repairable items.
5.5 5.5.1 to 5.5.2	Non-destructive Testing Procedures, Personnel Qualifications, and Acceptance Criteria - This section requires non- destructive testing to be performed in accordance with the requirements of the ASME Boiler and Pressure Vessel Code	Liquid penetrant, magnetic particle, ultrasonic and radiograph inspections were performed on identified items. These were in accordance with ASTM specifications, the ASME Code, and Westinghouse process specifications or as noted on detailed drawings and provide similar results to the requirement of the ASME Code.
6 6.1 6.2 6.3	Special Lifting Devices for Critical Loads - These sections contain special requirements for items handling critical loads.	It is assumed that compliance with NUREG 0612, Section 5.1 has been demonstrated and therefore this section is not applicable to these devices.

SECTION 3 DISCUSSION

The reactor vessel head and internals lift rigs, load cell, load cell linkage and reactor coolant pump motor lift sling generally meet the intent of the ANSI N14.6 requirements for design and manufacture. However, they are not in strict compliance with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance.

Although no specific design specification was written, the assembly and detailed manufacturing drawings and purchase order documents contain equivalent requirements. A stress report has been prepared for these devices and the design criteria is considered satisfied. These devices, for the most part, were manufactured under Westinghouse surveillance with identified hold points, procedure review and personnel qualification which adequately meet these related ANSI requirements. Acceptance testing was not performed. However, an initial lift test was conducted for the head lift rig followed by the appropriate non-destructive testing following site assembly. Although the reactor coolant pump motor lift sling is called a sling, it is not a standard catalog item. Thus, it cannot be considered to be reviewed in accordance with the requirements of ANSI B30.9-1971.

It is anticipated that a 100 percent load test, performed on each device, followed by a visual check of critical welds would be sufficient to demonstrate compliance. This may require modification of existing Turkey Point operating and maintenance procedures.

SECTION 4 CONCLUSIONS

The review of the ANSI N14.6 requirements and comparison with the original Westinghouse requirements has shown that these items are generally in agreement for the design, fabrication and quality assurance of the lifting devices. However, the lifting devices are not in strict compliance with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance. These specific requirements that are incompatible with the lifting devices are discussed in Appendix B with suggested actions. Westinghouse's objective was to provide a quality product and this product was designed, fabricated, assembled and inspected in accordance with internal Westinghouse requirements. In general, Westinghouse requirements meet the intent of ANSI N14.6 but not all the specific detailed requirements.

APPENDIX A CRITICAL ITEMS LIST PER ANSI N14.6-1978

1. GENERAL

Section 3.1.2 of ANSI N14.6-1978 specifies that the design specification shall include a critical items list, which identifies critical components and defines their critical characteristics for material, fabrication, non-destructive testing and quality assurance.

"Critical items list" is further defined in ANSI N14.6, Section 2 as:

"critical items list. A list that specifies the items of a special lifting device and their essential characteristics for which specified quality requirements shall apply in the design, fabrication, utilization, and maintenance of the device."

Load carrying members and welds of these special lifting devices are considered to be the critical items.

Tables A-1, A-2, A-3, A-4, and A-5 are the critical items list of parts and welds for the reactor vessel head lift rig, the reactor vessel internals lift rig, load cell and load cell linkage, and the reactor coolant pump motor lift sling respectively. These tables include the material identification, and the applicable volumetric and surface inspections that were performed in the fabrication of these special lifting devices. In some instances, non-destructive testing was not specified since the material selection and strength result in very low tensile stresses and thus, non-destructive testing was not justified.

The material selection for critical load path items was made to ASTM, ASME or special material requirements. However, the non-designed items of the reactor coolant pump motor lift sling were selected based on

their load carrying capacities. The material requirements were supplemented by Westinghouse imposed non-destructive testing, and/or special heat treating requirements for almost all of the critical items. Westinghouse required all welding, welders, and weld procedures to be in accordance with ASME Boiler and Pressure Vessel Code Section IX or Westinghouse specifications. Westinghouse required certificates, or letters of compliance that the materials and processes used by the manufacturer were in accordance with the purchase order and drawing requirements. Westinghouse also performed final inspection on these devices and issued quality releases for the internals and head lifting rigs.

TABLE A-1
REACTOR VESSEL HEAD LIFT RIG
CRITICAL ITEMS LIST OF PARTS
PER ANSI N14.6-1978

Item(a)	Description	Material	Non-destruct	tive Testing
			Material	Finished
1,7,10,13	Pins	ASTM A434 Class BD	Ultrasonic	Magnetic Particle
2	Clevis Plate	ASTM A515 Grade 70	Ultrasonic Magnetic Particle	
3	Leg	ASTM A36		
4	Ring Girder	ASTM A285 Grade C		
5	Support Lug	ASTM A515 Grade 70	Ultrasonic Magnetic Particle	
6,9	Clevis	ASTM A237 Class A	Ultrasonic	Magnetic Particle
8	Arm	ASTM A306 Grade 70	Ultrasonic	Magnetic Particle
11,12	Sling Assembly Link and Lug	ASTM A105 Class 2	Ultrasonic	Magnetic Particle
15	Lifting Plate	ASTM A514	Ultrasonic	Magnetic Particle

⁽a) See figure A-1

TABLE A-2
REACTOR VESSEL HEAD LIFT RIG
CRITICAL ITEMS LIST OF WELDS
PER ANSI N14.6-1978

Item(a)	Weld	Non-destructive Testing		
	Description	Root Pass	Final	
2,3	Clevis Plate to Leg (fillet)	Visual	Magnetic Particle	
4,5	Ring Girder to Support Lug (fillet)		Magnetic Particle	
11,12	Link Lugs to Link (full penetration)		Radiograph Magnetic Particle	

⁽a) See figure A-1.

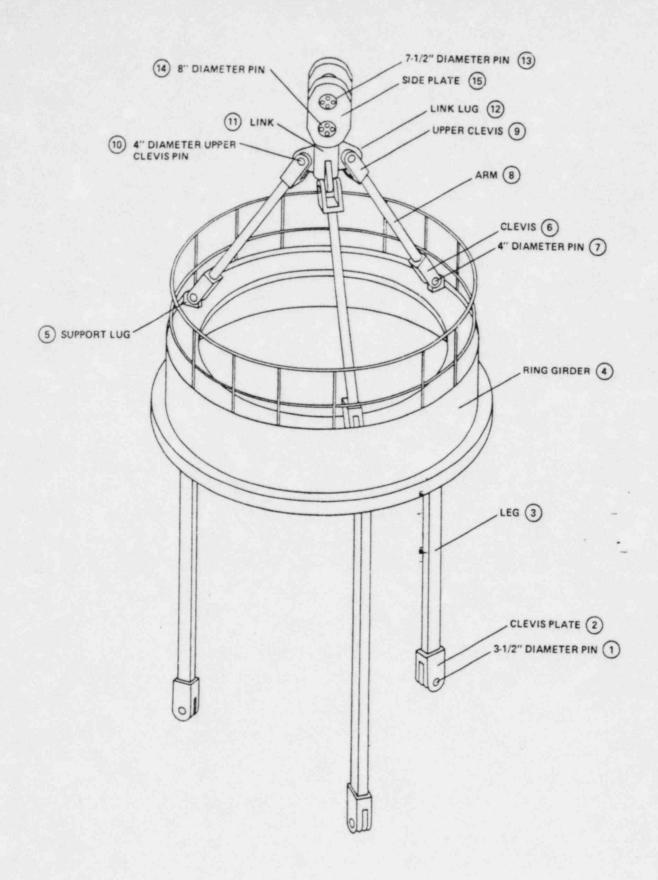


Figure A-1. Reactor Vessel Head Lift Rig

A-5

TABLE A-3 REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

Item(a)	Description	Material	Non-destruct	tive Testing
			Material	Finished
1,	Hook Pin	ASTM A434 Class BD	Ultrasonic	Magnetic Particle
3	Upper Adapter Pin			
7	Removable Pin			
2	Side Plate	ASTM A515 Grade 70 or ASTM A516 Grade 70	Magnetic Particle	
8	Top Lugs		•	
12	Sling Leg			
10	Side Lugs		•	
4	Upper Adapter	ASTM A540 Grade B24	Ultrasonic	Magnetic Particle or Liquid Penetrant
6	Lower Adapter			
5	Tension Cell	17-4 pH H-1100	Ultrasonic	Liquid Penetrant
9	Block	SA-105 Class 1 or 2 or SA-266 Class 1 or 2 or SA-508 Class 1 or 2		Magnetic Particle

⁽a) See figure A-2

TABLE A-3 (Cont) REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

Item(a)	Description	Material	Non-destructive Testing	
			Material	Finished
11	Upper Sling Leg Pin	ASTM A276 Type 304, Center Ground Condition A		
13	Lower Sling Leg Pin			
14A	Spacer Block	SA-105 Class 1 or 2 or SA-266 Class 1 or 2 or SA-508 Class 1 or 2	Ultrasonic	Magnetic Particle
14B,C,D	Spreader Assembly	ASTM A515 Grade 70 or ASTM A516 Grade 70		
15	Nut	ASTM A276 Type 304 HR and PKLD, Condition A		
18	Leg Leveling Sleeve	ASTM A276 Type 304 HR and PKLD, Condition A		
20	Supporting Ring Leveling Sleeve	ASTM A276 Type 304 HR and PKLD, Condition A		
22	Guide Sleeve	ASTM A276 Type 304 HR and PKLD, Condition A		

⁽a) See figure A-2

TABLE A-3 (Cont) REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

Item(a)	Description	Material	Non-destru	ctive Testing
			Material	Finished
16	Leg Adapter	ASTM A276 Type 304 HR and PKLD, Condition A		
23	Engaging Screw	ASTM A276 Type 304 HR and PKLD, Condition A		
19	Coupling	ASTM A312 Type 304		
21	Support Ring Outer Tube	ASTM A312 Type 304 Seamless, Cold Finish, HT TR		
17	Leg Outer Tube	ASTM A312 Type 304 Seamless, Cold Finish, HT TR		

⁽a) See figure A-2

TABLE A-4 REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE CRITICAL ITEMS LIST OF WELDS PER ANSI N14.6-1978

Item(a)	Weld	Non-destructive Testing		
	Description	Root Pass	Final	
8,9	Top Lugs to Sling Block (full penetration)	Magnetic Particle	Magnetic Particle	
9,10	Side Lugs to Sling Block (full penetraton)	Magnetic Particle	Magnetic Particle	
14	Spreader Assembly (fillet)	Visual	Magnetic Particle	
16,17	Leg Outer Tube to Leg Adapter (full penetration)	Liquid Penetrant	Liquid Penetrant	
17,18	Leg Outer Tube to Leg Leveling Sleeve (full penetration)	Liquid Penetrant	Liquid Penetrant	
21,22	Support Ring Outer Tube to Guide Sleeve (full penetration)	Liquid Penetrant	Liquid Penetrant	

⁽a) See figure A-2.

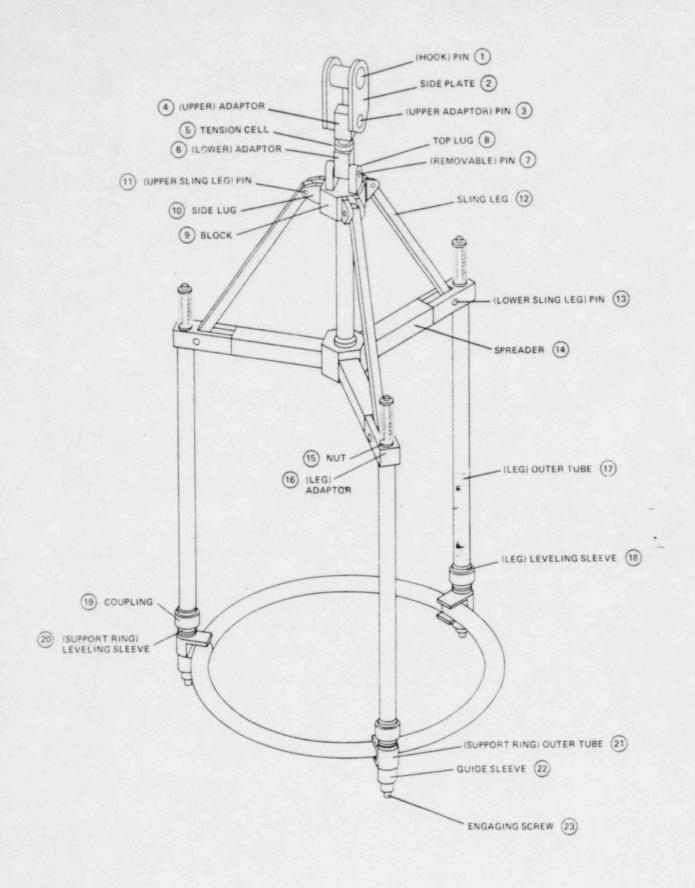


Figure A-2. Reactor Vessel Internals Lift Rig

TABLE A-5

REACTOR COOLANT PUMP MOTOR LIFT RIG

CRITICAL ITEMS LIST OF PARTS AND WELDS

PER ANSI N14.6-1978

Item(a)	Description	Material	Non-destructive Testing	
			Material	Finished
1	Spreader	ASTM A106 Grade B		Magnetic Particle on Welds Unly
2	Side Plate	ASTM A106 Grade B		
5	Master Link	Alloy Steel Forging	Radiograph	
6	Sling Assembly	Improved Plow Steel Grade		
7	Shackle	Alloy Steel Forging		
8	Turnbuckle	Alloy Steel Forging		
9	Eye Hook	Alloy Steel Forging		

⁽a) See figure A-3

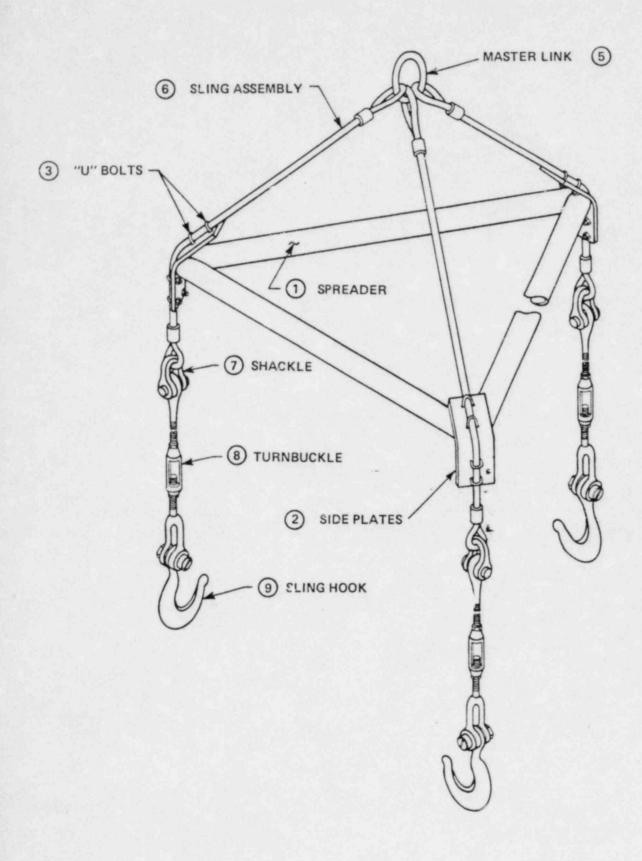


Figure A-3. Reactor Coolant Pump Motor Lift Sling

APPENDIX B

TABULATION OF ANSI N14.6-1978 REQUIREMENTS INCOMPATIBLE WITH THE TURKEY POINT LIFTING DEVICES

1. GENERAL

The comparison of the various ANSI N14.6 requirements and those of these lifting devices has shown that these devices are not in strict compliance with all the ANSI N14.6 requirements. Listed below is a tabulation of those sections of ANSI N14.6 considered most important in demonstrating the continued load handling reliability of these special lifting devices. Associated Westinghouse remarks are also listed and could be used as suggested actions and/or responses to demonstrate compliance to the NRC.

la. Requirement:

Para. 3.1.4 - requires the designer to indicate permissible repair procedures and acceptance criteria for the repair.

1b. Remarks:

Any repair to these special lifting devices is considered to be in the form of welding. Should pins, bolts or other fasteners need repair, they should be replaced, in lieu of repair, in accordance with the original or equivalent requirements for material and non-destructive testing. Weld repairs should be performed in accordance with the requirements identified in NF-4000 and NF-5000 (Fabrication and Examaination) of the ASME Boiler and Pressure Vessel Code, Section III, Division 1 Subsection NF.

2a. Requirement:

Para. 3.2.1.1 - requires the design, when using materials with yield strengths above 80 percent of their ultimate strengths,

to be based on the material's fracture toughness and not the listed design factors.

2b. Remarks:

High strength materials are used in these devices. Although the fracture toughness was not determined, the material was selected based on it's fracture toughness characteristics. However, in lieu of a different stress design factor, the stress design factors listed in 3.2.1 of 3 and 5 were used in the analysis and the resulting stresses are considered acceptable.

3a. Requirement:

Para. 3.2.6 requires material for load-bearing members to be subjected to drop-weight or Charpy impact tests.

3b. Remarks:

Fracture toughness requirements were not identified for the material used in these special lifting devices. However, the material selection was based on its fracture toughness characteristics.

4a. Requirement:

Para. 3.3.6 requires an indication that an actuating mechanism is engaged.

4b. Remarks:

The reactor vessel internals lift rig employs a long handled tool to engage the rig and the internals. The tool depresses a spring loaded tube and turns the engaging screw into the internals. Although no specific position indication is identified, the visual difference in the top of the spring loaded tube is considered sufficient indication that the internals are engaged.

5a. Requirement:

Para. 4.1.6 requires a formal quality assurance program for the manufacturer and para 4.1.7 requires certification and identification of materials.

5b. Remarks:

A formal quality assurance program for the manufacturer was not required. However, the manufacturers welding procedures and non-destructive testing procedures were reviewed by Westinghouse prior to use. All critical load carrying members require letters of compliance for material requirements. Westinghouse performed certain checks and inspections during various steps of manufacturing. Final Westinghouse review includes visual, dimensional, procedural, cleanliness, personnel qualification, etc. and issuance of a quality release to ensure conformance with drawing requirements. No information that a quality release was issued for the reactor coolant pump motor lift sling has been found, although Westinghouse performed the final inspection.

6a. Requirement:

Para. 5.1 lists <u>Owner Responsibilities</u> and 5.1.2 requires the owner to verify that the special lifting devices meet the performance criteria of the design specification by records and witness of testing.

6b. Remarks:

There wasn't any design specification for these rigs and load testing was not originally required or performed. However, the Westinghouse Quality Release may be considered an acceptable alternate to verify that the criteria for letters of compliance for materials and specifications required by the Westinghouse drawings and purchasing document were satisfied. Although

proof and functional testing was not required, the site assembly instructions require, after initial assembly on site, the following for the reactor vessel head lifting:

Raising the rig, assembled to it's respective attachment, slightly above the supporting surface to be free handing for one-half hour. Lowering the rig to its support and performing visual inspection and the appropriate nondestructive testing. No other checks of welds and/or dimensions after assembly was required of the other special lifting devices.

7a. Requirement:

Para. 5.1.3 requires periodic functional testing and a system to indicate continued reliable performance.

7b. Remarks:

Since maintenance and inspection procedures are available if these procedures do not contain this requirement, then these procedures should be revised to include a visual check of critical welds and parts when possible during lifting to comply with this requirement for functional testing.

8a. Requirement:

Para. 5.1.5, 5.1.5.1 and 5.1.5.2 require special identification and marking to prevent misuse.

8b. Remarks:

It is obvious, from their designs, that these rigs are specific lifting devices and can only be used for their intended purpose and parts are not interchangeable. Specific identification of the rig can be made by marking with stencils, the rig name and rated capacity, preferably on the spreader assembly.

9a. Requirement:

Para. 5.1.6, 5.1.7 and 5.1.8 require the owner to provide written documentation on the maintenance, repair, testing and use of these rigs.

9b. Remarks:

Since operating instructions and maintenance instructions have been written by the owner, these should be reviewed to assure that they contain the requirements to address maintenance logs, repair and testing history, damage incidents and other items mentioned in these paragraphs.

10a. Requirement:

Para 5.2.1 requires the rigs to be initially tested at 150 percent maximum load followed by non-destructive testing of critical load bearing parts and welds.

10b. Remarks

The head lift rig was load tested and inspected at assembly to approximately 100 percent of the load. The internals lift rig and the R.C. pump motor lift sling were not required to be load tested.

11a. Requirement:

Para 5.2.2 requires replacement parts to be individually qualified and tested.

11b. Remarks

Replacement parts, should they be required, should be made of identical (or equivalent) material and inspections as originally required. Only pins, bolt and nuts are considered replacement parts for the reactor vessel head and internal lift rigs. Some of the items comprising the reactor coolant pump motor lift sling are catalog items and should they need replacement should be as identified in table B-1.

12a. Requirement:

Para 5.3 requires testing to verify continuing compliance and annual 150 percent load tests or annual non-destructive tests and examinations to be performed.

12b. Remarks

These special lifting devices are used during plant refueling which is approximately once per year. During plant operation these special lifting devices are inaccessable since they are permanently installed and/or remain in the containment. They cannot be removed from the containment unless they are disassembled and no known purposes exist for disassembly. Load testing to 150 percent of the total weight before each use would require special fixtures and is impractical to perform. Crane capacity could also be limiting. It is suggested that a check (visual) of critical welds and parts be conducted at initial lift when possible prior to moving to full lift and movement for all three devices. Preferably, since these devices have been in use at least once per year for over ten years, that a visual check during initial lift would be acceptable. Further note that with the use of the load cell for the internals lift rig, all lifting and lowering is monitored at all times.

2. SUMMARY

The requirements for periodic checking and functional load testing appear to be the ANSI N14.6 requirements that are most difficult to demonstrate compliance. It is almost impractical to perform the 150 percent load test prior to each use. It is suggested that the proposal to the NRC include a 100 percent load test, similar to the original check-out test to be performed with a minimum of nondestructive testing, (visual-only) in the critical parts and welds.

TABLE B-1

BILL OF MATERIALS FOR THE NON-DESIGNED ITEMS OF THE REACTUR COOLANT PUMP MOTOR LIFT SLING

No. (1)	Item	Description		Rated(6)
		Drawing(2)	Catalog(7)	Load Value (Pounds)
3	"U" Bolts	7/8" "U" Bolts + Cable Saddle	(5) "U" Bolts - G-450 1-1/2" Dia. Wire 7/8" Dia. "U" Bolt Wire Rope Clips	
5	Alloy Oblong Link	2-3/4" x 9" x 18"	(3) 2-3/4" x 9" x 16"	211,500
6	Sling Assembly	1-1/2" x 11'4" (Total inc. turn-buckle, shackle, etc.) 6 x 37 improved plow I.W.R.C. 3 Bridle Sling (7'6" sling length only incl. eye)	1-1/2" x 11'4" 6 x 37 Improved Plowsteel I.W.R.C. 3 Bridle Sling with Mechanical Connections	81,000(2)
7	Shackle	Anchor Safety Shackle	(5) 2" Size G-213 or S-213	70,000
8	Turnbuckle	2" x 6" Jaw & Eye Turnbuckle	(5) G-227 Jaw & Eye Turnbuckle 2" Dia. thd., 24" take-up Class 8	37,000
9	Sling Hook	TAYCO A-73	(3) Sling Hook No. A-73 l" Dia. Chain with 2-1/2" Dia. Eye	38,750

NOTES:

- (1) See figure A-3 for identification of item numbers
- (2) Description is from Westinghouse drawing AED SK 618J644 TXK SUB 5

- (3) Taylor Chain Co., Inc., Alloy Steel Chain Assemblies, Attachments, Revised 4/80
- (4) Pennsylvania Sling Co.
- (5) Crosby Group, 950 General Catalog, June 1981.
- (6) Rated load value: The maximum recommended load that should be exerted on the item. The following terms are also used for the term Rated Load: "SWL", "Safe Working Load", "Working Load Limit", and the "Resultant Safe Working Load". All rated load values are for in-line pull with respect to the centerline of the item. Information is from catalogs identified in (3), (4), (5).
- (7) Ordering Information

Sling assembly per Westinghouse Drawing AED SK 618J644 TXK, SUB 5 composed of the items in table B-1 including the following requirements:

- a. The safe working load of this sling assembly is 81,000 lb. and a safety factor of 5:1".
- b. Perform lift test at assembly of 121,500 pounds (61 tons).
- c. The master link is to be magnetic particle inspected after load test to the requirements of ASME Boiler & Pressure Vessel Code. Section V Article 7. Acceptance standards are to be to ASME B&PV Code Section III Subsection NF 5341. Radiograph inspection may be substituted for magnetic particle inspection to ASME B&PV Code Section V, Article 2 with acceptance standard to ASME B&PV Code Subsection NF 5320.
- d. A certification is required for load testing, non-destructive testing and material used in this assembly.

ATTACHMENT B to WCAP-10168

STRESS REPORT

REACTOR VESSEL HEAD LIFT RIG

REACTOR VESSEL INTERNALS LIFT RIG,

LOAD CELL, LOAD CELL LINKAGE

AND REACTOR COOLANT PUMP

MOTOR LIFT SLING

FOR

FLORIDA POWER AND LIGHT COMPANY TURKEY POINT UNITS 3 AND 4

December, 1982

H. H. Sandner, P.E.

Approved:

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ABSTRACT

A stress analysis of the Turkey Point Units 3 and 4 reactor vessel head and internal lift rigs, load cell, load cell linkage and reactor coolant pump motor lift sling was performed to determine the acceptability of these devices to meet the design requirements of ANSI N14.6.

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TABLE OF CONTENTS

Section	Title	Page
	ABSTRACT	111
1	INTRODUCTION	1-1
	1.1 Background	1-1
2	COMPONENT DESCRIPTION	2-1
	2.1 Reactor Vessel Head Lift Rig	2-1
	2.2 Reactor Vessel Internals Lift Rig	2-1
	2.3 Load Cell and Load Cell Linkage	2-2
	2.4 Reactor Coolant Pump Motor Lift Sling	2-2
3	DESIGN BASIS	3-1
	3.1 Design Criteria	3-1
	3.2 Design Weights	3-2
4	MATERIALS	4-1
	4.1 Material Description +	4-1
5	SUMMARY OF RESULTS	5-1
	5.1 Discussion of Results	5-1
	5.2 Conclusion	5-6
APPENDIX A	DETAILED STRESS ANALYSIS - REACTOR	
	VESSEL HEAD LIFT RIG	A-1
APPENDIX B	DETAILED STRESS ANALYSIS - REACTOR	
	VESSEL INTERNALS LIFT RIG, LOAD CELL	
	AND LINKAGE	B-1
APPENDIX C	DETAILED STRESS ANALYSIS - REACTOR	
	COOLANT PUMP MOTOR LIFT SLING	C-1

LIST OF ILLUSTRATIONS

Figure .	Title	Page
5-1	Reactor Vessel Head Lifting Rig	5-11
5-2	Reactor Vessel Internals Lifting Rig	5-20
5-3	Reactor Coolant Pump Motor Lift Sling	5-22

LIST OF TABLES

Table	Title	Page
4-1	Reactor Vessel Head Lift Rig	
	Material Properties	4-2
4-2	Reactor Vessel Internals Lift Rig,	
	Load Cell and Load Cell Linkage	
	Material Properties	4-3
4-3	Reactor Coolant Pump Motor Lift Sling	
	Material Properties	4-5
5-1	Summary of Results - Reactor Vessel	
	Head Lift Rig	5-7
5-2	Summary of Results - Reactor Vessel	
	Internals Lift Rig, Load Cell and	
	Load Cell Linkage	5-12
5-3	Summary of Results - Reactor Coolant	
	Pump Motor Lift Sling	5-21
5-4	Comparison of Design Loads and Rated	
	Load Values of the Non-Designed Items	
	of the R.C. Pump Motor Lift Sling	5-23

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SECTION 1 INTRODUCTION

The Nuclear Regulatory Commission (NRC) issued NUREG 0612 "Control of Heavy Load at Nuclear Power Plants"[1] in 1980 to address the control of heavy loads to prevent and mitigate the consequences of postulated accidental load drops. NUREG 0612 imposes various training, design. inspection and procedural requirements for assuring safe and reliable operation for the handling of heavy loads. In the containment building, NUREG 0612 requires special lifting devices to meet the requirements of ANSI N14.6-1978 "American National Standard for Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Materials". [2] In general, ANSI N14.6 contains detailed requirements for the design, fabrication, testing, maintenance and quality assurance of special lifting devices. In addition, ANSI N14.6 requires that when wire rope or chain is used in the design of a lifting device, the wire rope or chain shall be in conformance with ANSI B30.9-1971 "American National Standard Safety Standard for Slings".[3] The NRC has requested operating plants to demonstrate compliance with these requirements.

This report contains the stress analysis performed on the Turkey Point reactor vessel headlift rig, reactor vessel internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling to determine the acceptability of these devices to meet these requirements.

1.1 BACKGROUND

The reactor vessel head lift rig, the internals lifting rig, load cell, load cell linkage, and reactor coolant pump motor lift sling were designed and built for the Turkey Point Units 3 and 4, circa 1967-68. The actual design criteria is unknown for the lifting devices. It appears that for the head lift rig, internals lift rig, load cell, cell linkage and the reactor coolant pump motor lift that in most cases the

design criteria used was that the resulting stress in the load members, when subjected to the total combined lifting weight, does not exceed one fifth (1/5) of the ultimate strength of the material. These items were not classified as nuclear safety components and thus requirements for formal documentation of design requirements and stress reports were not applicable. Thus, stress reports and design specifications were not formally documented. Westinghouse defined the design, fabrication and quality assurance requirements on detailed manufacturing drawings and purchase order documents. Westinghouse also issued field assembly and operating instructions, where applicable.

SECTIO 1 2 COMPONENT DESCRIPTION

2.1 REACTOR VESSEL HEAD LIFT RIG

The reactor vessel head lift rig^[4] is a three legged carbon steel structure, approximately 35 feet high and 14 feet in diameter, weighing approximately 25,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and control rod drive mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod sling assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operations, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2 REACTOR VESSEL INTERNALS LIFT RIG

The reactor vessel internals lift rig^[5] is a thret-legged carbon and stainless steel structure, approximately 29 feet high and 12 feet in diameter weighing approximately 13,000 pounds. It is used to handle the upper and lower reactor vessel internals packages. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The rig attaches to the internals packages by means of three engaging screws which are screwed into tapped holes in the internals flanges. These screws are manually operated from the spreader assembly using the internal tools. The screws are normally spring retracted upward and are depressed to engage the tapped holes.

2.3 LOAD CELL AND LOAD CELL LINKAGE

The load cell is used to monitor the load during lifting and lowering the reactor vessel internals to ensure no excessive loadings are occurring. It is installed between the load cell linkage and the lifting device. The load cell is a strain gage (tension) type, rated at 300,000 pounds, built by W. C. Dillon and Co. The load cell linkage is an assembly of pins, plates and bolts which connect the polar crane main hook to the load cell.

2.4 REACTOR COOLANT PUMP MOTOR LIFT SLING

The reactor coolant pump motor lift sling^[6] consists of a six (6) foot triangular carbon steel spreader assembly weighing approximately one thousand (1000) pounds. It has wire rope slings, shackles and turn-buckles attached which form a three point lift assembly. It is used to handle the reactor coolant pump motor.

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SECTION 3 DESIGN BASIS

3.1 DESIGN CRITERIA

NUREG 0612, paragraph 5.1.1(4) states that special lifting devices should satisfy the guidelines of ANSI N14.6. Further, NUREG 0612, 5.1.1(4) states: "In addition, the stress design factor stated in Section 3.2.1.1 of ANSI N14.6 should be based on the combined maximum static and dynamic loads that could be imparted on the handling device based on characteristics of the crane which will be used. This is in lieu of the guideline in Section 3.2.1.1 of ANSI N14.6 which bases the stress design factor on only the weight (static load) of the load and of the intervening components of the special handling device".

It can be inferred from this paragraph that the stress design factors specified in Section 3.2.1.1 of ANSI N14.6 (3 and 5) are not all inclusive. Also, it can be inferred that the specified ANSI N14.6 stress design factors should be increased by an amount based on the crane dynamic characteristics. The dynamic characteristics of the crane would be based on the main hook and associated wire ropes holding the hook. Most main containment cranes use sixteen (16) or more wire ropes to handle the load. Should the crane hook suddenly stop during the lifting or lowering of a load, a shock load could be transmitted to the connected device. Because of the elasticity of the sixteen or more wire ropes, the dynamic factor for a typical containment crane is not much larger than 1.0. The maximum design factor that is recommended by most design texts [7,8,9] is a factor of 2 for loads that are suddenly applied. The stress design factors required in Section 3.2.1.1 of ANSI N14.6 are:

- 3 (weight) < Yield Strength
- 5 (weight) < Ultimate Strength

The factor of 3 specified, certainly, includes consideration of suddenly applied loads for cases where the dynamic impact factor may be as high as 2.0. Thus, we feel that the use of the design criteria in ANSI N14.6 satisfies the NUREG requirement.

To provide flexibility on stress design factor, the summary tables list the stresses with stress design factors of 1, 3 and 5. Thus, any stress design factor may be easily applied to satisy any concerns.

3.2 DESIGN WEIGHTS

The following design weights were used in the analysis of the lifting devices:

- 3.2.1 Reactor Vessel Head Lift Rig
- (A) Design Weight for Lower Clevis and Pin (Items 14 and 15) is 253,000 pounds
- (B) Design Weight for rest of Lift Rig is 278,000 pounds
- 3.2.2 Reactor Vessel Internals Lift Rig, Load Cell, and Load Cell Linkage

The design weight is 260,000 pounds which is the total weight of the lifting device and the lower internals.

3.2.3 Reactor Coolant Pump Motor Lift Sling

The design weight is 81,000 pounds which is larger than the total weight of the lifting device and the reactor coolant pump motor. It is the safe working load identified on the assembly drawing.

SECTION 4 MATERIALS

4.1 MATERIAL DESCRIPTION

The materials and material properties for the reactor vessel head lift rig, the reactor vessel internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling are listed in Tables 4-1, 4-2 and 4-3.

TABLE 4-1
REACTOR COOLANT HEAD LIFT RIG MATERIAL AND MATERIAL PROPERTIES

Item(a)	Description	Materials	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
1 7 10 13 14	3 1/2" Diameter Pin 4" Dia. Bottom Clevis Pin 4" Dia. Upper Clevis Pin 7 1/2" Dia. Pin 8" Dia. Pin	ASTM A434 Class BD	110 110 105 100	140 140 135 130 130
2,5,15	Side Plate Support Lug	ASTM A515 or GR70	38	70
3	Leg	ASTM A36	36	58
4	Ring Girder	ASTM A285 GR. C	30	55
6,9	Upper Clevis Clevis	ASTM A237 CL A	50	80
8	Arm	ASTM A306 Gr. 70	35	70
11, 12	Clevis Plate Link Link Lug	ASTM A 105 CL 2	36	70

⁽a) See figure 5-1

TABLE 4-2
REACTOR VESSEL INTERNALS LIFT RIG MATERIAL AND MATERIAL PROPERTIES

Item(a)	Description	Materials	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
1	Hook Pin	ASTM A434 CL BD	100	130
3	Upper Adaptor Pin	ASTM A434 CL BD	105	135
7	Removable Pin			
11	Upper Sling Leg Pin	ASTM A 276 Type 304 Center Ground Cond. A		75
13	Lower Sling Leg Pin			
2	Side Plate	ASTM A-515 Gr. 70 or 4 ASTM A-516	38	70 70
8	Top Lugs	Gr. 70		
10	Side Leg			
12	Sling Leg			
14,B,C,D	Spreader Assembly			
4	Upper Adaptor	ASTM A-540 Gr. B-24	120	135
6	Lower Adaptor			
5	Tension Cell	17-4 pH H1100	115	140

⁽a) See figure 5-2

TABLE 4-2 (Cont)
REACTOR VESSEL INTERNALS LIFT RIG MATERIAL AND MATERIAL PROPERTIES

Item(a)	Description	Materials	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
9	Block	SA-105 CL 1 or 2	36	70
		SA-266 CL 1 or 2	30	60
		SA-508 CL 1 or 2	35	70
14A	Spacer Block			
15	NUT	ASTM A-276 Type 304 HR+PKLD, Cond. A	38	82
18	Leg Leveling Sleeve			
20	Support Ring Leveling Sleeve			
22	Guide Sleever			-
16	Leg Adaptor	ASTM A276 Type 304 HR+PKLD, Cond. A	43.9	86.5
23	Engaging Screen	ASTM A276 HR+PKLD, Cond. A	36.8	80.9
19	Coupling	ASTM A312,	30	75
17	Leg Outer Tube	Type 304 ASTM A312 Type 304 Seamless, Cold Fanish and Heat Treat	38.0	83.4
21	Support Ring Outer Tube			le de la Constantina

⁽a) See figure 5-2

TABLE 4-3
REACTOR COOLANT PUMP MOTOR LIFT SLING MATERIALS AND
MATERIAL PROPERTIES

Item(a)	Description	Materials	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
1	Spreader (pipe)	ASTM A106 Grade B	35	60
2	Side plates	ASTM 212 Grade A	36	58

⁽a) See figure 5-3

SECTION 5 SUMMARY OF RESULTS

Tables 5-1, 5-2, and 5-3 summarize the stresses on each of the parts which make up the reactor vessel head and internals lift rig, load cell, load cell linkage and reactor coolant pump motor lift sling, respectively. All of the tensile and shear stresses, with the exception of the tensile stress at the minimum section of the internals lifting rig engaging screw, item (23), meet the design criteria of section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with the accompanying allowable stress limits of yield and ultimate strength, respectively. Application of the ANSI N14.6 criteria to pins subject to bending, structural members subject to buckling and bending loads, and various parts subject to bearing loads result in some stresses exceeding this criteria. However, when using more appropriate criteria, the resulting stresses are acceptable.

- 5.1 DISCUSSION OF RESULTS
- 5.1.1 Application of ANSI N14.6 Criteria

The design criteria of section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with the accompanying allowable stresses, are to be used for evaluating load bearing members of a special lifting device when subjected to loading conditions resulting in shear or tensile stresses. Application of these design load factors to other loading conditions is not addressed in ANSI N14.6. However, these two stress design factors have been used to determine the stresses of the load carrying members when subject to other loading conditions, viz. bearing, bending, buckling. This is an extremely conservative approach and in some cases the resulting stresses exceed the accompanying allowable stress limit.

In the internals lift rig engaging screw (item 23) the tensile stress slightly exceeds the criteria that three times the calculated stress must be less than the yield stress. However, the conservative criteria

that five times the calculated stress must be less than the ultimate is met and thus the engaging screw is considered acceptable.

For items 11, 13, 15, 22 and 23 of the internals lifting rig, the bearing stress exceed the criteria that three times the calculated stress must be less than the yield stress. However, all the bearing stresses meet the conservative criteria that five times the calculated stress must be less than the ultimate stress and thus the parts are considered acceptable for bearing.

The bending stresses in the internals lifting rig pins (items 3, 7, 11 and 13) were calculated using the conservative approach shown in the Machine Design Fastening and Joining Issue. This approach, coupled with the use of the ANSI 14.6 stress factor for this condition, results in bending stresses exceeding the allowables. However, the acceptance criterion for pin design is shear stresses and the results of all shear stress calculations are below acceptable limits. The bending stress calculations are included for reference.

The bending stress in the internals lifting rig spacer blocks (item 14-A) does not meet the criteria of section 3.2.1.1. However, this is a local fiber stress. Even if the fiber stress reached the yield stress, and it doesn't, the rest of the cross-section could assume the additional load. This localized stress can be considered under section 3.2.1.2 which states that the stress design factors of 3.2.1.1 are not intended to apply to situations where high load stresses are relieved by slight yielding. The shear stress in the block is extremely low and well within the section 3.2.1.1 criteria.

Structural elements loaded in compression and bending are analyzed by the empirical equations of the ASME and AISC rules. Buckling stresses are expressed as the ratios of actual stresses to the allowable stresses with the acceptable ratio being < 1.0. These equations do not determine the limiting stresses of members in buckling but indicate whether or not the calculated stress is or is not within the allowable values. Instead, the ultimate load carrying capability of the member is the

determining factor in the structural member's acceptability. Timoshenko^[12] notes that the ultimate load for short struts is equal to the material yield point. Calculation of the ultimate load results in this load being 1.3 times the nominal design load and thus, these members are considered acceptable.

5.1.2 Pin Bending

Several of the internal lifting rig pins do not meet the ANSI N14.6 criteria (3 and 5) when analyzed for bending stresses. In calculating the bending stresses in pins, it is assumed that loads from the outside lugs are linerally distributed and from the inside eye are uniformly distributed along the pin. Further, the assumption is made that span to diameter ratios are large enough such that the assumptions inherent to simple beam theory are valid (Neither condition is met, however, by the actual joint geometry).

Pin deflections and local yielding of the pin, lugs, and eye cause the loads to be non-uniform and their centroids to shift towards the interfaces between the lugs and eye, i.e., the shear planes in the pins. This concentration of load near the shear planes reduces the effective bending arms at which loads are applied thereby reducing the bending moment.

The calculated bending stresses are thus over estimated and are tabulated for reference. Shear stresses in the pins are the governing parameter for pin strength. Using shear stresses as the criteria for pin design results in all pins meeting three times the calculated stress being less than the yield stress and the ANSI criteria of 5 times the calculated stress being less than the ultimate stress. Thus all the above pins are considered acceptable.

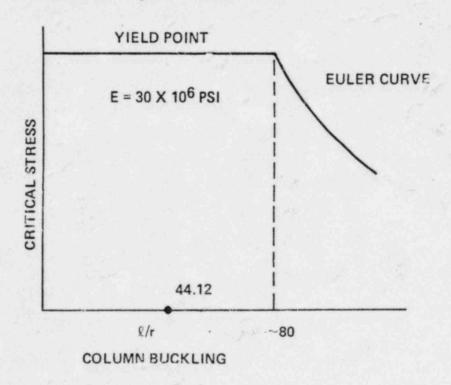
5.1.3 Structures Loaded in Compression and Bending

The spreader assemblies of the reactor vessel internals lift rig and the reactor coolant pump motor lift sling do not meet all the ANSI N14.6 criteria (3W and 5W) when analyzed for combined axial compression and

bending (buckling) or bending loadings. Structures loaded in combined compression and bending are analyzed by the empirical equations of the [10] ASME Boiler and Pressure Vessel Code Section III, Appendix XVII. (Same as the AISC [11] Part 5 rules). Ratios less than 1.0 indicate the calculated stress is within the allowable values; greater than 1.0, the stress is greater than the allowables.

When comparing the spreader arm to the above criteria, the acceptable ratio < 1.0 is not satisfied. However, these empirical equations do not identify the maximum stress for this loading condition. When the ratio is >1.0, these equations identify an unacceptable design condition. If we were designing a new structure, the material and member size would be changed to ensure these ratios would be satisfied for all loading conditions. However, these calculations are being applied to an existing structure and since these conditions are not satisfied then the ultimate load carrying capability must be determined.

The column under consideration is relatively short ($\frac{k\epsilon}{r}$ = 44.12). Timoshenko^[12] states that experiments show that short columns buckle when the compressive strength reaches the material yield point. (The horizontal line on the figure below).



Therefore the total stress

$$\sigma = \frac{P}{A} + \frac{M}{Z} = Direct Stress + Bending Stress$$

must be less than or equal to the material yield stress.

Since the column is loaded eccentrically (i.e., M is proportional to P, so the two could be equivalently replaced with a load P displaced from the column centerline by a distance e) as the column bends the effective lever arm of the load increases. For this condition

$$\sigma = \frac{P}{A} + \frac{Pe}{Z} \quad \text{secant} \quad \sqrt{\frac{P}{EI} \frac{\hat{x}}{2}}$$

For this particular column (internals lift rig spreader)

the expression
$$\left[\begin{array}{c} \text{secant} & \sqrt{\frac{P}{EI}} \frac{z}{7} \end{array}\right]$$

is essentially 1.0 for even 5 times the nominal load and the equation reverts to

$$\sigma = \frac{P}{A} + \frac{Pe}{Z} = \frac{P}{A} + \frac{M}{Z}$$

For the case of the internals lift rig spreader;

$$\frac{P}{A}$$
 = 3290 psi and $\frac{M}{Z}$ = 25,135 psi

Then the total stress is:

which is less than the material yield strength (Sy)

Then to find the ultimate column load, let $\sigma_{\text{max}} = S_y = 38,000 \text{ psi}$

Then the maximum column load is the ratio of

$$\sigma_{\text{max}}/\sigma_{\text{total}} = \frac{38,000}{28,425} = 1.337$$

Thus the ultimate column load is 1.3 times the nominal value.

The internals lift rig spreader members are considered acceptable for this condition of combined stresses for axial compression and bending.

Similarly, the ultimate load for the reactor coolant pump motor lift sling is 3.7 times the nominal value and also considered acceptable.

5.1.4 Rated Load Values of the Reactor Coolant Pump Motor Lift Sling

Since most of the components that comprise the reactor coolant pump motor lift sling are non-designed components application of the criteria of section 3.2.1.1 of ANSI N14.6 to these components are not appropriate. Therefore, Table 5-4 has been prepared from catalog information and list the various load conditions. Noting that the safety factors are based on the ratio of the ultimate strength of the material to the rated load value, the sling is acceptable for five times the design load.

5.2 CONCLUSION

Application of the ANSI N14.6 criteria of (3 and 5) to these special lifting devices results in acceptable stress limits for tensile and shear stresses with the exception of the internals lifting rig engaging screw. Application of this criteria to all structural members subject to other types of loadings tend to result in oversimplified conservatism and with some stresses exceeding the accompanying allowable limits. However, when using the more appropriate criteria for those cases not addressed by the ANSI N14.6 criteria the stresses are within the appropriate allowable limits.

TABLE 5-1 SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG

Item(a)	Part Name	Calculated Str	1.50	Material Aliowable			
No.	And Material	Designation	W(P)	Value 3W	I 5W	C (C) (I	(si)
	7110 710007107	aco ignation		JW.) MC	Sy(c)	Sult (d)
1	3-1/2" dia. pin	Shear	4.8	14.4	24.0	110	140
	ASTM A434	Bearing on Head Clevis	8.8	26.4	44.0	110	140
	Class BD	Bearing on Clevis Plate	8.3	24.9	41.5		
		Bending	17.0	51.0	85.0		
2	Clevis Plate	Shear Tear-Out at Pin Hole	4.1	12.3	20.5	38	70
	ASTM A515	Bearing on Pin	8.3	24.9	41.5	30	70
	Grade 70 Q&T	Tension at Pin Hole	4.1	12.3	20.5		
3	Leg	Fillet Weld on Clevis				weld allowable = 18(e)	
7.24	ASTM A36	Plate to Leg - Shear	7.8	12.0	20.0		1
		Tension	7.8	23.4	39.0	36	58
4	Ring Girder	Total Shear	2.4	7.2	12.0	30	
	ASTM A285 GRC Firebox	Combined Tensile Weld of Ring Girder to	3.5	10.5	17.5	30	55
W. Trial		Support Lug	2.4	7.2	12.0		

(a) See figure 5-1 for location of item number and section

 ⁽b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)
 (e) Stress limit for fillet weld from ASME Boiler and Pressure Vessel Code, Section III Division 1 - Subsection NF 1980 Edition, Table NF-3292 1-1, page 43

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG

Item(a)	Part Name	Calculated Stre	esses (ksi)	W-3			Allowable
No.	And Material	Designation	М(р)	Value 3W	5W	Sy(c)	Sult (d)
5	Support Lug ASTM A515 GR70 Normalized and Tempered	Tension at Pin Hole Shear-Tear-Out at Pin Hole Bearing on Pin	5.8 5.0 7.5	17.4 15.0 22.5	29.0 25.0 37.5	38	70
6	Clevis ASTM A237 Class A	Thread Shear Bearing on Pin Shear Tear-Out at Pin Hole Tension at Pin Hole Tension at Thd. Relief	3.4 6.7 4.3 4.3 2.1	10.2 20.1 12.9 12.9 6.3	17.0 33.5 21.5 21.5 10.5	50	80
7	4" Dia Pin ASTM A434 Class BD Min. Tempering Temp. = 1100°F	Shear Bearing on Support Lug Bearing on Lower Clevis Bending	4.1 7.5 6.7 14.3	12.3 22.5 20.1 42.9	20.5 37.5 33.5 71.5	105	135
8	Arm ASTM A306 GR 70	Thread Shear Thread Tension	3.4 9.2	10.2 27.6	17.0 46.0	35	70

(a) See figure 5-1 for location of item number and section
 (b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG

Item(a)		Calculated Stres	ses (ksi)			Materia	1 A: lowable
No.	Part Name And Material	Doctoration	W(b)	Value	T 5:1	(6)	(si)
10.	And Material	Designation	Wish	3M	5W	Sy(c)	Sult (d)
9	Upper Clevis	Thread Shear	3.4	10.2	17.0	50	80
		Bearing on Pin	6.7	20.1	33.5		00
		Shear Tear-Out at Pin Hole	4.3	12.9	21.5		
		Tension at Pin Hole	4.3	12.9	21.5		
		Tension at Thd. Relief	2.1	6.3	10.5		
10	.4" Dia Upper Clevis	Shear	4.1	12.3	20.5	105	135
	Pin	Bearing on Link Lug	7.3	21.9	36.5	103	133
	ASTM A434 Class BD	Bearing on Upper Clevis	6.7	20.1	33.5		
	Min. Tempering Temp. = 1100°F	Bending	14.1	42.3	70.5		
11	Link	Bearing on Hook Pin .r.	• ,4.3	12.9	21.5	36	70
	A105	Tension at Pin Hole	4.0	12.0	20.0	30	70
	Class 2	Shear Tear-Out at Pin Hole	4.0	12.0	20.0		
		Tension at Cylindrical Section	5.3	15.9	26.5		

(a) See figure 5-1 for location of item number and section

(b) W is the total static weight of the component and the lifting device (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG

Item(a)		Calculated Stres	ses (ksi)			The second secon	Allowable
No.	Part Name And Material	Designation	M(P)	Value 3W	5W	Sy(c) (k	Sult (d)
12	Link Lug A105 Class 2	Tension at Pin Hole Shear Tear-Out at Pin Hole Bearing at Pin Hole Shear at Root of Lug Combined Tension at Lug Root	4.9 4.9 7.3 1.6 3.7	14.7 14.7 21.9 4.8 11.1	24.5 24.5 36.5 8.0 18.5	36	70
13	7-1/2" Dia. Pin ASTM A434 Class BD	Shear Bearing on Hook Bearing on Side Plate Bending	3.2 4.6 6.2 10.7	9.6 13.8 18.6 32.1	15.0 23.0 31.0 53.5	100	130
14	8" Dia. Pin ASTM A434 Class BD	Shear Bearing on Link Bearing on Side Plate Bending	2.8 4.3 5.8 8.8	8.4 12.9 17.4 26.4	14.0 21.5 29.0 44.0	100	130
			n				
15	Side Plate ASTM A514	Bearing at 7-1/2" Hole Tension at 8" Hole Shear Tear-Out at 8" Hole	6.2 5.5 5.5	18.6 16.5 16.5	31.0 27.5 27.5	90	100

(a) See figure 5-1 for location of item number and section (b) W is the total static weight of the component and the lifting device (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

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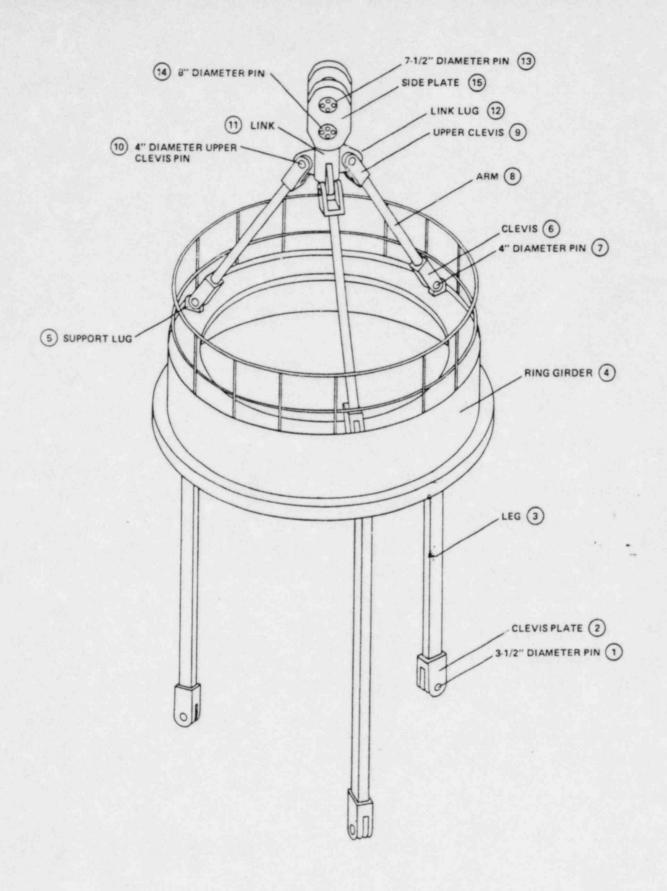


Figure 5-1. Reactor Vessel Head Lifting Rig

TABLE 5-2 SUMMARY OF RESULTS REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE

Item(a)	Part Name	Calculated Stre	esses (ks			Material Allowable	
No.	And Material	Designation	W(p)	Value 3W	5W	Sy(c) (ksi	Sult (d)
1	(Hook) Pin ASTM A434 Class BD AISI 4340 Hot Rolled and Quenched & Tempered	Shear Bearing on Hook Bearing on Side Plates Bending	3.0 4.4 10.1 9.3	9.0 13.2 30.3 27.9	15.0 22.0 50.5 46.5	100	130
2	Side Plate ASTM A515 GR 70 or .ASTM A516 GR 70	Tension at 7.515 Dia. Hole Bearing at 7.515 Dia. Hole Shear Tear-out at 7.515 Dia. Hole	10.1 10.1 10.1	30.3 30.3 30.3	50.5 50.5 50.5	ASTM A515 38 ASTM A516	70
	Q&T	Tension at 4.385 Dia. Hole Shear Tear-out at 4.385 Dia. Hole Bearing at 4.385 Dia. Hole	9.2 9.2 10.6	27.6 27.6 31.8	46.0 46.0 53.0	38	70
3	(Upper Adaptor) Pin ASTM A434, Class BD, AISI 4340 Hot Rolled and Q&T	Shear Bearing on (Upper) Adaptor Bearing on Side Plate Bending	8.8 *9.6 10.6 42.4	26.4 28.8 31.8 127.2	44.0 48.0 53.0 212.0	105	135

(a) See figure 5-2 for location of item number and section (b) W is the total static weight of the component and the lifting device (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)	Part Name	Calculated Str	esses (ksi			Material Allowable	
No.	And Material	Designation	М(р)	Value 3W	5W	Sy(c) (k	Sult(d)
4	(Upper) Adaptor ASTM A540 Grade B-24	Tension at 4.387 Dia. Hole Bearing at 4.387 Dia. Hole Tension at Thread Relief Thread Shear Shear Tear-out at 4.387 Dia. Hole	10.6 9.5 7.0 11.2 9.1	31.8 28.5 21.0 33.6 27.3	53.0 47.5 35.0 56.0 45.5	120	135
5	Tension Cell 17-4 pH ss H-1100°	Tension at Threads Thread Shear	19.4	58.2 33.6	97.0 56.0	115	140
6	(Lower) Adaptor	Tension at 4.387 Dia. Hole Bearing at 4.387 Dia. Hole Tension at Thread Relief Thread Shear Shear Tear-out at 4.387 Dia. Hole	10.6 9.5 7.0 11.2 9.1	31.8 28.5 21.0 33.6 27.3	53.0 47.5 35.0 56.0 45.5	120	135
7	(Removable) Pin ASTM A434 Class BD, AISI 4340, Hot Rolled, Q&T	Shear Bearing on Lower Adaptor Bearing on Top Lugs Bending	8.8 9.6 10.0 42.4	26.4 28.8 30.0 127.2	44.0 48.0 50.0 212.0	105	135

(a) See figure 5-2 for location of item number and section (b) W is the total static weight of the component and the lifting device (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

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TABLE 5-2 (cont) SUMMARY OF RESULTS REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE

Item(a)	0	Calculated Stresses (ksi)				Material Allowable	
No.	Part Name And Material	Designation	W(D)	Value 3W	5W	Sy(c) (ksi	Sult(d)
3	Top Lugs ASTM A515 GR 70 or ASMT A516 GR 70	Bearing on Pin Tension at Pin Hole Shear Tear-out at Pin Hole	10.0 9.8 9.8	30.0 29.4 29.4	50.0 49.0 49.0	ASTM A515 38 ASTM A516	70
	Q&T	Tension at Weld	4.9	14.7	24.5	38	70
9	Block SA 105 CL 1 or 2 or ASTM A266 CL 1	Tension in Block	1.5	4.5	7.5	SA 105 36 ASTM A266 CL1 30 CL 2 35	70 60 70
	SA 508 CL 1 or 2	•				SA 508 CL 2 35 CL 2 50	70 80
10	ASTM ASIS GR 70	Tension at Pin Hole Shear Tear-out at Pin Hole Bearing on Pin	5.6 5.6 9.1	16.8 16.8 27.3	28.0 28.0 45.5 9.5	ASTM A515 38 ASTM A516	70
	ASTM A516 GR 70 Q&T CL 2	Shear at Lug Root Weld Combined Bending and Tension at Lug Root Weld	1.9	5.7 16.5	26.0	38	70

(a) See figure 5-2 for location of item number and section
 (b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)

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Item(a)	Part Name	Calculated Stresses (ksi) Int Name Value					llo able
No.	And Material	Designation	M(p)	3W	5W	Sy(c) (ksi	Sult(d)
11	(Upper Sling-Leg) Pin ASTM A276 Type 304 Cent. Grd. Cond. A	Shear Bearing on Sling Leg Bearing on Side Lug Bending	7.5 11.8 9.1 28.8	22.5 35.4 27.3 86.4	37.5 59.0 45.5 144.0	30	75
12	Sling Leg ASTM A515, GR 70 or ASTM A516, GR 70	Bearing on Pins Tension in Leg Tension at Pin Hole Shear Tear-out at Pin Hole	11.8 5.9 12.0 12.0	35.4 17.7 36.0 36.0	59.0 29.5 60.0 60.0	ASTM A515 38 ASTM A516 38	70 70
13	(Lower Sling Leg) Pin ASTM A276 Type 304 Cent. Grd. Cond. A	Shear Bearing on Sling Legr. r Bearing on Spreader Bending	7.5 11.8 6.1 35.3	22.5 35.4 18.3 105.9	37.5 59.0 30.5 176.5	30	75

(a) See figure 5-2 for location of item number and section
 (b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)

(-)		Calculated Str	Material Allowable				
Item(a) No.	Part Name And Material	Designation	. M(P)	Value 3W	5W	Sy(c) (ks	Sult(d)
14	SPREADER					SA 105 36	90
14-A	Spacer Block SA 105 CL 1 or 2 Q&T, or ASTM A266 CL 1 or 2 or SA 508 CL 1 or 2 Q&T	Bending Shear Bearing	16.3 3.4 6.1	48.9 10.2 18.3	81.5 17.0 30.5	ASTM A266 30 CL1 35 CL2 SA508 35 CL1 50 CL2	60 CL1 70 CL2 70 CL2 80 CL2
14-B, C, D	Spreader Assembly ASTM A515 GR 70 or ASTM A516 GR 70	Nominal Compression Buckling Welds (max. shear stress)	3.3 1.2 11.1	9.9 NA 33.3	16.5 NA 55.5	38 Ratio 18(f)	70 < 1.0(g)

(a) See figure 5-2 for location of item number and section

(b) W is the total static weight of the component and the lifting device

 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)
 (f) Stress limit for fillet weld from ASME Boiler & Pressure Vessel Code, Section III, Division 1 -Subsection NF 1980 Edition, Table NF - 3292.1-1 page 50

(g) Stress limit ratio's for buckling from ASME Boiler & Pressure Vessel Code, Section III, Division 1, Appendices, Article XVII-2215, 1980 Ed.

tem(a)	Part Name	Calculated Stresses (ksi)				Material Allowable	
No.	And Material	Designation	M(p)	Value 3W	5W	Sy(c) (ks	Sult(d)
15	NUT ASTM A276 Type 304 HR & PKLD. Cond. A	Thread Shear Bearing on Spacer Block	11.9 14.5	35.7 43.5	59.5 72.5	38(e)	82(e)
16	(Leg) Adaptor ASTM A276 Type 304 HR & PKLD. Cond. A	Thread Shear Tension at Threads	11.9	35.7 30.6	59.5 51.0	43.9(e)	86.5(e)
17	(Leg) Outer Tube ASTM A312 Type 304 SMLS CF and HT. TR.	Tension .F.	10.3	30.9	51.5	38.0(e)	83.4(e)
18	(Leg) Leveling Sleeve ASTM A276 Type 304 HR & PKLD. Cond. A.	Tension at 6.60 Dia. Thread Shear	3.2 4.0	9.6	16.0 20.0	30	75

(a) See figure 5-2 for location of item number and section

(b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sylt is the ultimate strength of the material (ksi)
 (e) These are actual Sy and Sylt taken from the material certification

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Item(a)	Part Name	Calculated Stresses (ksi)					Material Allowable	
No.	And Material	Designation	. W(b)	Value 3W	5W	Sy(c)	Sult (d)	
19	Coupling ASTM A312 Type 304 SMLS CF & HT Tr.	Thread Shear Tension at Thread Relief	4.0	12.0 13.5	20.0 22.5	30	75	
20	(Support Ring) Leveling Sleeve ASTM A276 Type 304 HR & PLKD. Cond. A.	Thread Shear on 7.000-8UN Thread Tension at THD. Relief Thread Shear on 5.500-12UN Thread	4.0 5.6 4.9	12.0 16.8 14.7	20.0 28.0 24.5	30	75	
21	(Support Ring) Outer Tube ASTM A312 Type 304 SMLS CF & HTTR	Thread Shear Tension at Thd. Relief Tension at Weld	4.9 8.4 6.5	14.7 25.2 19.5	24.5 42.0 32.5	30	75	

(a) See figure 5-2 for location of item number and section
 (b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)

Item(a)	David Name	Calculated Str	Material Allowable				
	Part Name And Material	Docimentia	W(b) Value			(ksi)	
	And Macerial	Designation	M(D)	3W	5W	Sy(c)	Sult (d)
22	Guide Sleeve ASTM A276 Type 304 HR & PKLD. Cond. A	Bearing on Engaging Screw Nominal Compression Below Engaging Screw	13.6 8.3	40.8 24.9	68.0 41.5	30	75
23 .	Engaging Screw ASTM A276 Type 304	Bearing on Guide Sleeve Tension at Minimum Section Thread Shear	13.6 13.5 5.1	40.8 40.5 15.3	68.0 67.5 25.5	36.8(e;	80.9(e)

(a) See figure 5-2 for location of item number and section

(b) W is the total static weight of the component and the lifting device
 (c) Sy is the yield strength of the material (ksi)
 (d) Sult is the ultimate strength of the material (ksi)
 (e) These are actual Sy and Sult taken from the material certification

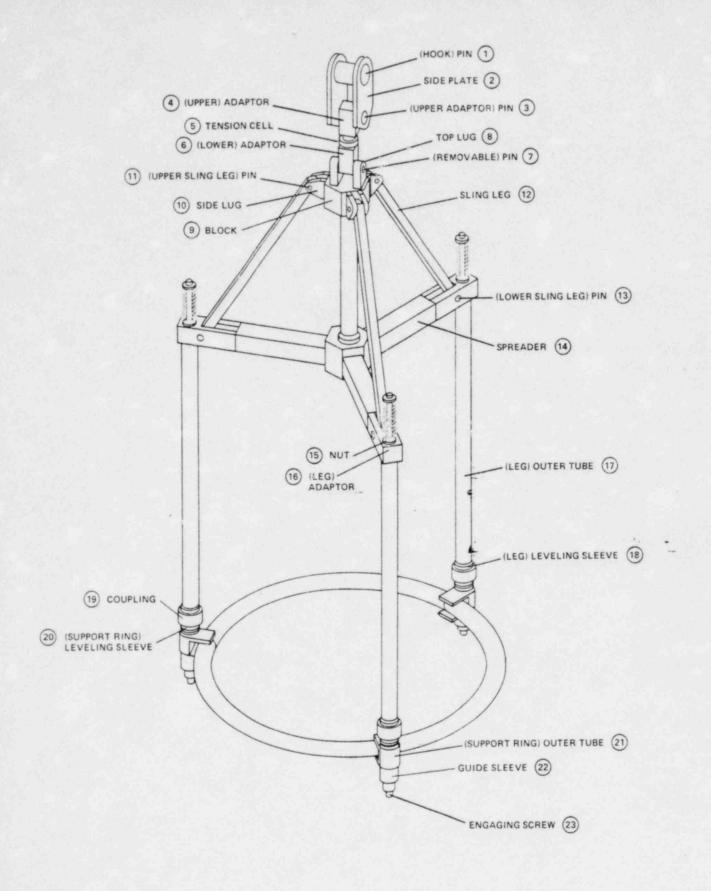


Figure 5-2 Reactor Vessel Internals Lifting Rig

I tem(a) No.	Part Name And Material	Calculated Stresses (ksi)					Material Allowable		
			Value			(ksi)			
		Designation	M(p)	3W	5W	s _y (c)	Sult (d)		
1	Spreader	Compressive Buckling	5.2	15.4	25.7	F _c (e)	= 19.35 ksi		
		Stress on Tube-to-Tube Weld	2.9	8.6	14.3	35	60		
		Stress on Tube-to-Plate Weld	7.1	21.3	35.5				
2	Plate	Tension	1.9	5.6	9.3	36	58		

(a) See Figure 5-3 for location of item number and section(b) W is the total static weight of the component and the lifting device

(c) S_y is the yield strength of the material (ksi) (d)- S_{ult} is the ultimate strength of the material (ksi)

(e) F_c is the compressive buckling strength of the material (ksi)

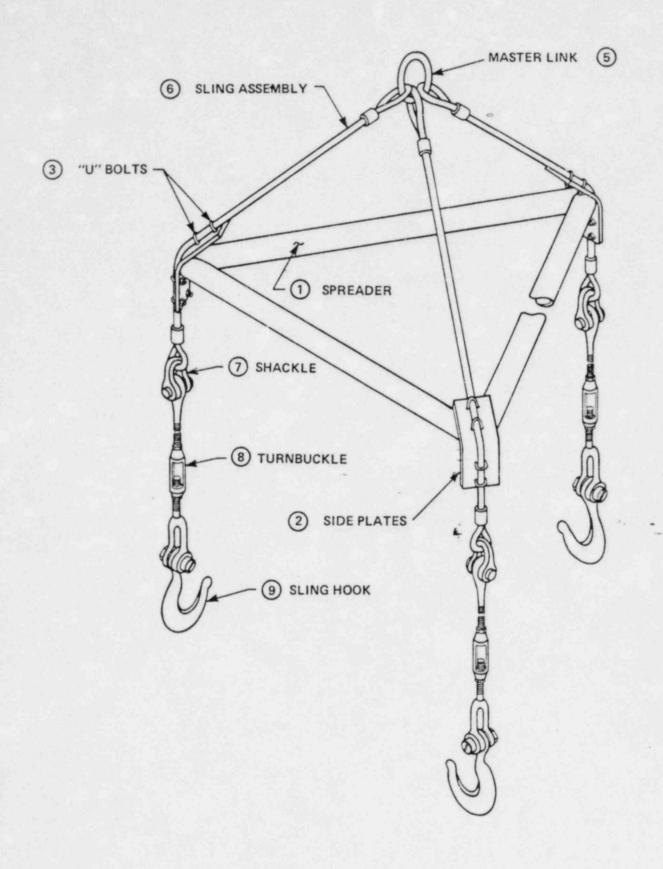


Figure 5-3. Reactor Coolant Pump Motor Lift Sling

TABLE 5-4

COMPARISON OF DESIGN LOADS

AND RATED LOAD VALUES OF THE NON-DESIGNED

ITEMS OF THE R.C. PUMP MOTOR LIFT SLING

No.(7)			Loads (Pound		Safety Factor(4)	
	Item	Design	Rated(1) Load Value	Proof(2) Load	Ultimate(3) Load	
5	Master Link	81,000	160,000	320,000	544,000	3.4:1
6	Sling	81,000	81,000 ⁽⁵⁾	94,000 ⁽⁵⁾	405,000	5:1 ⁽⁵⁾
7	Shackle	27,000	70,000	154,000	420,000	6:1
8	Turnbuckle	27,000	37,000	74,000	185,000	5:1
9	Hook	27,000	38,750	77,500	131,750	3.4:1

NOTES:

- (1) RATED LOAD VALUE The maximum recommended load that should be exerted on the item. The following terms are also used for the term Rated Load: "SWL", "Safe Working Load", "Working Load", "Working Load Limit", and the "Resultant Safe Working Load." All rated load values, are for in-line pull with respect to the centerline of the item.
- (2) PROOF LOAD The average force to which an item may be subjected before visual permanent deformation occurs or a force that is applied in the performance of a proof test.
- (3) ULTIMATE LOAD The average load or force at which item fails or no longer supports a load.
- (4) SAFETY FACTOR An industry term denoting theoretical reserve capability. Usually computed by dividing the catalog stated ultimate load by the catalog stated working load limit and generally expressed as a ratio, for example 5 to 1.

TABLE 5-4 (Cont)

- (5) This information is as stated on Westinghouse drawing AED SK 618J644 TXK Sub 5 as follows: "Safe working load of this sling assembly is 81,000 lb and a safe factor of 5:1." Catalog information is not applicable.
- (6) The rated load value, proof load, ultimate load and safety factor information was obtained from the following vendor catalogs:
 - a. S.G. Taylor Chain Co., Inc., Bulletin AS-67 Alloy Steel Chain Assemblies, Attachments for items 5 and 9.
 - b. Pennsylvania Sling Co. for item 6.
 - c. Crosby Group, 950 General Catalog, June 1981 for items 7 and 8.
- (7) Refer to figure 5-3 for identification of items.

APPENDIX A DETAILED STRESS ANALYSIS - REACTOR VESSEL HEAD LIFT RIG

This appendix provides the detailed stress analysis for the Turkey Point Units 3 and 4 Reactor Vessel Head Lift Rig in accordance with the requirements of ANSI N14.6. Acceptance criteria used in evaluating the calculated stresses are based on the material properties given in section 5.

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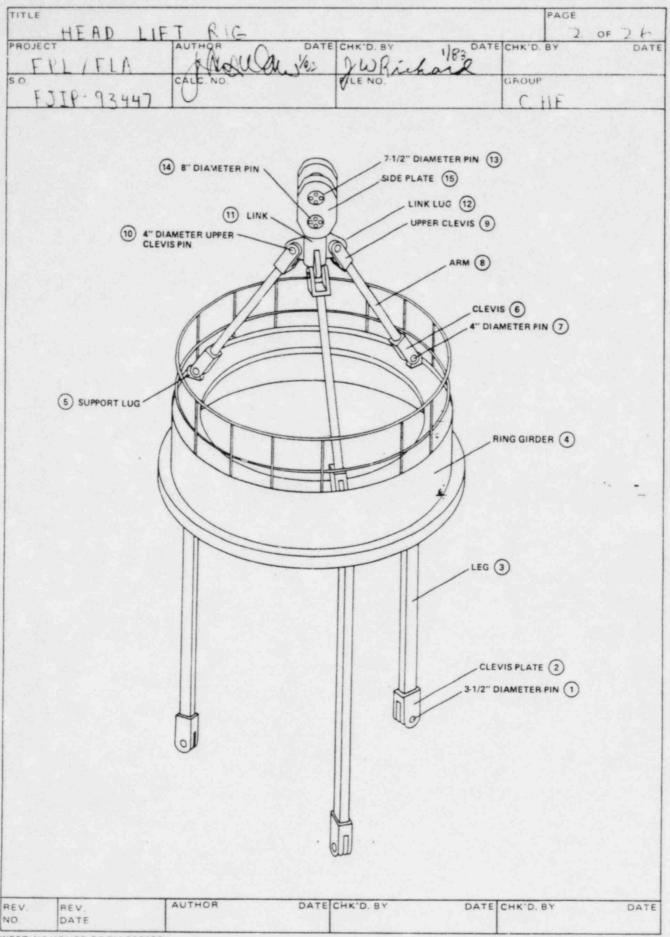
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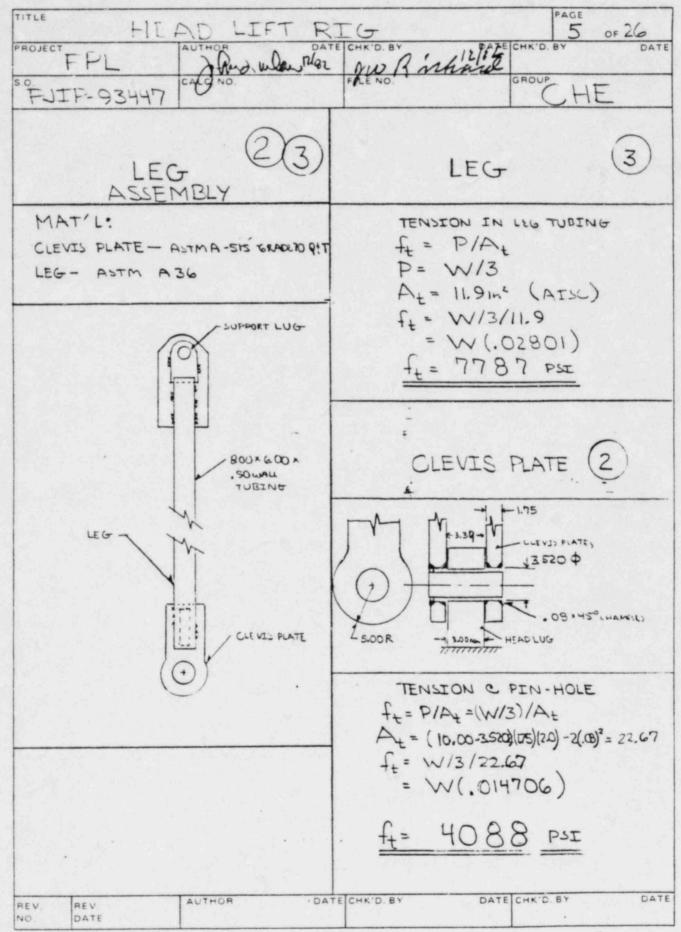
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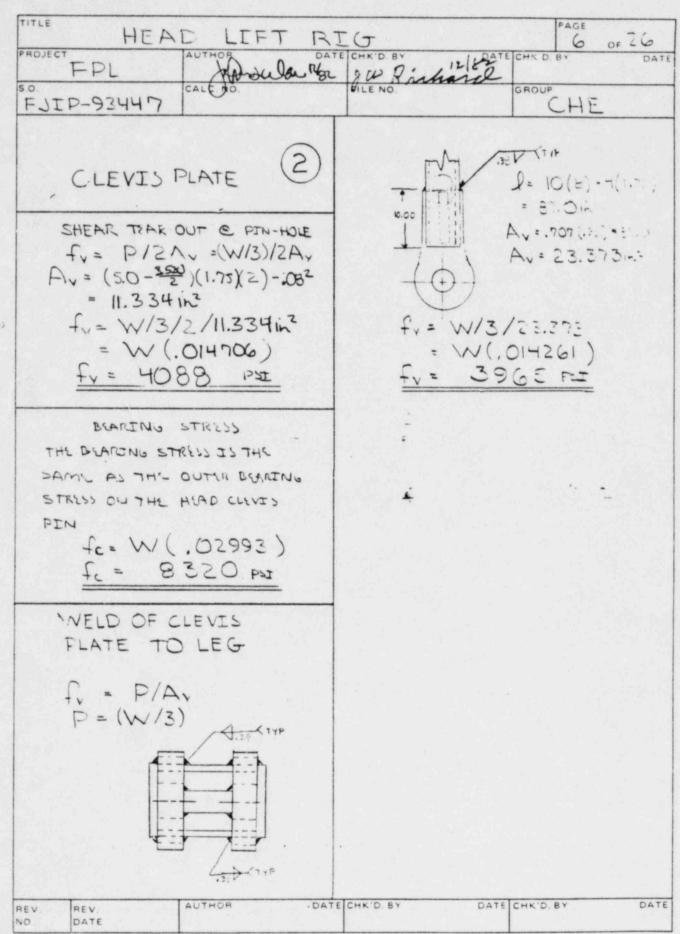
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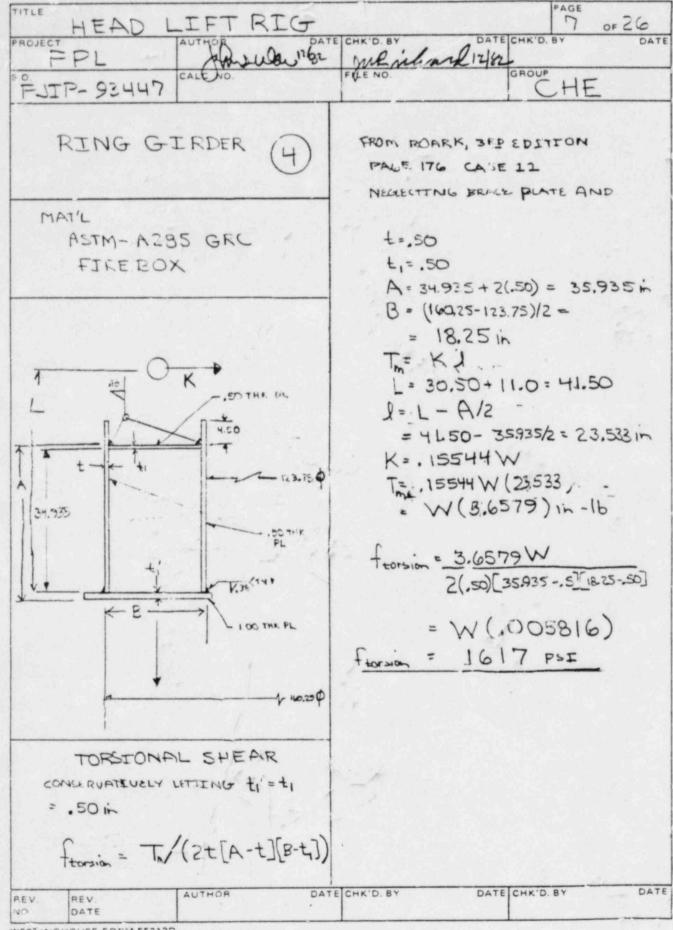
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BENDING STRESS $f_b = 16P(\frac{3}{2}+g+\frac{9}{4})/(\pi d^3)$ $= 16*(W/3)*(\frac{59}{3}+.265+\frac{3.90}{4})$ $/(\pi*3.5025^3)$ $= W(.06104)$ $f_b = 16.970$ psi

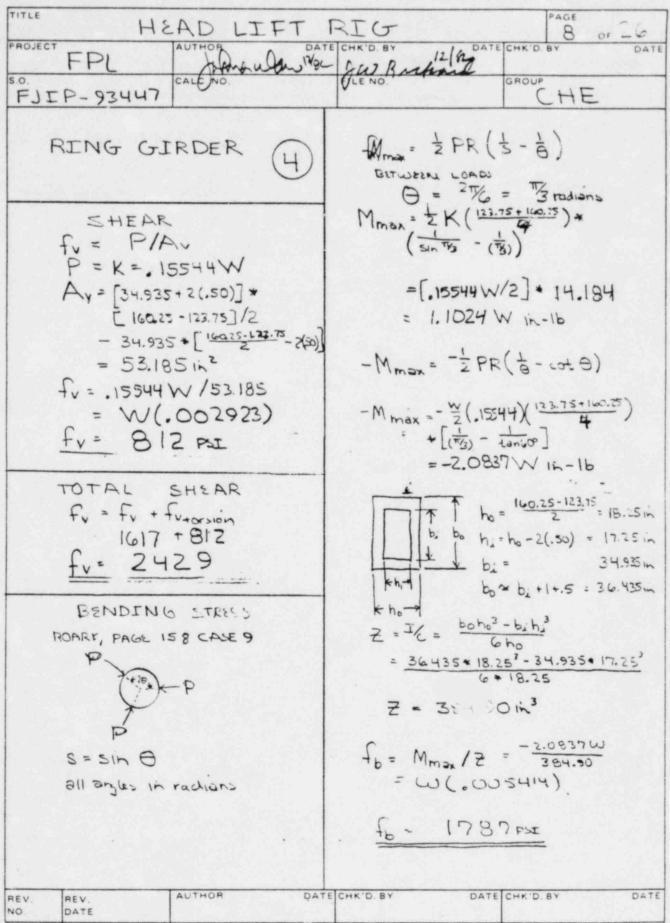


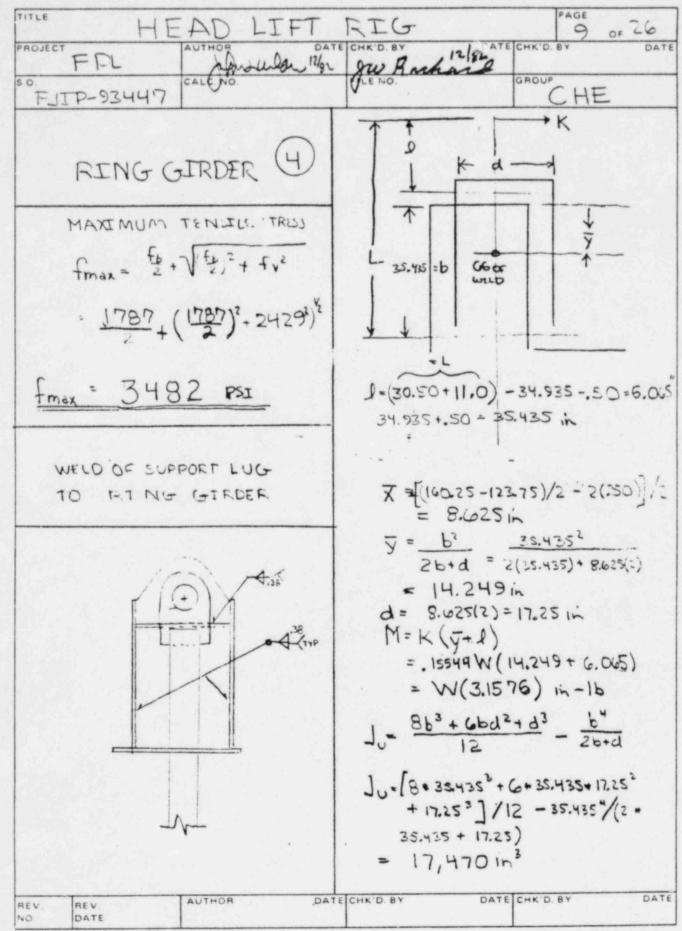


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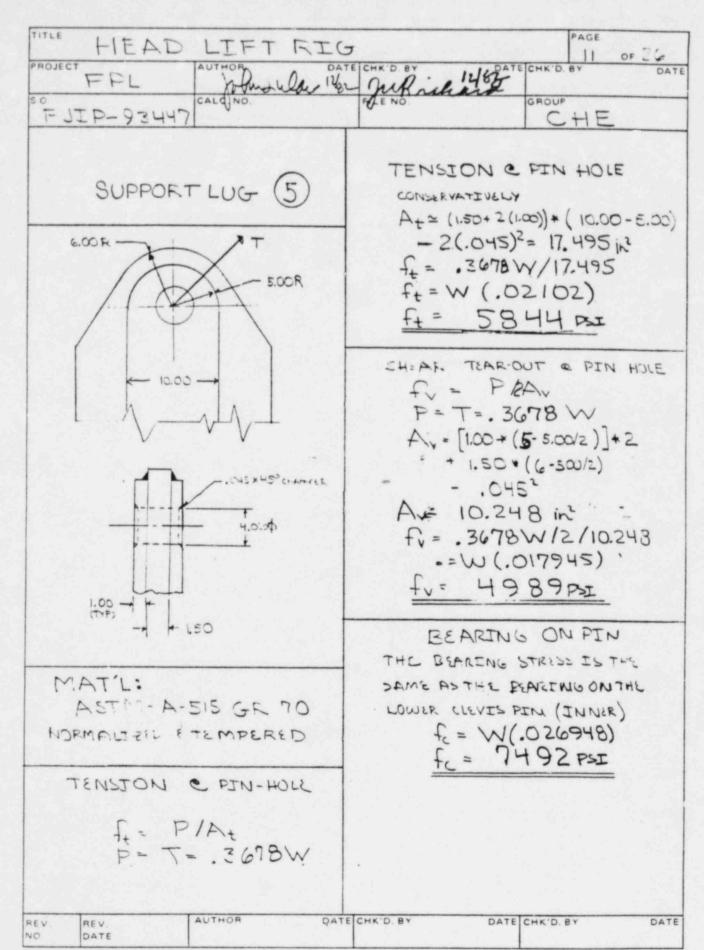
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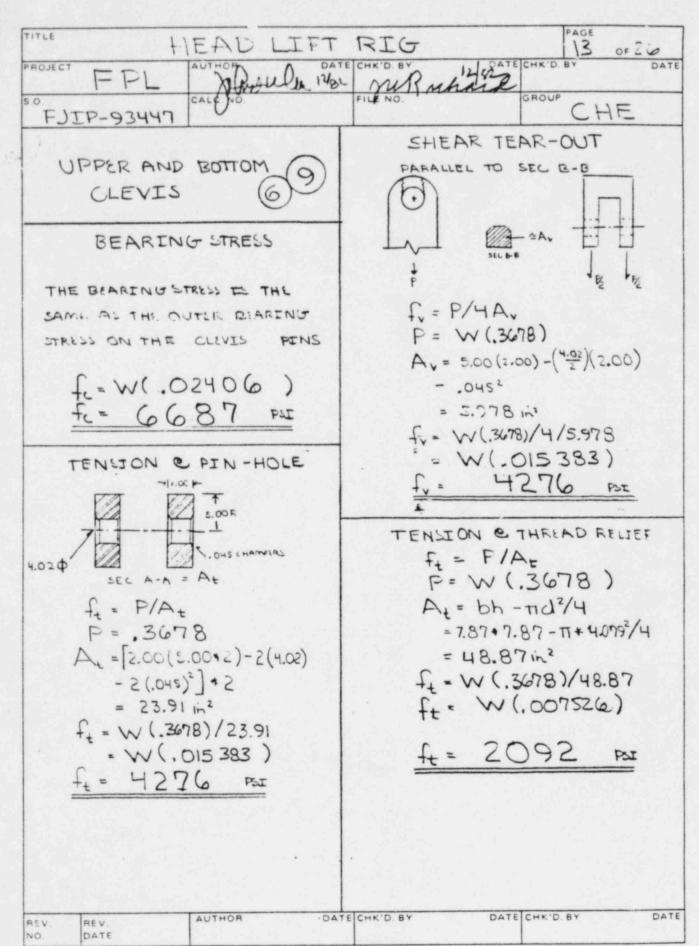


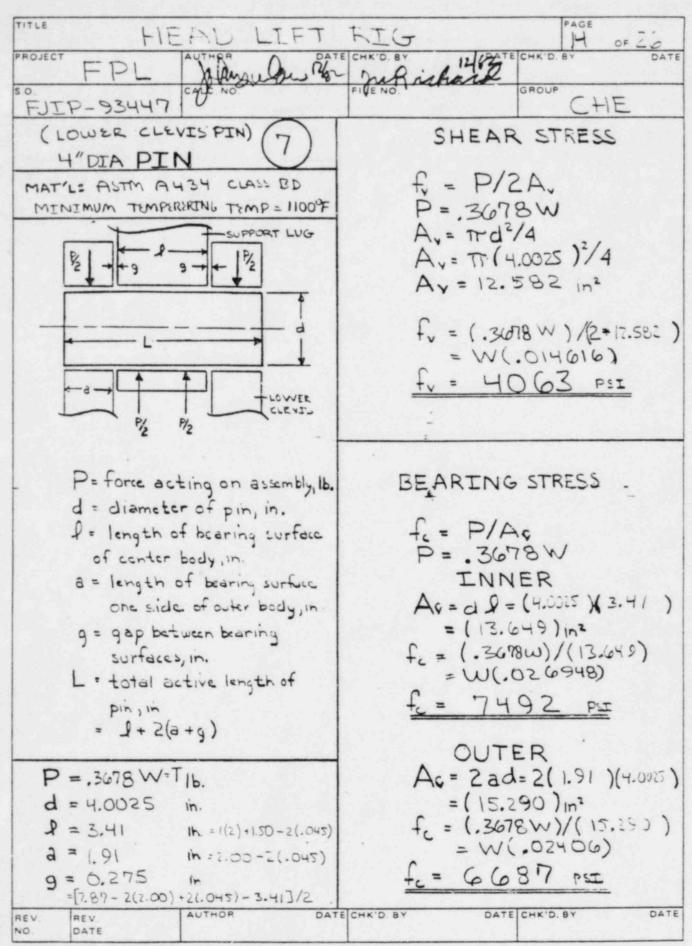
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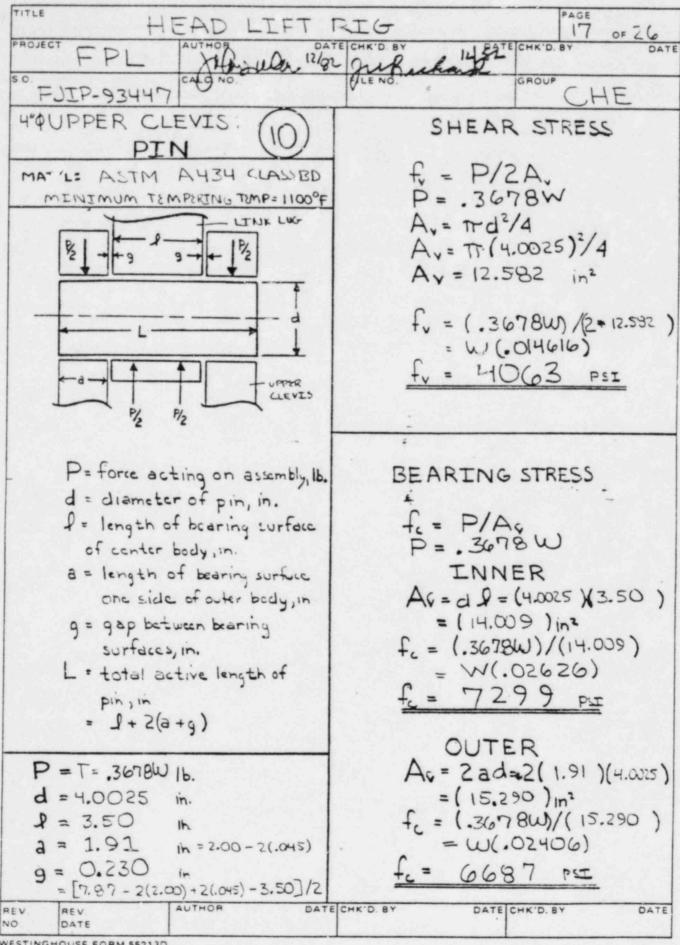
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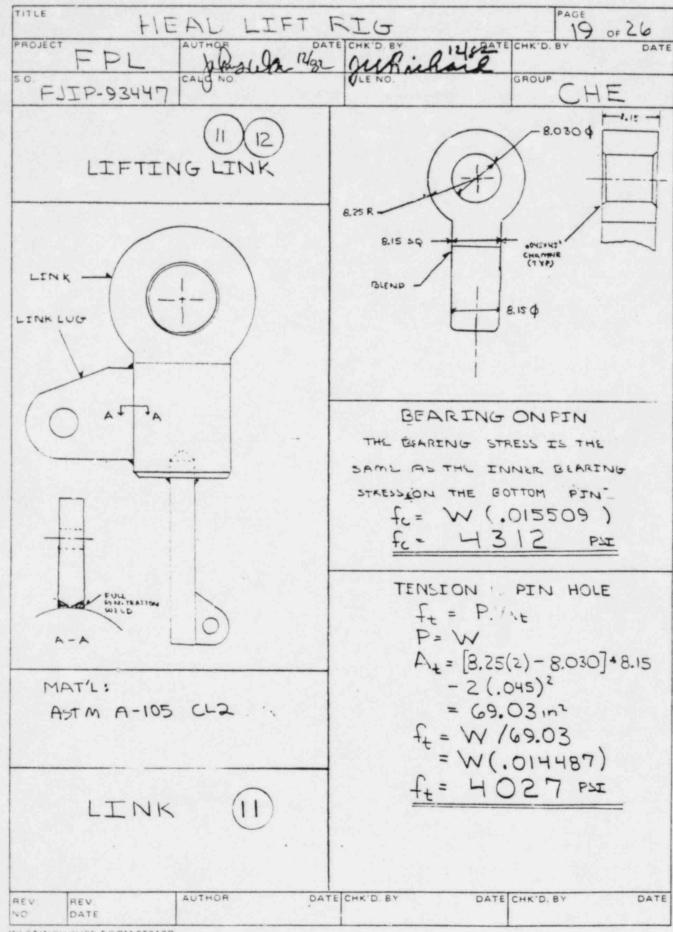


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I = Td 4/64	
C= d/2	
fb = \(\frac{2}{3} \arr + \frac{4}{4} \right) \left(\frac{d}{2} \right) \\ = 16P(\frac{3}{3} \arr + \frac{4}{4} \right) \right) \left(\frac{d}{2} \right) \\ \left(\frac{d}{3} \right) \\ \le	
= 16 (.3678 W)/T/4.00253 *	
$\left(\frac{1.91}{3} + .230 + \frac{3.50}{4}\right)$	
= W(.050881)	
f= 14,145 PI	
(1)	
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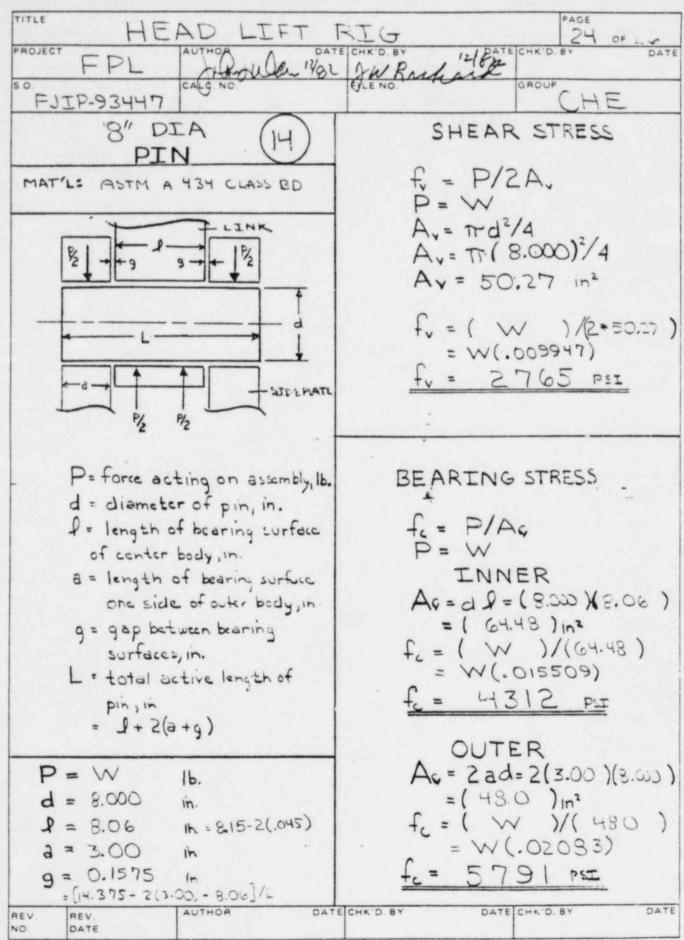
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HEAD LIFT	RIG 20 01 26
FPL Januaring	TE CHK'D BY DATE CHK'D BY DATE
SO. CAYONO	FLE NO. GROUP
FJIP-93447	CHE
SLING ASSEMBLY (1)	
LINK	
LINIX	350
	THK 45°
SHEAR TEAR-OUT & PIN	~ /
fv: P/2Av	X
P= W	17.00
1.8*(\(\frac{\circ_{0.030}}{2}\) = \(\lambda\)	1 ((*) 1
0452	1
= 34.513	E00 R
fy = W/2/34.513	TX/
= W(.014487)	4.02 0
tv= 4027 PSE	
	TENSION @ PIN-HOLE
TENSION & CYLINDRICAL	ft = P/At
SECTION	P=iT=.3678W
$f_t = P/A_t$	At= [5,00(2)-4.02] + 3.50
P=W	= 20.93 in²
At = 14 (8.15)	f. = .3678W/20.93
= 52.168 in2	= W(.017573)
ft = W/52.168	f+= 4885 P>E
= W(.019169)	1 1000
ft = 5329 P>I	SHEAR TEAR-OUT @ PIN-HOLE
#F - 025 9 by	fy = P/2Ay = T/2A = .3608W/(2A,)
	Av= (5.00-4.02)(3.50) = 10.465 12
	fv= .3678W/2/10.465
	= W(.017573)
	fy= 4885 PSI
LINK LUG 12.	
12	
REV REV. AUTHOR DATE	TE CHK'D. BY DATE CHK'D. BY DATE

TITLE HEAD LIFT RIC	
FJIP-93447 CACONO	FILE NO. GROUP CHE
LINK LUG (2) EEARING & PIN-HOLE THE BEARING STRESS ES THE SAME AS THE INNER BEARING STRESS OF THE UPPER CULVES PIN $fC = W(.02626)$ $f = 7299$ SHEAR & THE FULL-PENETRATION WELD $f = F/A_V$ $P = W/3$ $Ay = bh = 17.00(3.50)$ $= 59.5 in^4$ $f_V = W/3/59.5$ $= W(.005602)$ $f_V = 1557$ PSI	$f_b = Mc/I$ $I = bh^3/12 - 3.50 (17.00)^3/12$ $= 1433.0 \text{ m}^4$ $C = 17.00/2 = 8.50 \text{ in}$ $M = (W/3) * (11.06 - 8.15/2)$ $- K (\frac{17.00}{2} - 5.00)$ $= W (2.329)15544W(3.50)$ $= W (1.7840)(8.50)/1433$ $= W (.010582)$ $f_b = 29 42 \text{ PDI}$ $f_{tomained} = f_4 + f_6$ $= 726 + 2942$ $f_{tomained} = 3668 \text{ PDI}$
FROM TENJON AND BENDING	
$f_{t} = P/A_{t}$ $P = K = .15544W$ $A_{t} = bh = 17.00(3.50) = 59.5 m^{2}$ $f_{t} = .15544W/59.5$ $= W(.002612)$ $f_{t} = .726 PSE$	

(

TITLE HEAD LIFT RIC	FAGE 22 OF LE
FPL Jamoula 1402	JW Richard CHKD. BY DATE
FJIP-93447 CALONO.	GROUP CHE
7½" DIA (13)	SHEAR STRESS
MAT'L: ASTM A 434 CLASS BD	f. = P/2A. P=W
T HOOK	A = Td2/4
\$ 1 + 9 9 + 1 PZ	$A_v = Tr(7.495)^2/4$ $A_v = 44.120 in^2$
	f, = (W)/(2+44.120)
	= W(.011333)
PL PLATE	ty = 3151 PSI
2 2	
P = force acting on assembly, lb.	BEARING STRESS
d = diameter of pin, in.	ć 5/A
of center body, in.	f = P/AG P = W
a = length of bearing surface	INNER
one side of outer body, in	A0=al=(7.495 X 8)
g = gap between bearing	$f_c = (W)/(59.96)$
L = total active length of	= $W(.016678)$
pin, in	fc = 4636 PSI
= 1 + 2(a+9)	OUTER
P = W 16.	Ac = 2ad=2(3.00)(7.495)
d = 7.495 in.	= (44.97) _{in} ²
2 = 8 mm Ih	fc = (W)/(44.97) = W(.02224)
a = 3.00 in g = 0.1875 in	fe = 6183 PSE
= [14.375 - 2(3.00) - 8]/2	
REV. REV. AUTHOR DATE	CHK'D. BY DATE CHK'D. BY DATE

TITLE HEAD LIFT R	IG	PAGE 23 of
FPL Xwydr 482	marcheis	CATE CHK'D. BY DATE
FJIP-93447 CALE)NO	FILE NO.	CHE
75" DIA PIN B		
BENDING STRESS (2)		
M= = (= a+ g+ = 1)		
f _b = Mc/I I = πd"/64 c= d/2		
fb = \frac{F}{3} a + g + \frac{1}{4} \left(\frac{1}{2}\left(\frac{1}{10}\right)\left(\frac{1}{2}\left(\frac{1}{10}\right)\right)\left(\frac{1}{10}		
$= 16W(\frac{3.90}{3} + 0.1975 + \frac{9}{4})/\pi/7.46$ $= W(.03856)$	4	
fb = 10,720 PSI		
(1) ADAPTED FROM FACTENING AND JOINING,		
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TITLE HEAL LIFT	RIG	25 of 2.6
FFL Samuelan 125	THE CHK'D. BY 12/800 12/800 12/800 12/800 12/800 12/800 12/800 12/8000	TE CHK'D. BY DATE
FJIP- 93447 CACONO.	FU E NO	CHE
8" DIA PIN (14)		
BÉNDING STRESS (2)		
M= = (= a+ g+ = 1)		
fo = Mc/I I = Td 1/64 c = d/2		
fb = \frac{P}{3} a + g + \frac{1}{2} \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \right) \left(\frac{d}{2}\right) \left(\frac{d}{2}\right) \right) \right(\frac{d}{2}\right) \right) \right) \right(\frac{d}{2}\right) \right) \right(\frac{d}{2}\right) \ri		
$= 16 W \left(\frac{3.00}{3} + .1575 + \frac{8.0k}{4} \right) / \pi / 8.003^{3}$ $= W \left(.03156 \right)$	4	
fb= 8774 BSI		
(1) ADAPTED FROM		
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MACHINE DESIGN, PENTON PUBLISHERS		
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TITLE LIEAD LIFT	RIG	PAGE 26 OF 26
FDL AUTHORITION 1201 FJIP-93447 CALE NO.	waithart	GROUP CHE
PLATE SIDE PLATE 16.50 16.50 16.50 16.50 E.56 & X,50 DIEP C'GOK, CN ONE FATE MAT'L: ASTM A514 OR USST-1 ELARING C 7.515 \$\Phi\$ THE MAXIMUM BEARTING STRUSS IS THE SAME AS THE OUTER BEARING STRUSS ON THE HOOK PIN fc = W(.02224) fc = 6183 PSI	$f_{2} = P/A_{1}$ $P = W/2$ $A_{1} = 16.50 - 3$ $- (8.56 - 3)$ $f_{2} = W/2/3$ $f_{3} = W/2/3$ $f_{4} = W/2/3$ $f_{5} = F/2/3$ $F = W/2/3$	3.030 \$ HOLE 3.030 \$ HOLE 3.030 (3.00) 3.
NO DATE		

APPENDIX B DETAILED STRESS ANALYSIS - REACTOR VESSEL INTERNALS LIFT RIG, LOAD CELL AND LINKAGE

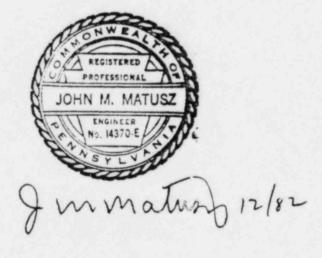
This appendix provides the detailed stress analysis for the Turkey Point Units 3 and 4 reactor vessel internals lift rig, load cell and linkage, in accordance with the requirements of ANSI N14.6. Acceptance criteria used in evaluating the calculated stresses are based on the material properties given in section 5.

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

TALTECNIAL	CLIETIA	uc 010-	4	PAGE 2 of 39
INTERNAL PROJECT FPLIFLA 5.0. FJIP-93447	ALC NO.	OLE NO.	ichard GROW	D. BY DATE
(1) (V	(1) SIDE LUG	1 10	SIDE PLATE 2 UPPER ADAPTORI PIN 3 TOP LUG 8 IREMOVABLEI PIN 7 SLING LEG 12 (LOWER SLING SPREADER 14	
(SUPPORT RING) LEVELING SLEEVE		ADAPTOR I	(SUPPORT RING) OUT	
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FJIP-93447	Turkey Point Units 3 and 4	1 of 39
R. V. Internals Lift Rig, Loa	d Cell & Linkage PDC -	
AUTHOR & DATE J. Urban PURPOSE AND RESULTS:	12/2 J. Matusz & m water	6 12/82
PURPOSE AND RESULTS:		

- 1. The purpose of this analysis is to determine the acceptability of this rig to the requirements of ANSI N14.6.
- 2. The results show that all stresses are within the allowable stresses with the exception of the internals lifting rig engaging screw.

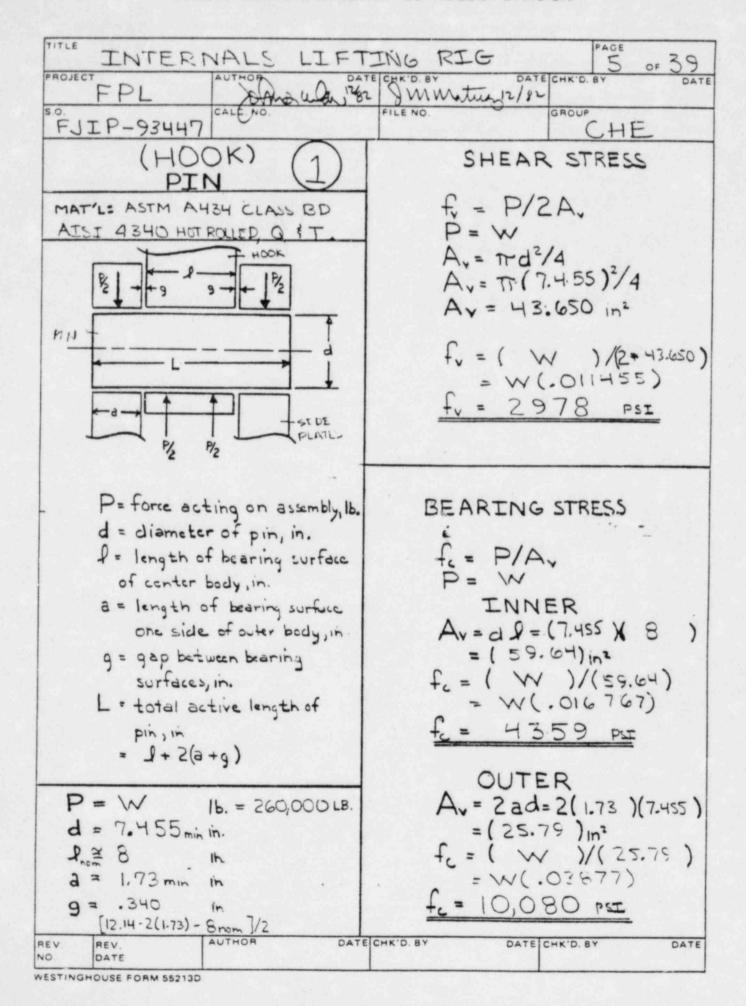


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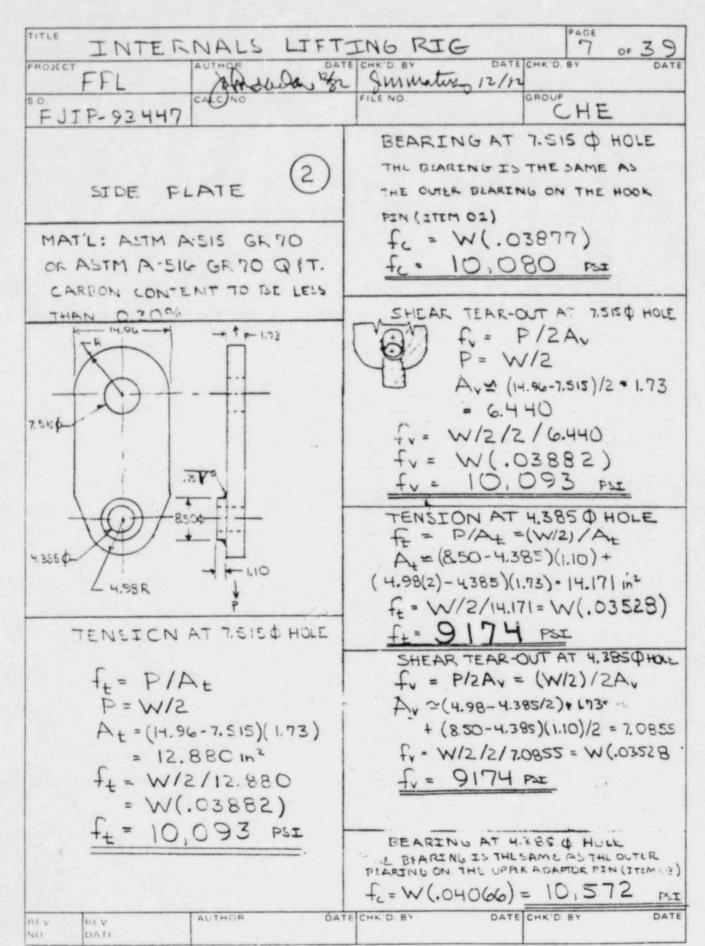
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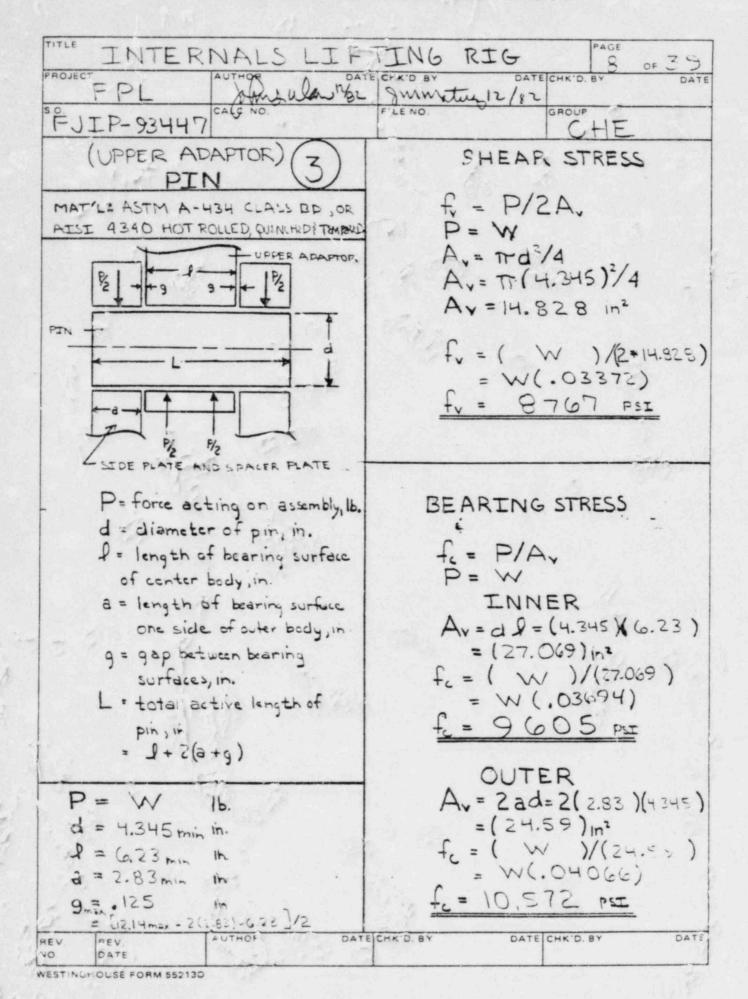
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INTERNALS LIFT	TING RIG	3 of 39
PROJECT AUTHOR DAT	E CHK'D. BY DATE	HK'D. BY DATE
FFL SANDULU 12 82	FILE NO.	ROUP
FJIF-93447	FILE NO.	CHE
DESIGN WEIGHT	NOMENCIA	TURE
LIFT RIG (EST) 13,000 LOWER INTERNALS 236,000 CONTINGENCIES 11,000 CESIGN WEIGHT, W,= 260,000		
SLING LEGY 77.108(KU) (ASSUMING SUPPORT COLUMN INACTIVE)		
TOS Q = W/3 T= W/3/CO 34.774° TIGG CENTERLINE 77.108 77.10		
T= W(.4058) LOWER - JN. / W/3 tand = (WE)	CHK'D BY DATE C	HK'D. BY DATE

TITLE INTERNAL	& LIFTIN	VG RTG	PAGE	0,39
FPL AUTHOR		I preservantary	DATE CHK'D. BY	DATE
FJI P-93447). Fi	LE NO.	CHE	
SLING ASSE	MBLY			
		(HOOK)F:	IN (1)	
3 (UPPER ADAPTOR) PIN —		— Side Pl	ATE 2	
6 (LOWER) ADAPTOR -		TENSION	CELL 5)
(UPPER SLING	91/1		DE) LUG (O	(0)
LEG.) PIN	/ LBLOCK	m.O.	SLING LEG	, (IL
G-(10	OWER SLINGLE	G) MIN (3)		
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INTERNALS LIFTI	NO RIG	PAGE 6 OF	39
FPL CALCINO.	E CHK'D. BY MWatury 12	ATE CHK'D. BY	DAT
FJIP-93447	1.00	CHE	18.
(HOOK) PIN (1)			
BENDENG STRESS (1)			
M= = (= a+g++1)			
fb = Mc/I I = πd"/64 c = d/2			
fb = ξ(3 a + g + 4 l)(2)(πd²)			
$= W(16)(\frac{1.73}{3} + .340 + \frac{4}{4})/(11 + \frac{1.455^{3}}{5})$ $= W(.03585)$ $f_{b} = 9321 \text{ Par}$	•		
(1) ADAPTED FROM FASTENING AND JOINING, 4th Ed, A REFERENCE ISSUE OF . MACHINE DESIGN, PENTON PUBLISHURS PAGE 27			
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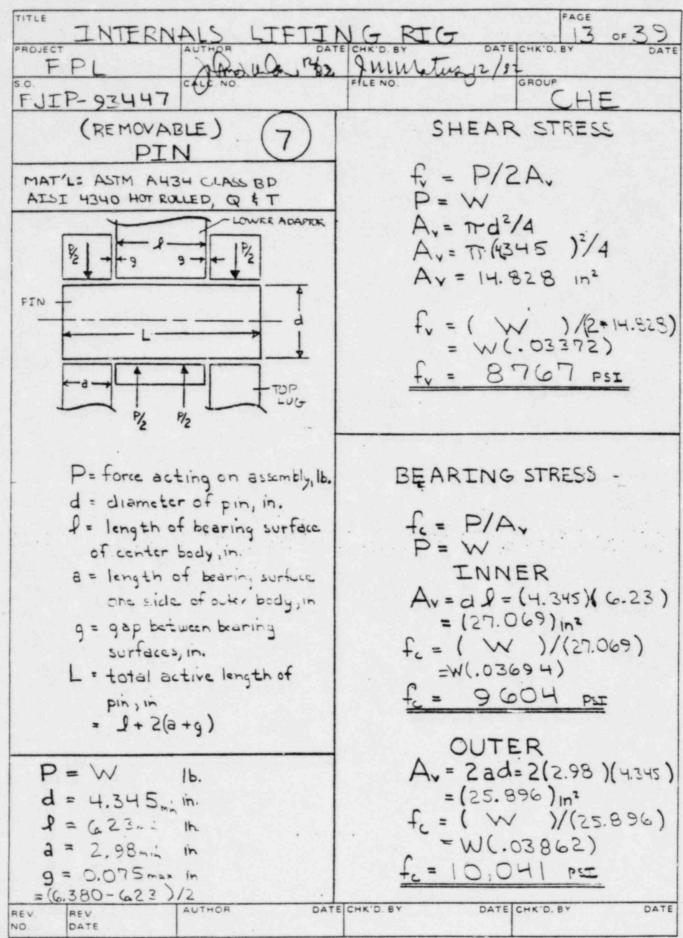


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INTERNALS LIFT	ING RIG	9 of 3 9
FPL CALENO	2 Sm Watur 12/12	GROUP
FJIP-93447		CHE
(UPPER ADAPTOR) PIN 3		
BENDING STRESS (2)		
M= = (= a+g++1)		
f _b = Mc/I I = πd"/64 c = d/2		
fb = \frac{P}{3} a + g + \frac{1}{4} \left(\frac{d}{2} \right) \left(\fra		
$= W(16)(\frac{2.83}{3} + .125 + \frac{6.23}{4})/(\pi * $ $= W(.16303)$	•	
fb= 42,388 PSI		
(1) ADAPTED FROM FASTENING AND JOINING.		
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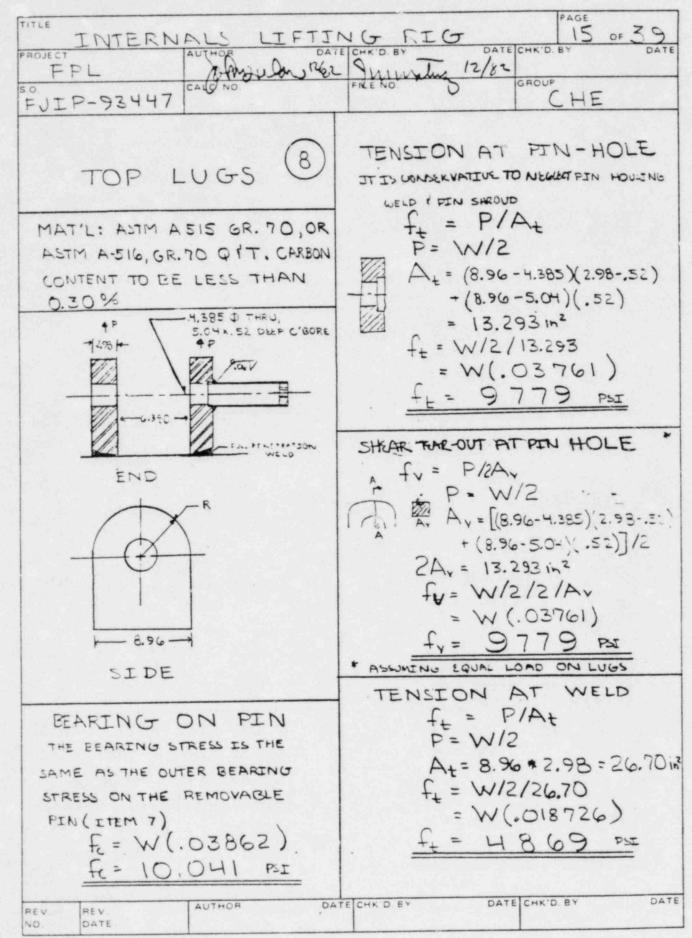
	FILE NO. PAGE 10 OF 39 DATE CHK'D. BY DATE PAGE 10 OF 39 GROUP GROUP CHE
(UPPER AND LOWER) (4) ADAPTOR MAT'L: ASTM A-540 GRADE	BODY AREA A = 2 \ b2 - a2 + \pi b2 \(\frac{360-4x}{1440}\) WHERE
B-24, YIELDmin = 120,000 PMI TENNILE min = 125,000 PMI	$\cos \alpha = a/b \rightarrow \alpha = 44.176^{\circ}$ $A_{8} = 51.933 \text{ in}^{2}$
AA B	h= 4.48 min IN a= 6.23 min IN b= 8.94 \$\Pmin IN (Z) \(Y) N-2B = 44-8N-2B IN - IN K= 3.932 min IN
THORUTE M K THORUTE THO THO THO THO THO THO THO T	D= 4.387 max in D= 4.387 max in d= 4.31 max in P= W= 16 IS C < D?
3	$C = \sqrt{6^2 - 3^2}$ $= \sqrt{(8.94)^2 - (6.23)^2}$ $= 6.412 \text{ in}$ $D = 4.397 \text{ in}$
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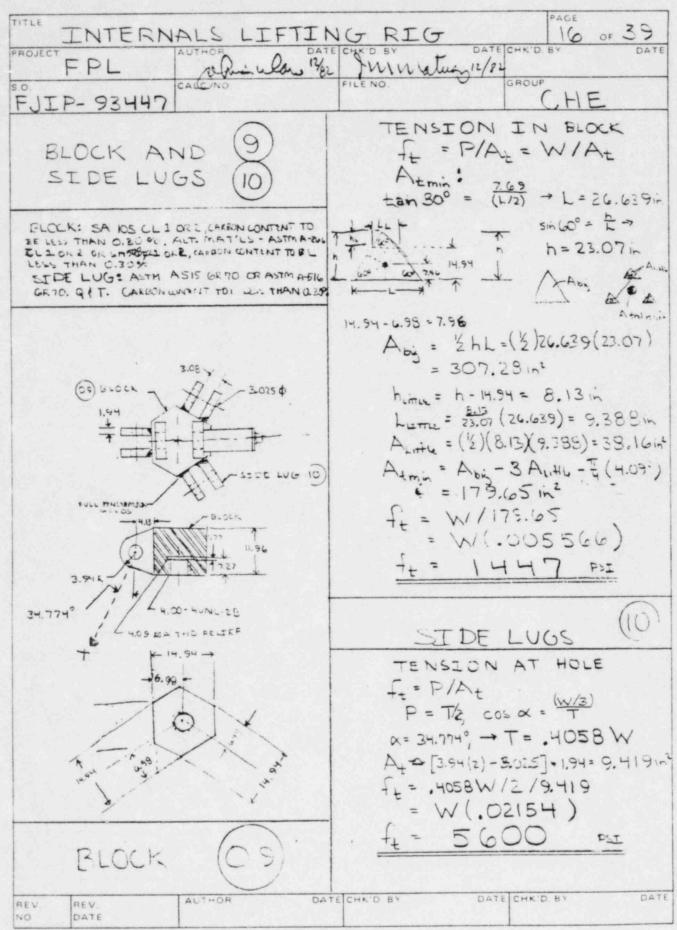
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FPL Jefandieden Mar	FILE NO. GROUP
FJIP-93447 C > D	CHE
	THREAD SHEAR
(UPPER AND LOWER) (4)	f, = P/A, = W/A,
ADAPTOR	Av = 17 DpHch * 1/2
TENSILE STRESS C A-A	= (3.932)-(.39)
f = P/At	= 3.542 in
P=W	Dpitu = 7 - 64952/N
At = AB-Ahole	$= (4.1688) in.$ $A_{V} = (23.194)_{10}^{2}$
Ande Da At = (51.933) - (4.387)(6.23)	fy = W/Ay
= 24.602 in2	= W(.04311)
ft = W/At	fv = 11,209 PSE
= W (.04065)	
#= 10,569 PLE	SHEAR TEAR - OUT
BEARING STRESS & A-A	$f_v = P/2A_v$ $A_v = (h - D/2)a$
fe = P/Ac	= (4.48 - 4.387/2) * (6.23
P=W	= 14.245 m²
Ac= aD	P = W
• (6.23 X 4.387)	fv = W/(2Av)
fc = W/Ac	= W (.035100)
W (.03659)	TV - 9120 F4
fc= 9513 PX	
	* TETTENG AV BE
TENSILE STRESS & B-B	THE AREA OF THE
ft = P/At = W/At	DERECTLY ABOVE THE
$A_t = A_B - \pi d^2/4$ = $(51.933) - \sqrt{4} (4.31)^2$	AXIS
= 37.34 m2	
ft=W(.02678)	
F+= 6963 PIT	TE CHK'D. BY DATE CHK'D. BY DATE
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INTERNALS LI	FTING RIG 12 OF 39 TELCHKID. BY DATE
FPL CAL OF CALOR	PILE NO. DATE CHK'D, BY DATE CHE CHE CHE
TENSION CELL 5	THREAD SHEAR IS THE SAME AS THAT OF THE ADAPTORS
MAT'L- 17-4 PH SS H 1100	(ITEM) 4 AND 5) fy = W(.04311) fy = 11,209 px
↑P	
7 LP 244-8-2A	
TENSION AT THREADS $f_t = P/A_t = W/A_t$ FROM MARKS HANDBOOK SEC 8-13 STRESS AREA: $A_t = 13.3682 \text{ in}^2$ $f_t = W(.07480)$	
ft = 19,448 PXE	TE CHK'D BY DATE CHK'D BY DATE

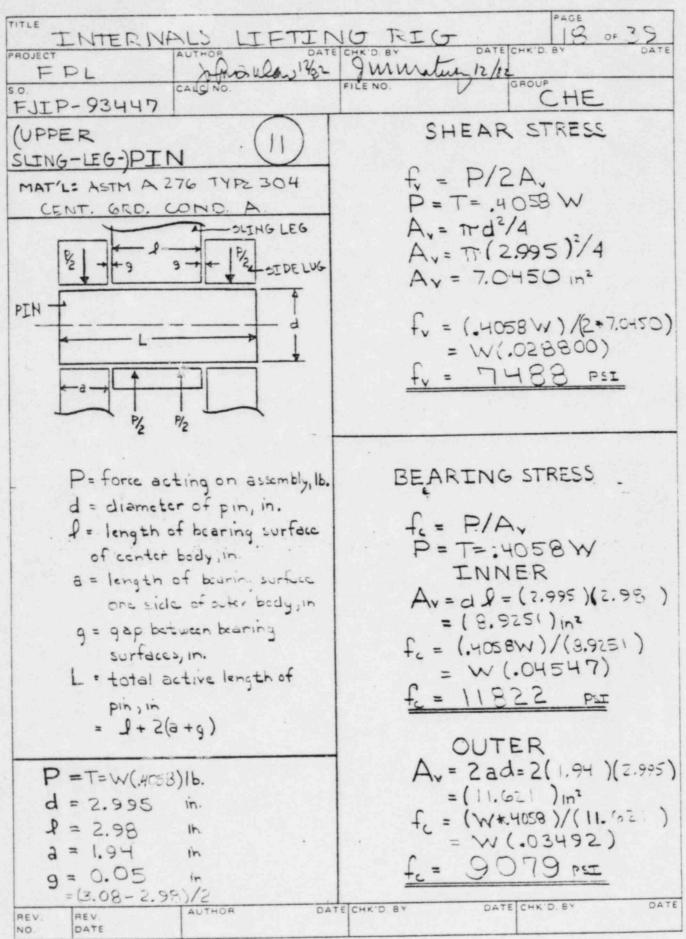


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FJIP-93447	CALE. NO.	F	ILE NO.	tuspiz/s		HE
(REMOVABLE)	\	1				
PIN	. 7					
BENDING	STRESS (2)	1				
M= 2 (3 a+	9+4.0)					
f. = Mc/T						
f= mc/I						
c= d/2						
fb = \frac{2}{2} (\frac{1}{3} a + 9)	+ 4 1)(2) (md2	-)				
$= 16 * W * (2.9)$ $+ 6.23/4)/\pi$ $= W(.163)$	/4.3453					
fb= 42,3	88 PSI					
(1) ADAPTED FRO	M					
FASTENING AND	THE RESERVE TO SERVE THE PARTY OF THE PARTY					
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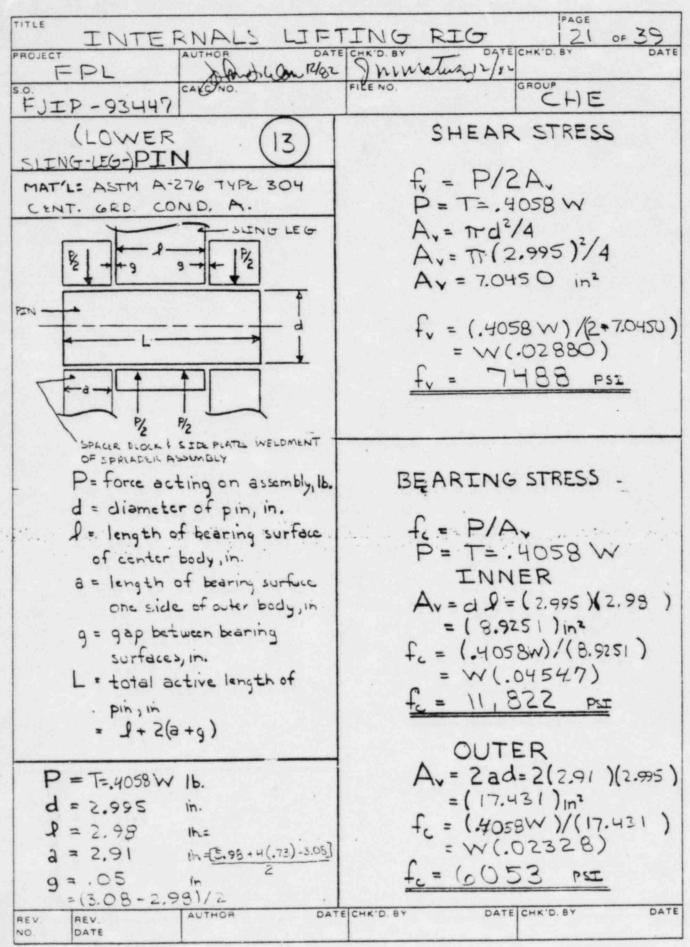


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FPL AUTHOR DA 12/87	Jumatus 12/8	CHK'D. BY DATE
FJIP-93447 CALG NO.	FILE NO.	CHE
SIDE LUG SIDE LUG SHEAR TEAR-OUT AT PIN HOLE $f_v = P/2A_v$ $P = T/2 = .4058 W/2$ $A_v = (3.94 - 3.025/2) \cdot 1.94$ $= 4.709 in^2$ $f_v = .4058 W/2/2/4.709$ $= W(.02154)$ $f_v = 5600$ PSI BEARING ON PIN THE BEARING STRESS IS THE SAME AS THE OUTER BEARING STRESS ON THE UPPER SLING-LEG-PIN (ITEM II) $f_c = W(.03492)$ $f_c = 9079$ PSI SHEAR AT F.P. WELD $f_v = P/A_v$ $P = (W/3)/2$ $A_v = 11.96 * 1.94$ $= 23.202 in^2$ $f_v = W/3/2/23.202$ $f_v = W(.007183)$ $f_v = 1868$ PSI	# T * (sin 3) = T (.012 = W (.0) ft = 12 MAXIMUM BENDING fb = MC/I I = bh3/12 C = h/2 M = \$P\$ F = (W/3)/2 A = 4.13 in fb = (W/3/2) * 4.13 i	F. P. WELD 174°)/2 1.94 = 23.202in² 4.774°)/2/23.202 291)=.4058W(.01229i) 04988) 97 PAI : : : : : : : : : : : : : : : : : : :
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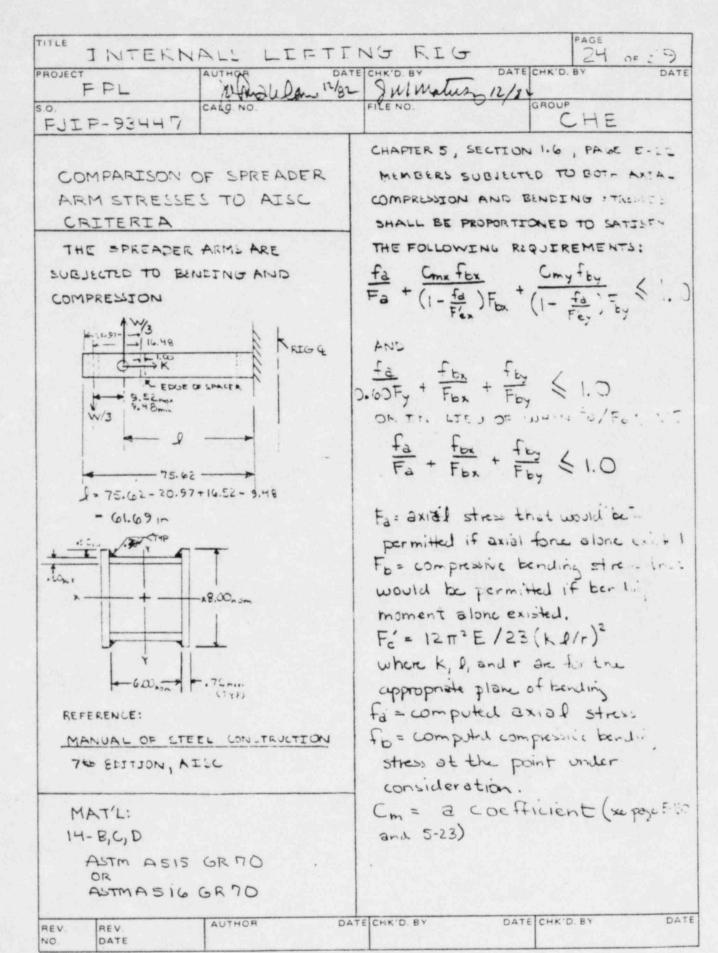
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FJIP-9344	7 CALENO.	FILE NO.	CHE
(UPPER SLI	NG-LEG-) _		
PIN	(1)		
BENDIN	G STRESS (2)		
M= 2 (3 a	+ 9+ + 1)		
fb = Mc/I			
I = π d4/	64		
c= d/2			
C = (+,	q+41)(2)(4+		
+b= 2 (30 +	G+41)/(md2)		
= 16(.405	9+4x //(11a-)	The second second	
11.94 + 05	+ 2.98/4)/(T+2.995)		
= W (.11		•	
fr = 28.	_		
Tb 20,	021.4		
(1)			
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FFL About On 182	MI WATER 12/82 DATE CHK'D BY DATE
S.O. CALC NO.	FILE NO. GROUP
FJIP-93447	
	BEARING ON PINS THE BEARING STRESS IS THE SAME
SLING LEG (2)	AS THE INNER BEARING ON THE
322110 220	SUNG-LEG-PINS (ITEMS II AND 13)
MATL:	£= W(.04547)
ASTM ASIS GROOD OR	fc= 11,822 PSI
ASTM A-516 GR 70, Q 1 T.	
C12 < 0.30%	TENSION IN LEG
1.52 (TVP)	ft = F/At = T/At = .4058W/Az
	At = 2.98 * 5.98 = 17.320 m2
2.96	ft= .4058W/17.820=W(.02277)
3.025 \$ THRU(2	74 3 7 2 0 1 1
	TENSION AT FIN-HOLE
Lai	f. = P/A+=T/A+=.4058W/A+
115 1	A++(5.98-3.5LE)+2.98 = 8.7855-
1 7:37 30	(800HO.) N 6508.8/M 800H. = 2
SECA-A	es 186,11 - ft
SCALE DWG SIGNENG	
OF RESCIRCLE	SHEAR TEAR-OUT AT PIN-HOLE
	fy = P/2Av = T/2Av = 4058 W/2Av Av = (5.98-2.025) + 2.98/2 = 4.403 m
	fy= .4058 N/2/4.403 = W(.04608)
	fy = 11 981 BI
ACO) 2A	
<u>↓ ^ ∏</u>	
T 1	
500	
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5.0. FJIP-93447	CALG. NO.		FILE NO.		GROUP	HE	
(LOWER SLIN	1G-LEG-)	13					
BENDING		(1)					
M= 2 (3 a+	9+40)						
f _b = Mc/I I = πd"/61 c = d/2	4						
fb = = = (3 a + a	+ 4 2)(2)	(πd²)					
$= 16 * W * .405$ $(2.91/3 + .05)$ $(11 * 2.995^{3})$	+ 2.98/4)	/		÷		•	
fb = W(.1	3578)						
fb= 35,3	303 psi						
(1) ADAPTED FRO							
FASTENING AN	the same of the sa	-					
MACHINE DESIGN	, PENTON PUB	LISHERS					
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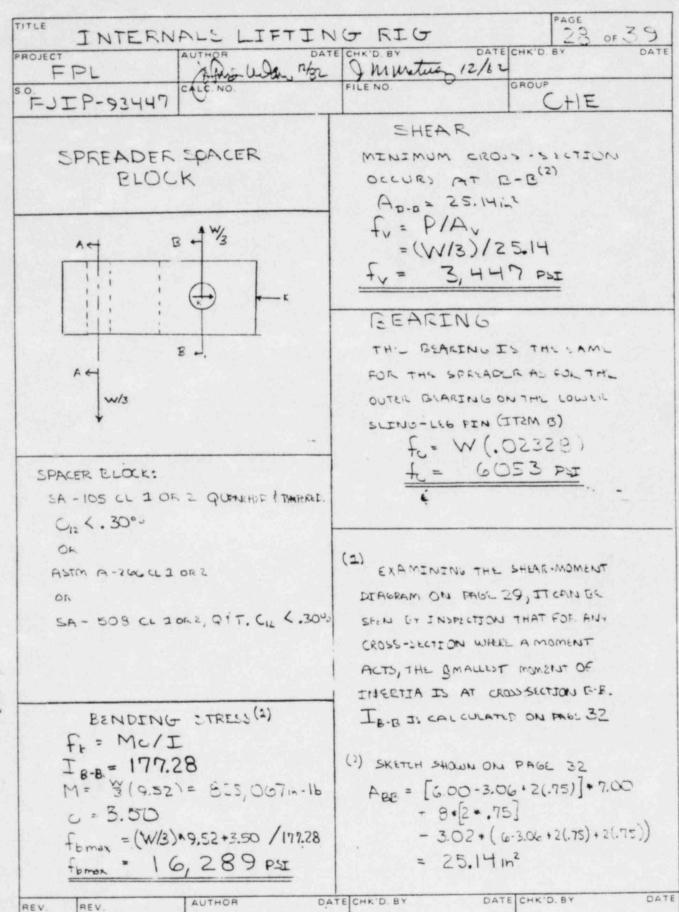
TITLE	TINTERN	.\ . TF	TING RIG	PAGE 23 OF 39
PROJECT		A Proced	DATE CHK'D. BY	DATE CHK'D. BY DATE
FIT	P-93447	CALE NO.	FILE NO.	CHE
		ER ASSE	MBLY (H)	
SPACE	ER BLOCK A	5.48	16.48	B BACKEUP PLATE
-11	- ,73144		E3.08	SIDE PLATE
7.98	- 5.98 - NU	20.97		.50 to .50 to 1
14			3,025 Φ	DLATE.
	.33		DUTER TUBE ASSEMBLY CONNECTED TO SPREADER BY NUT	
		₩/3		
REV.	REV. DATE	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY DATE



FPL AND LIFTI	DATE CHEVE BY
	Willemo. GROUP
FJIP-93447 CAG NO.	FILE NO. GROUP CHE
COMPARISON OF SPREADER	ASSUMING THE VERTICAL TO BE
ARM STRESSES TO AISC	RELATIVELY LONG AND FLEXIBLE, AND SO
CRITERIA	THEREFORE INCAPADLE OF EXERTING AN
	INFLUENTIAL MOMENT, K THEN IS
DETERMINING fa	Kx= 0.80 (Case b)
C 1/4 22-11/4	IN THE HORIZONTAL PLANE, THE
fa = K/At = . 2314W/At	INNUR ENL IS AGAIN CIXED, BUT
At= 2 *.50 * 6.00 +2 *.75 * 8.00	THE OUTER END MAY TRANSLATE AND
+ 4 * .33 * .38/2	ROTATE. THE THEORETICAL K VALUE
At= 18.29 in2	IS 2.0 . THE AISC RECOMMENDED ,
fa = .2314W/18.29 =	UALUE IS
W(.012653)	$K_{y} = 2.1$
fa = 3,290 FSI	DETERMINING RACII & GYRATION
DETERMINING Fa	FOR A RECTANGLE IG = bh 3/12 FOR A TRIANGLE AGE bh 3/36 ALSO, I = IG + Ad 2 bh 3/36
	Ix-x (2(.75 * 9.003/12).
SECTION 1.5.1.3	+ 2 (6.00 + .503/12 + .50(600)(3.25)
CITIKMINING C.	+ 4 (.38 + .38 3/36 + .38 2 (3.5033)
Cc = 12m2 E/Fy	$\perp_{x-x} = 131.30 \text{ in}^4$
= (211, 30'000'000\38'000),5	rx-x = VIx-x/A
= 124.93	= (131.30/18.29)/2
DETERMINING KD/r	$r_{x-x} = 2.679 in$
D = 61.69in	/
DETERMINIONS K's (page 5-138)	Iyy = 2 (.50+6.003/12)
THE INNER END IS FIXE, AND	+ 2 (8.00(.75)/12 +.75(8.00) 3.00-
HELD FROM TRANSLATION BY THE CENTRAL	+ 4 (.38 + .38 / 36 + 23 (3.00 - 3)
COLUMN, RELATIVELY CLOSE TO THE	Iy-y = 157.64in
COLUMN END IS THE LOWER	
SLING LEG PIN, THE SPRINDER	ry-y-VIy-y/A.
MAY ROTATE ACOUT THIS FIN,	= (157.64/18.29)2
BUT THE PIN IS HELD FROM	ry-y = 2.936 in
TRANSLATING BY THE SLING LEG.	
REV. REV. AUTHOR DAT	CHK'D. BY DATE CHK'D. BY DAT

INTERNALS LIFTI	NG RIG 26 OF 39
DJECT AUTHOR DA	2 9 m matura 12/84
JIP-93447 CAG NO.	FILE NO. GROUP CHE
COMPARISON OF	SINCE fby = O ONLY
SPREADER ARM STRESSES	CM MUT BE DETERMENTED. IN
TO AISC CRITERIA	THE VERTICAL PLANE, AS DETERMINED
	IN THE SELECTION OF KA, JOINT
THEREFORE	TRANSLATION IS PREVENTED. THE
(KP/r) = 0.80(61,69)/2.679	BOADING CONDITION THEREFORE
= 18.422	CONFORMS TO CATEGORY B (PAGE 5-17
$(KQ/r)_y = 2.1(61.69)/2.936$	AND M, = M2 (SEL PAGE E-132 FUR
= 44.124	SION CONVENTION) AND WOULD PRODUCE
FOR BOTH AXIS KS/r < CL	REVERSE CURVATURE STOPE POINT
SO EQUATION 5-1.5-1	SO THAT
GOVERNS (page 5-16)	Cm = .6 4 or + w/3
,	. 4, WHICHEVER IS LESS
Fa = (1- 2N2) Fy/(53+38-8)	:. Cm = . 4
WHERE N = (KS/r)/Cc	
Nx = 18.422 /124.83	DETERMINING for
= ,14758	
Ny = 44.124/124.83	fox = Mc/Ix-x
= . 35347	$M = \frac{3}{3}(9.52) = 825,067 \text{ m-1b}$
:. Fa = 21,832, PLI	C= 8.0/2 = 4 in
Fay = 19, 862 PSI	fb, = 825,067 * 4/131.30
Fa = Famin = Fan	fbx = 25, 135 PSI
Fa = 19,862 PSI	
	DETERMINING Fb.
DETERMINING IF fa/Fa IS	Fb = .60 Fy (page 5-17)
EEDW .15	= .60 * 38,000
fa/fa = 3,290/19,862	Fb = 22,800 PM
fa/Fa = .16564 & .15	
	DETERMINING Fex
THE FIRST SIT OF EQUATIONS	Fe' _x = 12π° E/23/(κ g/r) ²
(page 5-22) GOVERN AND Cm	= 12 = 30,0000 /23 / 18.442
MUST BE DETERMINED.	Féx = 455, 199,

INTE	RNALL L	IFTE	NG RIG		27 of	39
PROJECT F-PL	AUTHOR AUTHOR	Day 12/02	MMatus	DATECHK	D. BY	DATE
50. FJIP-9344	7 CACO NO.		FILE NO.	GPO	CHE	
	ON OF SPRE					
$\frac{f_a}{F_a} + \frac{C_{mx} f_1}{1 - \frac{f_a}{F_{ex}}}$	NDJIJON(2) OX Cmyfe OFN + (1 fd)	7_				
BECOMES	(25,135) (3290) 22,80					
= .6098	= 1					
3,290 3,290 + 2	22,800 + 0					
= 1.2'47 ≶	. 1					
REV. REV. NO. DATE	AUTHOR	DAT	CHK'D. BY	DATE CHK	C'D. BY	DAT

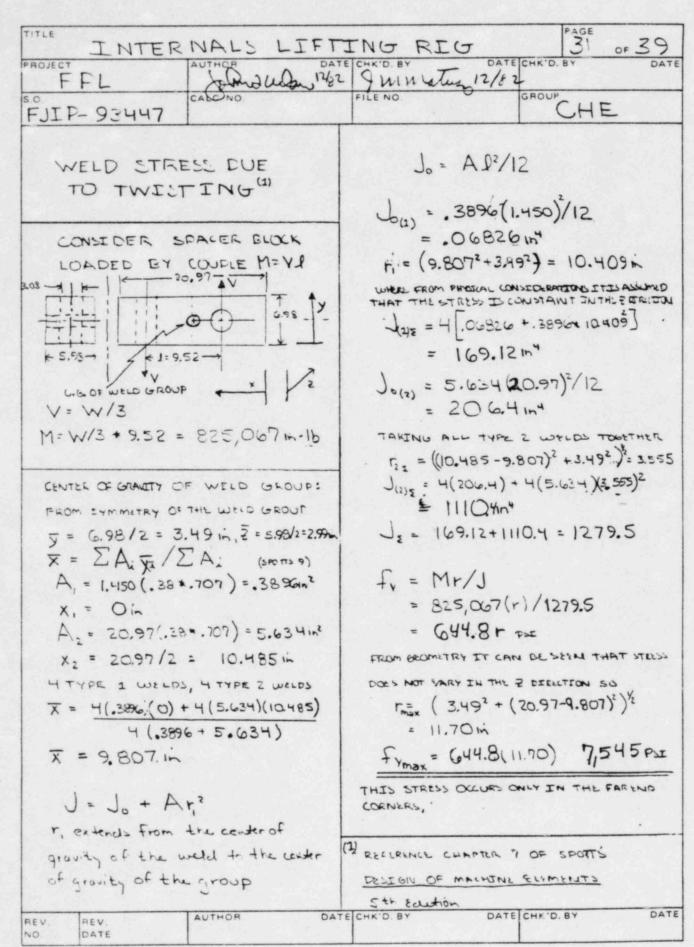


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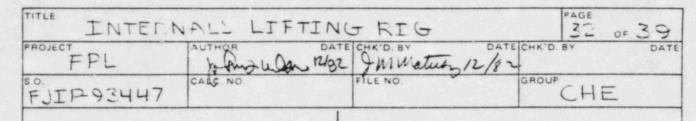
TITLE INTERN	VALS LIFTI	VG RIG	29 of 39
PROJECT	AUTHOR ILLOW POR	177	E CHK'D. BY DATE
FJIP-93447	CALONO.	FILE NO.	CHE
SPREADE	RWELDS	WELD GEO	METRIES
LOADS ON S	SPREADER ARM		
A PB	- C	SPACER BLOCK	TO SIDE
A LE		A CONTRACTOR OF THE PARTY OF TH	-1,450(11P) 6.93 -1,450(11P)
9.52-) m	5.98 -3.03 = 2.90 BACK-UP PLATE 7	
SHEAR DIAGRAM		1 merens	6.98 6.98 10.73
MOMENT DIAGRAM		PLATES TO S	I DE PLATES
₹(9.52) in-1b		6,93	
REV. REV.	AUTHOR .DA	TE CHK'D. BY DAT	E CHK'D. BY DATE

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

INTERN	ALS LIFTI	NG RIG	PAGE 30 of	39
OJECT	AUT OR OLL	14gr Juniatus	DATE CHK'D. BY	DAT
FPL	CAUC. NO.	FILE NO.	IGROUP	
FJIP-93447			CHE	
WELD STRE	ESS DUE			
TO SHEA	R			
CONSIDER S	PACER BLOCK			
	SHEAR FORCE H	<		
	()	• K		
C D/A	- V/A			
1 = 1 = 1	V(uecos) = K/Ava	4(0)		
Ay= 4(.389				
= 24.0		. 00		
	= . 2314 W/20	4.09		
2400.)+				
tr= 543	/ PLI			
	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DA



796 *202



WELD STRELL DUE

THE WELDS ALSO SEC SECONDARY
STRESSES DUE TO SHEAR FLOW (2)

V= external vertical shear on brom

I = moment of mertic of antirexation

to thickness of section at plane

when stress is desired

a = area of xection beyond plane

where stress is desired

y = distance of center of gravity

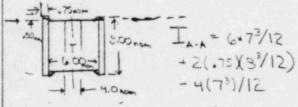
of area to boutral axis of

entire section

AT SEC A-A V= W/3

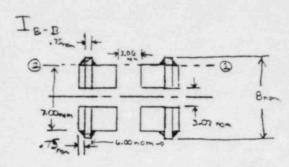
E-E V= W/3

C-C V= 0



IA-A = 121.17 in4

(1) Ref. DE-16N DE UNILLE TE



 $I_{B-G} = [6+2(.75)] 7^{2}/12$ $+2(.75)8^{3}/12$ $-3.06(7.00^{3}/12)$ $-(6+4(.75)3.06)(3.02)^{3}/12$ $= 177.28 \text{ in}^{4}$ AT AXIS-1-1 $t_{i} = \frac{2}{2}(.75) = 1.50 \text{ in}^{4}$ $y = 3.5 + .50/2 = 3.75 \text{ in}^{4}$

fran = (W/3)*.75+375/121.17/1.50

= W(.005158)

1,341par

fran = W/3*.75 * 3.75/177.23/1.50

= W(.002525)

fran = 917 psi

The Horizontal SHEAR FACE IS

F = frt

FA.A = 2012 1b/in

Fran = 13761b/in

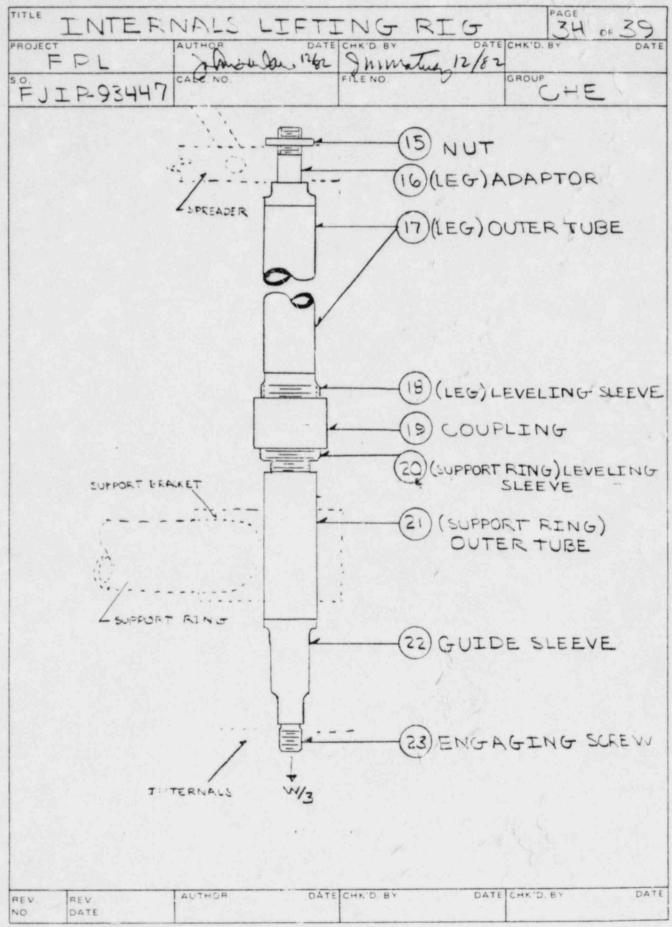
fran = Face (4(.38)(.707)) = 3.745 par

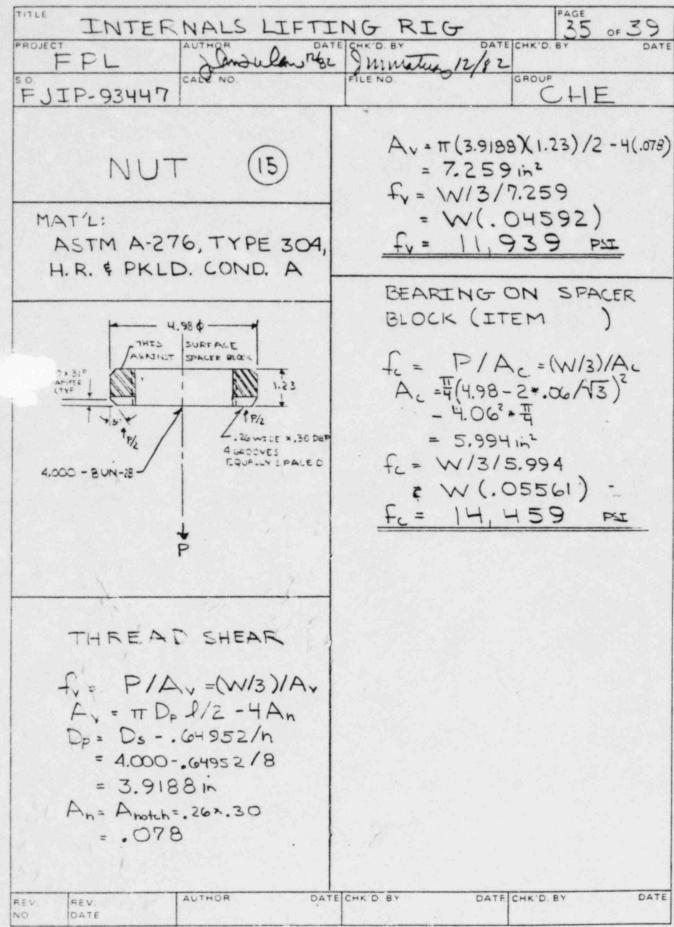
france = Face (4(.38)(.707)) = 1280 f =

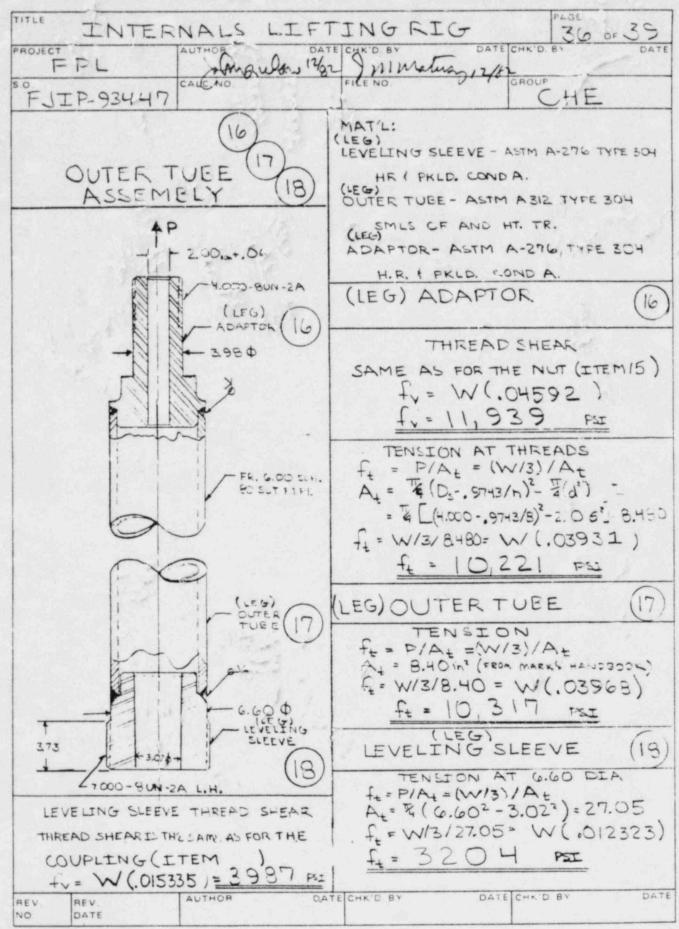
PAGE 2.6-3

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TITLE	TNITER	ENALS LTE	- T T	NITR	T (+	PAGE	or 39
PROJECT	FPL	AUTHOR CALLING	DATEC	HK'G. BY	12 /62	CHK'D. BY	DATE
s.o.FJ	IP-93447	CA (E. NO.	F	LE NO.	7,0/10	CHE	
	IN W	TRESSES ELDS		Frwsiama.	= 11	1,098 2	1
ARI	COUSTOERED	SHEAR STRESSES ADDED ALGORIDA					
		THE SPRIADER	16)				
<u>t</u>	= 7,545 + = 10,0						
	SEC A-A VE = FV (TWIST I + FV (SHIAR F)	Ne) + fr(>m+AR FORCE		•			
t	V(TWISTING = (2.492 + [r = (3.492 + [r = 7.531 V(TWISTING) = 4856 + 2	4856 PS					
fre	= 11,09 SEE SKITCH, P	SPI					
REV.	REV	AUTHOR	DATECH	K'D. BY	DATE	CHK'D. 8Y	DATE







INTERNALS LIF	TING RIG 37 of 39
	TE CHK'D BY DATE CHK'D BY DATE
FJIP-93447	FILE NO. GROUP CHE
1311 33111	0,10
COUPLING (19)	(SUPPORT RING) LEVELING SLEEVE
8.6250 (REF) 7.000 J gun-28 (UH)	MAT'L: ASTM A 276 TYPE 304 HR & PKLD. COND A.
MOST BL CISTAIN D ON EACH SIDE (NOTE FROM ASSEMBLY DWG) 7.000 - BUN-28	5.37 ¢ TH D RELIEF 5.500-12 UN-2A
MAT'L:	THEEAD SHEAR ON 7.000 - 9UN THE.
SMLS OF \$ HT TR.	SAME AS THREAD SHEAR FOR WIND (SEE STO.) $f_v = W(.015355)$ $f_v = 3987$ $f_v = 3987$
THREAD SHEAR	TENLION AT THE RELIEF
fy = P/A, =(W/3)/A,	ft = F/At = (W/3)/At
Av= πDp1/2 Dp= Q64952/n	$A_t = \sqrt{4} (5.37^2 - 3.02^2) = 15.435 \text{ m}^4$ $f_t = \sqrt{3}/15.485 = \sqrt{(.02153)}$
* 7.00064952/8 = 6.919in	f = 5597 RE
Ay = T(6.919)(2.00)/2=21.74	THREAD SHEAR ON 5.500 - 12 UN -14, THE.
fy = W/3/21.74 = W(.015335)	f. = P/Av = (W/3)/Av
fv = 3987 REE	A = TD P/2
TENSION AT THREAD RELIEF F = P/A = (W/3)/A +	Dp=D364952/n=5.5006495-/-2 D=2.482814=2.06in
At = 7 (8.6252-7.062) = 19.279 in2	Av= 17 (5.564952/12)(2.00)/2=17.622ii
ft = W/3/19,779 = W(.017290)	f. = W/3/17.622 : W(,018916)
REV. REV AUTHOR DAT	E CHK'D, BY CATE CHK'D, BY DATE

INTERNALS LIFT	NGRIG 38 or 39
FFL Soloulan 1/82	
FJIP-93447 CAUC NO.	FRENO. DI GROUP CHE
SUFFORT RING) OWIER TUBE (21) AND GUIDE SLEEVE (22)	TENSION AT THD. RELIEF $f_t = P/A_t \cdot (W/3)/A_t$ $A_t = T4(6625^2 - 5.55^2)$ = 10.279 in 2
MAT'L: (SUPPORT RING) OUTER TUBE- ASTM A 312 TYPE 304 SMLS CF ! HT. TR. GUI DE SLEEVE- ASTM A 276 TYPE 304 HE! PKID. CONDA.	$f_t = W/3/10.279$ = $W(.03243)$ $f_t = 8432$ PST
5.000 - 12 UN - 26 6.000 SCH. 140 SST FIFE 5.56 AT THE PROFE	TENSION AT WELD $f_{+} = P/A_{+} = (W/3)/A_{+}$ $A_{+} = AFEA OF PIFE = 13.33 in^{2}$ (FRUM MARKS HANDENJOK)
HANCEOUN) - SUBSE SIEEVE	f. W/3/13.33= W(.02500) f. 6,502 pst GUIDE SLEEVE 22
LESS THAN MEAN OF ELLEN CONTRACT MINE) CUTTER TUBE	EEARING ON ENGAGING SLIFTY $f_{c} = P/A_{c} = (W/3)/A_{c}$ $A_{c} = \frac{\pi}{4} \left[\frac{4.98 - 21.00}{4.98 - 21.00} \right]^{2} - \left[\frac{8.00 - 21.11}{4.98 - 21.00} \right]^{2}$ $= 6.3586 in^{2}$ $f_{c} = W/3/6.3583$ $= W(.05242)$ $f_{c} = 13,629 pst$
THREAD SHEAR THE THREAD SHEAR TO THE SAME ASTHE THREAD SHEAR TO THE SAME ASTHE THREAD SHEAR AT THE S. SOUTH DON THE ISUPPORT AING LEVELING SIGEVE (STEM 20) F. = W(.018916) = 4918 PSI	NOMINAL COMPRESSION BELOW ENGAGING SCREW $f_{L} = F/A_{L} = (W/3)/A_{L}$ $A_{L} = [(5.189015)^{2} - 3.66^{2}]^{4} + 10.50^{2}$ $A_{L} = W/3/10.504 = W(.03173)$ $A_{L} = 8250$ FIE
EV REV. AUTHOR DAT	E CHK D BY DATE CHK D. BY DA

INTERNALS LIFTI	NG RIG 39 0,39
	TE CHK'D. BY DATE CHK'D. BY DATE
FJIP-93447	FRENO. CHE
ENGAGING SCREW 23) MAT'L: ASTM A-276 TYPE 304 HR & PKLD. COND A.	BEARING ON GUIDE SLEEVE THE BEARING STRESSIST-E SAME AS THE BEARING STRESS ON THE GUIDE SLEEVE (ITEM 22) $f_c = W(.05242)$ $f_c = 13,629$ PSI
OG XMSO CHAMFER	TENSION AT MINIMUM SECTION $f_t = P/A_t = (W/3)/A_t$ $A_t = T/4(2.86)^2 = 6.424 in^2$ $f_t = W/3/6.424$ $= W(.05189)$ $f_t = 13, 491$ PSZ
2.86 ¢ 3.48 3.48	TAREAD SHEAR fv = P/Av = (W/3)/Av Av = π Dp 1/2 Dp = Ds64952/n = 3.25064952/4 = 3.0876in 1= 3.48 in Av = π (3.0876) + 3.48/2 = 16.978 in ² fv = W/3/16.878 = W(.019750)
REV REV AUTHOR DAT	$f_v = 5135$ PSI

APPENDIX C DETAILED STRESS ANALYSIS - REACTOR COOLANT PUMP MOTOR LIFT RIG

This appendix provides the detailed stress analysis for the Turkey Point Units 3 and 4 reactor coolant pump motor lift sling, in accordance with the requirements of ANSI N14.6. Acceptance criteria used in evaluating the calculated stresses are based on the material properties given in section 5.

FJIP-93447	Turkey Point Units 3 a	and 4 1 of 20
R. C. Pump Motor Lift Sling	PDC -	ONS NO.
M. F. Hankinson 12/82	J. M. Matusz 8	1 Matur 12/82

- 1. The purpose of this analysis is to determine the acceptability of this sling to the requirements of ANSI N14.6.
- 2. The results show that all stresses are acceptable.



	NHA		
		Original Issue	M. Hankinson
REVISION NO.	DATE	DESCRIPTION	BY

	or Lift Sling			PAGE 2 OF	20
FPL/FLA	M. F. Hankman	/EL 9 M matur	DATE CHK	0.89	DAT
	CALC NO.	FILE NO	GROL		
FJIP-93447				CHE	_
③ "U" BOLTS	3 TURNBU		MASTER		
4	7 - 0 511	NG HOOK	96		
•	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			
	9 SLI	NG HOOK			

TITLE D. C. D No.	ton lift Clina			PAGE	OF	20
PROJECT FPL/FLA		E CHK'D. BY	DATE CHK'C			DATE
FJIP-93447	CALC NO.	FILE NO.	GROU	CHE		

Design weight for R.C.P. Motor Sling

weight of Motor - 66,000 to 72,000 lbs.

weight of sling Assembly - 1,000 lbs.

73,000 lbs.

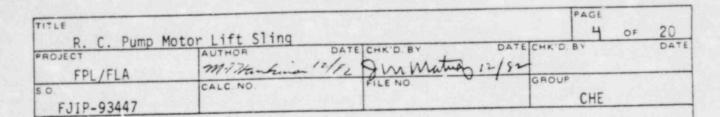
Plus 1070 for contingencies 7,300 lbs.

80,300 lbs.

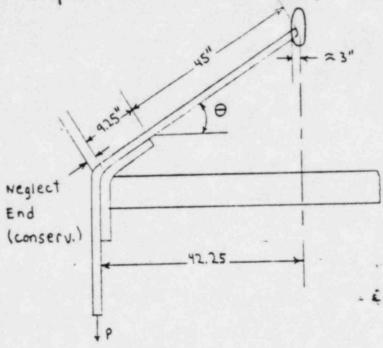
use 81,000 lbs. which is rated lift of sling For design weight.

. .

REV. REV. AUTHOR DATE CHK'D. BY DATE CHK'D. BY DATE



I Establish Tension Value In Tripod Section of wire Rope, Tmax



w = Rated Lift weight of RIG = 81,000 lbs.

with no information to the controry, assume load is approximately centered, i.e.

P = Tension in each vertical rope = 3

= 81,000 lbs. /3 = 27,000 lbs,

 $\theta = \cos^{-1}\left(\frac{42.25-3}{45+9.25}\right) = \cos^{-1}\left(\frac{39.25}{54.25}\right) = 43.66^{\circ}$

Tm = PISIN 0 = 27.000/.6903 = 39.109165.

	1 LD	L 1 - 1 - 1 - 1 - 1 - 1 - 1 - 1 - 1 - 1		DATE CHK'D BY	DATE
REV.	REV.	AUTHOR	DATE CHK'D BY	DATE	4 - 7
NO.	DATE				

R. C. Pump Motor Lift Sling

PROJECT

FPL/FLA

CALC. NO.

PAGE

5 OF 20

DATE CHK'D. BY

DATE CHK'D. BY

DATE CHK'D. BY

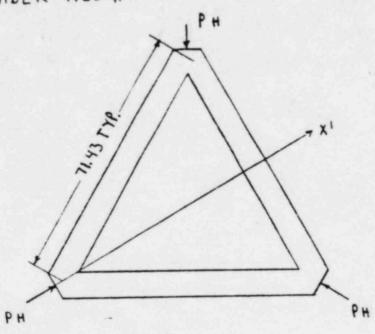
DATE CHK'D. BY

CALC. NO.

FILE NO.

CHE

SPREADER ASSY.



PH = Horizontal Force on Speader (See page 4)

= Tm cos 0 = 39109cos (43.66) = 28 293 16

Let Fc = compressive force in speader

EFX=0: 2(.866)FC-PH=0

 $Fc = \frac{28293}{2(.866)} = 16,335 16$

For 4" SCH 40 PIPE OD = 4.500

wall = .237", 10 = 4.500 - .237 x2 = 4.026"

A = AREA = 1/4 (4.52-4.0262) = 3.174 1n2

REV REV AUTHOR DATE CHK'D. BY DATE CHK'D. BY DATE

The compressive stress is fc

FOR ASTM A106 GRB

Check compressive allowable

From AISC CODE SEC 1.5.1.3

$$Cc = \sqrt{\frac{2\pi^{2}E}{F_{y}}} = \sqrt{\frac{2(3.1416)^{2}(29.1106psi)}{35,000psi}}$$

= 127.89

F = RAD. OF GYR.

$$= \int \frac{I_B}{A} = \int \frac{7.237}{3.174} = 1.510$$

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NO.	DATE					

TITLE D. C. Dump Mo.	tor Lift Sling			PAGE	OF	20
FPL/FLA		2 9 M mater	DATE CHK'D			DATE
FJIP-93447	CALC NO.	FILE NO.	GROUP	CHE		

For Ideal fixed ends, K=.5

This symmetric case probably approaches K=.5

use K=.65 for conservativism

Kl = .65(71.43) = 30.75 < Cc

1.510

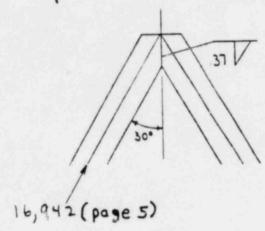
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TLE D. C. D Mod	or lift Sling			PAGE 8	of 20
FPL/FLA	N. F Hanking	PES m mal	tus 12/52	D. BY	DATE
FJIP-93447	CALC NO	FILE NO.		CHE	
6					
Since	KL CC:				
Fq = M	ax Comp. str	255			
=	1- [KB/4/5]	Fy			
	200	×0/c/3			
	5 + 3(k(t)2 - 1) 3 8Cc	863			
	_				
=	1- (30.75)2	Fy			
	5 + 3 (30.75) - 3 8 127.89	(30.75)3			
	3 8 127.89	8 (127,89)3			
	[10289] Fx				
	5 + .0910017				
	3				
= -	[10289] FY	= .553 Fy = .	533 (35 KSi)		
	1.755				
	19.35 KSi				
	11.33 131				
fc	< Fa				

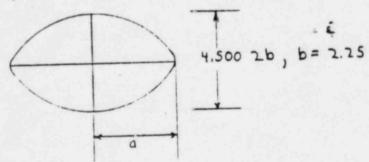
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	1	AUTHOR	DATE CHK'D BY	DATE CHK'D. BY	DATE
REV.	REV				
NO.	DATE				

TITLE D. C. D Mate	- Lift Sling			PAGE	OF	20
R. C. Pump Moto	Marking 1482	61	E CHK'D.	BY		DATE
FJ1P-93447	CALC NO.	FILE NO.	GROUP	CHE		

Analyze compressive stress on Tube to Tube Weld



The length of weld will be length of a half ellipse (projection of 10 of pipe)



$$a = 1 \left[\frac{4.50}{5 \cdot 10.30^{\circ}} \right] = 4.50$$
 $m = a - b = 2.25 = .333$ ath 6.75

From Marks Handbook, 7 TH ED. Pages 2-18 L = Length of perimeter of half-ellipse = 1/2 [T(a+b) k] = 1/2 [T(6.750)(1.029)] = 10.91 in

		1	DATE CHK'D. BY	DATE CHK'D. BY	DATE
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NO	DATE			And the same of the same of the	

R. C. Pump Mot	or Lift Sling		F	AGE	OF	20
PROJECT FPL/FLA	AUTHOR	Sministry 148-	E CHK'D. B	¥		DATE
FJIP-93447	CALC. NO.	FILE NO.	GROUP	CHE		

The throat area of the weld is:

The compressive force on the weld

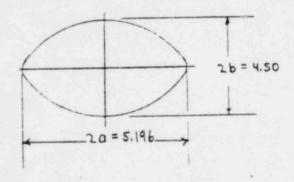
The compressive stress on the weld

Fc = Max allowable stress (AISC, 8th. ED, Table 1.5.3)

= Same as base metal

. fccfc

Check Plate to Tube Weld



				DATE CHE'D BY	DATE
		AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	0
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TITLE				PAGE	20
R. C. Pump Mot	or Lift Sling AUTHOR AT F Harfmar 12/12	gunatura 12/8	ATE CHK'D.	9 1	DATE
FJIP-93447	CALC NO.	FILE NO.	GROUP	CHE	

L= Perimiter of Ellipse

 $= \pi (2.598 + 2.25) K$

= T (4.848)(1.00K)

= 15.246 In

At = (15.246)(.707)(.37) = 3.988 102

Fc = (Page 5) Compressive force on weld

= 29344 lb

fc = Compressive stress on weld

= 2829316 = 7094 psi 3.988 in2

f: 4 .60 (35000 psi) = 21000 psi; ok

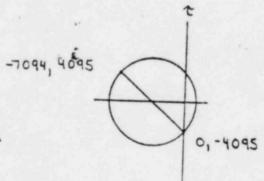
Note that we assumed on page 9 that the tube -totube compression would be taken by the tube - totube weld. Even if this force went through the tube to plate weld the max stress would be:

				DATE CHK'D. BY	DATE
REV.	REV.	AUTHOR	DATE CHK'D. BY	5416	
NO	DATE				

TITLE				PAGE	20
PROJECT FPL/FLA	M. F Hanking (2)	F2 & M. Mutua	12/52		DATE
FJIP-93447	CALC. NO.	FYLE NO.	GROU	CHE	F. (-)

T= Shearing stress

$$\sigma_{1,2} = \frac{0-7094}{2} \pm \sqrt{\frac{1094}{2}^2 + 4095^2}$$



The force trying to slide the U-bolts is the difference between the rope tensions. (No credit for friction) This force will also put the plate in tension.

-		Lauranon		DATE CHK'D, BY	DATE
REV.	REV.	AUTHOR	DATE CHK'D BY	DATE CHA D. D.	04.6
NO.	DATE		18 cm. 22 cm. 75 cm.		

TITLE D. C. Pump Mo	tor Lift Sling		PAGE 13 OF	20
PROJECT FPL/FLA	MIT Hawkinson 4/2	gunatur 12/	DATE CHK'D. BY	DATE
FJIP-93447	CALC NO	FYLE NO.	CHE	

SIDE PLATE

Tensile Strength of Plate - (Assume taken by top boits)

FVB = 39,109 - 27,000 = 12,109 1b.

$$A = 6\frac{1}{2}$$
" \times 1" = 6.50 in²

Fr = Tensile Stress = FuB

A

127,000

= 121091b = 1863 psi
6.50 in²

From AISC Rules, allowable stress for members
with boits holes is .45 ry

(A = .45 x 30,000 = 13,500 psi

(T < (TA :: OK

-		1	DATE CHK'D. BY	DATE	CHK'D. BY	DATE
REV.	REV	AUTHOR	DATE CHA D. B.			
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WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

TITLE P C Pump Mo	tor Lift Sling			PAGE 14	OF	20
PROJECT FPL/FLA	M+ Handener 12/5	S Muratury	12/82			DATE
FJIP-93447	CALC NO	FILE NO.	GROUP	CHE		

The following items are bought from sling suppliers per the referenced drawing. Stress calculations are not performed since material requirements are, for most items, unknown.

The attached Bill of Materials describes these items with the available information and lists design loads together with the catalog and for drawing rated loads.

REV. REV. AUTHOR DATE CHK'D BY DATE CHK'D BY DATE

WESTINGHOUSE FORM 552130

. C. Pump Moto	At Hanking	DATE 14/52	19 m mater 12/8:	CHK'D.	15 of
P-93447	CALC NO		FILE NO.	GROUP	CHE
DESIGN LOAD (POUNDS)		81,000	Each leg. 39,000	000'11	21,000
RATED ⁽¹⁾ LOAD VALUE	(PCUMDS)	1,000		000'01	31,000
CATALOG (8) LO	(6) "u" Bolts - C - 450	(3) 4 x 9" x 16" (5)		2" SIZE (6)	G-227 Jaw & eye (6) turnbuckle 2" Dia. 1hd., 24"
NO. Item DRAWING(2) C	3""" Bolts + Cable Saddle	23"X9" X 18" 1 2"X 11"4" (Total	shackle, etc.) 6x7 Improved PLow I.W.R.C 3 Bridle sling (7'6" sling length	Anchor Safety Shackle	z"x6" Jaw \$ Eye turnbuckle
Item	"u"Bolts	Masterlink		Shackle	Turnbuckle
3.0v	~	0 0		7	60

R. C	. Pu	ump Motor	Lift S	ling	DATE	CHK'D. BY		DATECH	FAG	16 OF	20
FPL/	FLA		M-T Han	hum."	4/12	SM MA	Tung 12,	/52 GA	CH	E	
	DESIGN	0	000'17								
ING	RATED ⁽¹⁾	LOAD VALUE (POUNDS)	38,750								
THE R.C. MOTOR LIFT SLING	TION	CATALOG (7)	TAYCO SLing (4)	A-73, 1" Dia chain with 21"	100						
OF	DESCRIPTION	DRAWING (2)	TAYCO A 13								
FOR NON-DESIGN ITEMS		Ltem	HOOK								
FOR		Son	6 AUTHOR			E CHK'D BY			HK'D. BY		

P C Pump Me	otor Lift Sling			PAGE 17	OF	20
PROJECT FPL/FLA	MI Hanking	1/2 9 m water	12/84			DATE
FJIP-93447	CALC. NO.	FYLE NO.	GROUP	CHE		

NOTES : TABLE !

- (1) See page 2 for identification of item nais
- (2) Description is from Westinghouse drawing AED SK 618 J 644 TXK SUB 5
- (3) Actual purchase order changed link to 23" x 9" x 16.
- (4) S.G. Taylor Chain Co. Inc., Bulletin As-67 alloy steel chain assemblies, attachments.
- (5) Pennsylvania Sling Co.
- (6) Crosby Group, 950 General Catalog, June 1981
- (7) Rated load value: The maximum recommended load that should be exerted on the item. The following terms are also used for the term Rated Load: "SWL", "Safe working Load", "working Load Limit", and the "Resultant Safe. Working Load". All rated load values are for In-line pull with respect to the centerline of the item. Information is from catalogs Identified in (4),(5),(6).
- (8) Information from catalogs may not be identical to that which is INSTALLED. HOWEVER, THIS INFORMATION IS ALL THAT IS AVAILABLE AND IS REPRESENTATIVE.

	acceptance -	TAUTHOR	DATE CHK'D BY	DATE CHK'D. BY	DATE
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NO	DATE				

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

R. C. Pump Mo	otor Lift Sling			or 20
PROJECT FPL/FLA	AUTHOR	FL & MWW	tury 1/54	DATE
FJIP-93447	CALC NO	FILE NO.	CHE	

NOTES: continued

(9) Information from the note 2 drawing states!

The safe working load of this sling assembly is 81,000 lb and a safety factor of 5:1".

REV. REV. AUTHOR DATE CHK'D BY DATE CHK'D BY DATE

TITLE	un lift Sling		19 OF 20
R. C. Pump Moto	IAUTHUR	1 9 minday 482	GROUP DATE
FJIP-93447	CALC NO	FRENO. U	CHE

The rated loads of all items are greater than the design loads. Catalog information on the sling 15 questionable, since the design is a special sling, However, reviewing the PENCO sling catalog (circa 1975) shows that for 3 legs, lift angle of 45°, 6x37, 1x2 Dia. bridle slings, the safe working LOAD is 60,400 pouros. (Latest catalog information (1982) states the S.W.L. is 74,000 pounds.) However, this isling rating maynot be for the special manufactured sling described on the W drawing. The W drawing states that the S. W.L. for the sling is 81,000 lb with a safety factor of 5:1 with a proof test of 47 tons (94,000 pounds) at assembly.

For comparison, the ANSI B 30.9 - 1971 Slings table 8- for 3 leg bridle sling, 6x37 IWRC, 12" DIA.

		Tauruon	DATE CHK'D. BY	DATE CHK'D. BY	DATE
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TITLE D. C. Pump M	otor Lift Sling		PAGE 20	of 20	
PROJECT FPL/FLA	AULHUM	2 gumatia	DATE CHK'D BY	DA	TE
FJ1P-93447	CALC. NO	FRE NO.	CHE		

45 degree angle with mechanical splice rating 15
37 ton (74,000 pound). The proof load (section 9.2.3.1)
15 twice the single leg vertical capacity which 15
(Table 4) 17 tons (34,000 pounds). This table 4 value indicates that the sings are acceptable.

REV REV. AUTHOR DATE CHK'D. BY DATE CHK'D. BY DATE