

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Report No. 50-263/83-02(DPRP)

Docket No. 50-263

License No. DPR-22

Licensee: Northern States Power Company  
414 Nicollet Mall  
Minneapolis, MN 55401

Facility Name: Monticello Nuclear Generating Station

Inspection At: Monticello Site, Monticello, MN

Inspection Conducted: January 2-29, 1983

Inspector: *R. D. Walker for*  
C. H. Brown

2-23-83

Approved By: *R. D. Walker*  
R. D. Walker, Chief  
Reactor Projects Section 2C

2-23-83

Inspection Summary

Inspection on January 2-29, 1983 (Report No. 50-263/83-02(DPRP))

Areas Inspected: Routine, unannounced inspection by resident inspector of operational safety; surveillance; Licensee Event Reports; evaluation of potentially generic issues; and onsite review committee. The inspection involved a total of 59 inspector-hours onsite by 1 NRC inspector, including 14 inspector-hours onsite during off-shifts.

Results: No items of noncompliance or deviations were identified.

## DETAILS

### 1. Persons Contacted

- \*W. A. Shamla, Plant Manager
- \*M. H. Clarity, Plant Superintendent, Engineering and Radiation Protection
- \*H. M. Kendall, Plant Office Manager
- D. D. Antony, Superintendent, Operating Engineering
- W. E. Anderson, Plant Superintendent, Operations and Maintenance
- \*R. L. Scheinost, Superintendent, Quality Engineering
- J. R. Pasch, Superintendent, Security and Services
- F. L. Fey, Superintendent, Radiation Protection
- W. J. Hill, Superintendent, Technical Engineering
- W. W. Albold, Superintendent of Maintenance

The inspector also talked with and interviewed other licensee employees including members of the technical and engineering staffs and reactor and auxiliary operators.

\*Denotes those licensee representatives attending the management interviews.

### 2. Operational Safety Verification

The inspector observed control room operations, reviewed applicable logs and conducted discussions with control room operators during the month of January. The inspector verified the operability of selected emergency systems, reviewed tagout records and verified proper return to service of affected components. Tours of Monticello's reactor building and turbine building were conducted to observe plant equipment conditions, including potential fire hazards, fluid leaks, and excessive vibrations and to verify that maintenance requests had been initiated for equipment in need of maintenance. The inspector by observation and direct interview verified that the physical security plan was being implemented in accordance with the station security plan.

The inspector observed plant housekeeping/cleanliness conditions and verified implementation of radiation protection controls. During the month of January, the inspector walked down the accessible portions of the Standby Liquid Control system to verify operability. The inspector also witnessed portions of the radioactive waste system controls associated with radwaste shipments and barreling.

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established under technical specifications, 10 CFR, and administrative procedures.

No items of noncompliance or deviations were identified in this area.

3. Monthly Surveillance Observation

The inspector observed technical specifications required surveillance testing in the No. 160 MSIV Exercise Test, and verified that testing was performed in accordance with adequate procedures, that test instrumentation was calibrated, that limiting conditions for operation were met, that removal and restoration of the affected components were accomplished, that test results conformed with technical specifications and procedure requirements and were reviewed by personnel other than the individual directing the test, and that any deficiencies identified during the testing were properly reviewed and resolved by appropriate management personnel.

The inspector also witnessed portions of the following test activities:

- a. No. 0015 Main Steam Line Radiation Monitor Functional Test
- b. No. 0048 SRM Rod Block Calibration Test
- c. No. 1158 Diesel Fire Pump Engine Weekly Operability Test
- d. No. 00126/0040a APRM Scram and Rod Block Functional Test

No items of noncompliance or deviations were identified in this area.

4. Licensee Event Reports Followup

Through direct observations, discussions with licensee personnel, and review of records, the following event reports were reviewed to determine that reportability requirements were fulfilled, immediate corrective action was accomplished, and corrective action to prevent recurrence had been accomplished in accordance with technical specifications.

- a. (Closed) 263/83-01: Lapse in Primary Containment

On January 8, 1983, 3 valves in series were found open which constituted a loss of primary containment during 13 operating days. This item is covered in full detail in Inspection Report No. 50-263/83-01(DPRP) and is included here only for continuity.

- b. (Closed) 263/82-15: Abnormal Degradation of RCIC Minimum Flow Line

Two fatigue cracks were found in the carbon steel branch connection weld to the core spray test return line. The cracks were weld-repaired and the dynamic loads are being reviewed. Modifications are to be made as found necessary.

- c. (Closed) 263/82-16: Type B Containment Leakage Test Failure (MO-2030)

Shutdown Cooling Valve MO-2030 was found to be leaking due to the disc being undersized from excessive lapping of the disk to the body seats. Stellite was added to one side of the disc, all surfaces were refinished and the test was completed satisfactorily.

- d. (Closed) 263/82-17: Main Steam Drain, MO-1274, Outboard Isolation Valve Inoperable

The valve failed to close due to a DC motor failure. The motor was replaced and other valves were examined for similar problems. No problems were identified and the preventive maintenance program is to be modified to place these motors on the program. The inspector will followup on the modification to the preventive maintenance program as open item (263/83-02-01(DPRP)).

- e. (Closed) 263/82-18: RCIC Trip Latch Problem

The Reactor Core Isolation Cooling (RCIC) tripped during a fast start due to the overspeed latch being out of adjustment. The trip was readjusted and a modified trip lever is to be installed during the next outage. The inspector will followup on the installation of the modified trip lever as open item (263/83-02-02 (DPRP)).

No items of noncompliance or deviations were identified in this area.

#### 5. Generic Issues

Several generic items were reviewed to determine if they were applicable to the Monticello facility.

- a. HPCI High Steam Flow Set Point

The high steam flow isolation set point was calculated from data obtained during a "fast cold start" test on the High Pressure Coolant Injection (HPCI) system. The Reactor Core Isolation Cooling (RCIC) system isolation set point is also obtained from actual plant data.

- b. Reactor Water Sample Line Valves Isolation Signals

The inspector reviewed procedures, the functional surveillance test, FSAR and electrical logic prints and determined that the sample line valves do isolate on a Group I isolation (all five signals). This conforms with the Technical Specifications. The valves are prevented from reopening when the trip signal clears and the trip logic must be manually reset before the valves can be opened.

- c. HPCI and RCIC Steam Line Drain Valves

The review of control logic for the High Pressure Coolant Injection (HPCI) system and Reactor Core Isolation Cooling (RCIC) system steam line drain valves revealed that the valves were interlocked with the start signal and closed when the steam supply valve opened. The drain valves are not listed as containment valves and do not close (and are not interlocked closed) on an isolation signal.

The drain valves open when the steam supply valve shuts. The inspector discussed this item with the licensee and it is being evaluated.

No items of noncompliance or deviations were identified in this area.

6. Onsite Review Committee

The inspector observed selected portions of the onsite review committee's performance of their required functions during the month of January. The Operations Committee was verified to be conforming with the technical specifications and other regulatory requirements. The Committee was noted to be conforming to the group membership and qualifications. The required meeting frequency and quorum were being met. The Committee's review included proposed technical specifications changes, noncompliance items and corrective actions. The proposed design changes and revisions and procedure changes were reviewed and accepted by the Committee. The tests conducted per 10 CFR 50.59 and other requirements of the technical specifications were reviewed by the Committee.

No items of noncompliance or deviations were identified in this area.

7. Open Items

Open items are matters which have been discussed with the licensee, which will be reviewed further by the inspector, and which involve some action on the part of the NRC or the licensee or both. Open items disclosed during the inspection are discussed in Paragraphs 4.d and 4.e.

8. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) throughout the month and at the conclusion of the inspection on February 2, 1983, and summarized the scope and findings of the inspection activities. The licensee discussed the ongoing program resulting from the loss of containment integrity.