TENNESSEE VALLEY AUTHORITY

DIVISION OF NUCLEAR POWER

BROWNS FERRY NUCLEAR PLANT

MONTHLY OPERATING REPORT

February 1, 1983 - February 28, 1983

DOCKET NUMBERS 50-259, 50-260, AND 50-296

LICENSE NUMBERS DPR-33, DPR-52, AND DPR-68

Submitted by: _____
Plant Superintendent

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Operations Summary

February 1983

The following summary describes the significant operation activities during the reporting period. In support of this summary, a chronological log of significant events is included in this report.

There were 11 reportable occurrences and one revision to previous reportable occurrences reported to the NRC during the month of February.

Unit 1

There was one scram on the unit during the month. On February 5, the reactor scrammed when the turbine tripped (low oil pressure trip) due to oil pressure fluctuations caused by loss of nitrogen on the EHC nitrogen accumulator.

Unit 2

The unit was in its EOC-4 refueling outage the entire month.

Unit 3

There was no scrams on the unit during the month.

Operations Summary (Continued)

February 1983

Fatigue Usage Evaluation

The cumulative usage factors for the reactor vessel are as follows:

Location	Usage Factor		
	Unit 1	Unit 2	Unit 3
Shell at water line	0.00581	0.00455	0.00391
Feedwater nozzle	0.28196	0.19575	0.14871
Closure studs	0.22021	0.16602	0.12705

NOTE: This accumulated monthly information satisfies Technical Specification Section 6.6.A.17.B(3) reporting requirements.

Common System

Approximately 1.32E+06 gallons of waste liquids were discharged containing approximately 3.48E+00 curies of activities.

Operations Summary (Continued)

February 1983

Refueling Information

Unit 1

Unit 1 is scheduled for its fifth refueling beginning on or about April 15, 1983 with a scheduled restart date of August 15, 1983. This refueling will involve loading 8 X 8 R (retrofit) fuel assemblies into the core; finishing the torus modification; turbine inspection; finishing TMI-2 modifications; post-accident sampling facility tie-ins; core spray changeout; and changeout of jet pump hold-down beams.

There are 764 fuel assemblies in the reactor vessel. The spent fuel storage pool presently contains 52 new fuel assemblies, 260 EOC-4 fuel assemblies, 232 EOC-3 fuel assemblies, 156 EOC-2 fuel assemblies, and 168 EOC-1 fuel assemblies. The present capacity is 1,148 locations. Modification work and testing are in progress to increase the spent fuel pool capacity to 3,471 assemblies.

Unit 2

Unit 2 began its fourth refueling on July 30, 1982 with a scheduled restart date of March 12, 1983. This refueling outage will involve completing relief valve modifications, torus modifications, "A" low-pressure turbine inspection, generator inspection, MG set installation for LPCI modification, loading additional 8 X 8 R fuel assemblies into the core, TMI-2 modifications, post-accident sampling facility tie-ins, and changeout of jet pump hold-down beams.

Operations Summary (Continued)

February 1983

Refueling Information

Unit 2 (Continued)

There are 764 fuel assemblies in the reactor vessel. At the end of the month there were 248 EOC-4 fuel assemblies, 353 EOC-3 fuel assemblies, 156 EOC-2 fuel assemblies, and 132 EOC-1 fuel assemblies in the spent fuel storage pool. The present available capacity of the spent fuel pool is 861 locations.

Unit 3

Unit 3 is scheduled for its fifth refueling on or about October 1, 1983, with a scheduled restart date of January 31, 1984. This refueling will involve loading 8 X 8 R (retrofit) assemblies into the core, finishing the torus modifications, post-accident sampling facility tie-in, core spray changeout, finishing TMI-2 modifications, turbine inspection, and changeout of jet pump hold-down beams.

There are 764 fuel assemblies presently in the reactor vessel. There are 280 EOC-4 fuel assemblies, 124 EOC-3 fuel assemblies, 144 EOC-2 fuel assemblies, and 208 EOC-1 fuel assemblies in the spent fuel storage pool. The present available capacity of the spent fuel pool is 993 locations.

Significant Operational Event

Unit 1

Date	Time	Event
2/01	0001	Rolling turbine generator for startup.
	0023	Synchronized generator, commenced power ascension.
	0400	Commenced PCIOMR from 74 percent thermal power.
2/02	0500	Reactor thermal power at 98 percent, maximum flow, rod limited.
	1400	Reactor thermal power at 96 percent, maximum flow, rod limited.
	1444	Commenced reducing thermal power for removal of "A" string high-pressure heaters from service for maintenance.
	1542	"A" string high-pressure heaters out-of-service, reactor power at 79 percent.
	1640	Commenced power ascension, "A" string high-pressure heaters out-of-service for maintenance.
	1710	Reactor thermal power at 84 percent, "A" string high-pressure heaters limited.
2/03	0252	"A" string high-pressure heaters back in service, commenced power ascension.
	0315	Commenced PCIOMR from 85 percent thermal power.
	1235	Reactor thermal power at 99 percent, maximum flow, rod limited.
2/04	0920	Commenced reducing thermal power for removal of "A" string high-pressure heaters from service for maintenance (repair leak).
	1050	"A" string high pressure heaters out-of-service for maintenance, reactor power at 79 percent.
	2120	"A" string high-pressure heaters back in service, commenced power ascension.
2/05	0022	Reactor Scram No. 167 from 93 percent thermal power due to turbine stop valve closure. During weekly turbine checks (OI-47) while performing the overspeed trip test (oil trip) and resetting the turbine overspeed trip system lockout valve, the turbine tripped. Investigation revealed the EHC N_2 accumulator, designed to absorb pressure fluctuations in the oil system, had lost its N_2 charge. During the process of resetting the trip lockout valve, oil pressure fluctuations were sufficient to bring in the low oil pressure trip. Following the scram, relief valve PCV-1-22 failed to fully reseat. The unit will remain down for investigation and repair of PCV-1-22.
	0700	Maintenance complete on EHC system.
	1410	Reactor in cold shutdown for replacement of relief valve PCV-1-22.

Significant Operational Event

Unit 1

Date	Time	Event
		Unit 1 (Continued)
2/06	2145	RHR valve FCV 74-52 tagged for maintenance on motor.
2/07	2225	Maintenance complete on valve FCV 74-52.
2/08	0008	Commenced rod withdrawal.
	0135	Reactor Critical No. 187.
	0330	Received alarm on relief valve temperature instrumentation, PCV-1-22 leaking.
	0350	Commenced inserting control rods, reactor sub-critical, holding due to leak from PCV-1-22.
	1300	Commenced bringing reactor to cold shutdown for drywell entry to inspect PCV-1-22.
	1645	Reactor in cold shutdown for inspection of PCV-1-22. Inspection revealed that relief valve PCV-1-22 would have to be replaced with a relief valve from unit 2, maintenance continues on that valve.
2/09	2126	Changeout of PCV-1-22 complete. Commenced rod withdrawal for startup.
	2215	Reactor Critical No. 188.
2/10	0245	Cycled PCV-1-22, reactor pressure 255 PSI.
	0300	Personnel entered drywell for leak check on PCV-1-22.
	0305	Personnel out of drywell, no leaks found. Increasing reactor pressure to 1000 PSI.
	1253	Reactor pressure at 1005 PSI.
	1312	No leak found on PCV-1-22, decreasing reactor pressure.
	1350	Reactor at rated pressure.
	1837	Rolled turbine generator.
	1920	Synchronized generator, commenced power ascension.
2/11	1730	Commenced PCIOMR from 84 percent thermal power.
2/12	1200	Reactor thermal power at 95 percent, maximum flow, rod limited.
2/13	0015	Commenced reducing thermal power from 91 percent for control rod pattern adjustment.
	0030	Reactor thermal power at 85 percent for control rod pattern adjustment.
	0100	Control rod pattern adjustment complete, commenced power ascension.
	0200	Commenced PCIOMR from 89 percent thermal power.
	1800	Reactor thermal power at 99 percent, maximum flow, rod limited.

Significant Operational Event

Unit 1

Date	Time	Event
		Unit 1 (Continued)
2/14	1200	Reactor thermal power at 98 percent, maximum flow, rod limited.
2/15	0700	Reactor thermal power at 97 percent, maximum flow, rod limited.
2/16	0520	"C" condensate booster pump and "B" reactor feedwater pump tripped when unit preferred voltage began to decrease, reducing reactor thermal power to 70 percent.
	0541	"C" condensate booster pump and "B" reactor feedwater pump back in service, commenced power ascension.
	0637	Reactor thermal power at 97 percent, maximum flow, rod limited.
2/17	1500	Reactor thermal power at 96 percent, maximum flow, rod limited.
2/19	0400	Commenced reducing thermal power for turbine control valve test and SI's.
	0410	Reactor thermal power at 95 percent for turbine control valve test and SI's.
	0415	Turbine control valve test and SI's complete, commenced power ascension.
	0500	Reactor thermal power at 96 percent, maximum flow, rod limited.
2/20	0900	Reactor thermal power at 95 percent, maximum flow, rod limited.
2/21	0700	Reactor thermal power at 94 percent, maximum flow, rod limited.
2/24	0700	Reactor thermal power at 93 percent, maximum flow, rod limited.
2/25	2300	Reactor thermal power at 92 percent, maximum flow, rod limited.
2/28	2300	Reactor thermal power at 92 percent, maximum flow, rod limited.

Significant Operational Event

Unit 2

Date	Time	Event
2/01	0001	End-of-cycle 4, refuel outage continues.
2/28	2400	End-of-cycle 4, refuel outage continues.

Significant Operational Event

Unit 3

Date	Time	Event
2/01	0001	Reactor power at 32 percent holding due to main turbine vibration.
	0230	Commenced rod withdrawal for power ascension.
	0700	Reactor power at 45 percent, holding for investigation of high vibration on No. 8 main turbine bearing.
	1048	Commenced power ascension from 45 percent thermal power.
	2300	Reactor thermal power at 71 percent, computer out-of-service.
2/02	0700	Reactor thermal power at 67 percent, holding computer out-of-service.
	1345	Computer back in service, commenced power ascension.
	1500	Reactor thermal power at 75 percent, holding for inspection of A2 waterbox.
	1530	A2 waterbox in service, reactor power at 75 percent.
	1542	B2 waterbox out-of-service for inspection, reactor power at 75 percent.
	2255	Commenced rod withdrawal for power ascension, B2 waterbox out-of-service.
	2320	B2 waterbox back in service, power ascension in progress.
2/03	0030	Commenced PCIOMR from 88 percent thermal power.
	0830	"B" reactor feedwater pump tripped on low oil pressure when auxiliary oil pump was removed from service, reduced thermal power from 94 percent to 73 percent.
	0840	"B" reactor feedwater pump back in service, commenced power ascension.
	1030	Commenced PCIOMR from 94 percent thermal power.
2/04	1000	Reactor thermal power at >99 percent maximum flow, rod limited.
2/05	0145	Commenced reducing thermal power for turbine control valve test and SI's.
	0200	Reactor thermal power at 99 percent for turbine control valve test and SI's.
	0230	Turbine control valve tests and SI's complete, commenced power ascension.
	0300	Reactor thermal power at >99 percent, maximum flow, rod limited.
	1132	Reduced thermal power when jet pump flow instrument 68-43 pegged, indicated core flow increased to 107 x 10^6 lb/hr. (No change was indicated in actual electricxal or thermal power levels.)

Significant Operational Event

Unit 3

Date	Time	Event
		Unit 3 (Continued)
2/05	1200	Reactor thermal power at 83 percent for repair of jet pump flow instrument 68-43.
	1225	Commenced power ascension, repair of 68-43 in progress.
	1400	Jet pump flow instrumentation 68-43 repaired. Reactor power at 94 percent, power ascension in progress.
	1800	Reactor thermal power at >99 percent, maximum flow, rod limited.
2/11	2340	Commenced reducing thermal power for turbine control valve test and SI's.
	2400	Reactor thermal power at 93 percent for turbine control valve test and SI's.
2/12	0120	Turbine control valve test and SI's complete, commenced power ascension.
	0200	Reactor thermal power at 100 percent, maximum flow.
2/18	2240	Commenced reducing thermal power for control rod pattern adjustment and turbine control valve test and SI's.
	2300	Reactor thermal power at 91 percent for turbine control valve test and SI's and control rod pattern adjustment.
	2325	Turbine control valve test complete, reducing thermal power for control rod pattern adjustment.
2/19	0300	Reactor thermal power at 71 percent, control rod pattern adjustment in progress.
	0330	Control rod pattern adjustment in progress, increasing thermal power.
	1100	Commenced PCIOMR from 78 percent thermal power.
2/20	1500	Reactor thermal power at 100 percent maximum flow.
2/21	0330	Commenced reducing thermal power for SI 4.3.A-2 (CRD exercise).
	0345	Reactor thermal power at 95 percent for CRD exercise.
	0400	CRD exercise complete, commenced power ascension.
	0415	Reactor thermal power at 100 percent, maximum flow.
2/26	0110	Commenced reducing thermal power for turbine control valve test and SI's.
	0130	Reactor thermal power at 83 percent for turbine control valve test and SI's.
	0200	Turbine control valve test and SI's complete, commenced power ascension.
	0300	Reactor thermal power at 100 percent, maximum flow.
2/28	2400	Reactor thermal power at 100 percent, maximum flow.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-259
UNIT Browns Ferry 1
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

MONTH February

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	857	17	1032
2	965	18	1018
3	1016	19	1025
4	943	20	1016
5	2	21	1008
6	-14	22	1005
7	-14	23	998
8	-16	24	995
9	-13	25	985
10	53	26	991
11	748	27	991
12	979	28	963
13	1018	29	
14	1054	30	
15	1048	31	
16	1032		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-260
UNIT Browns Ferry 2
DATE 3/1/83
COMPLETED BY Ted Thom
TELEPHONE 205.729-0834

MONTH February

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-7	17	-6
2	-7	18	-6
3	-8	19	-6
4	-8	20	-7
5	-9	21	-6
6	-9	22	-6
7	-9	23	-6
8	-9	24	-6
9	-9	25	-8
10	-8	26	-10
11	-8	27	-8
12	-8	28	-6
13	-7	29	
14	-8	30	
15	-8	31	
16	-7		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-296
UNIT Browns Ferry 3
DATE 3/1/83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

MONTH February 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	544	17	1069
2	732	18	1055
3	991	19	856
4	1058	20	1032
5	1051	21	1062
6	1058	22	1072
7	1069	23	1070
8	1068	24	1069
9	1068	25	1067
10	1069	26	1069
11	1064	27	1076
12	1064	28	1055
13	1068	29	
14	1065	30	
15	1069	31	
16	1069		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-259
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 1
2. Reporting Period: February 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672	1,416	75,218
12. Number Of Hours Reactor Was Critical	556.37	1,254.92	48,644.46
13. Reactor Reserve Shutdown Hours	2.26	47.71	5,785.02
14. Hours Generator On-Line	532.65	1,212.10	47,612.22
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,593,566	3,660,396	135,433,400
17. Gross Electrical Energy Generated (MWH)	537,460	1,221,250	44,621,970
18. Net Electrical Energy Generated (MWH)	520,584	1,186,031	43,335,810
19. Unit Service Factor	79.3	85.6	63.4
20. Unit Availability Factor	79.3	85.6	63.4
21. Unit Capacity Factor (Using MDC Net)	72.7	78.6	54.1
22. Unit Capacity Factor (Using DER Net)	72.7	78.6	54.1
23. Unit Forced Outage Rate	20.7	14.4	24.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

DOCKET NO. 50-260
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 2
2. Reporting Period: February 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672	1416	70,159
12. Number Of Hours Reactor Was Critical	0	0	43,293.47
13. Reactor Reserve Shutdown Hours	0	0	13,684.82
14. Hours Generator On-Line	0	0	41,975.45
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	120,480,340
17. Gross Electrical Energy Generated (MWH)	0	0	40,024,908
18. Net Electrical Energy Generated (MWH)	0	0	38,873,075
19. Unit Service Factor	0	0	59.8
20. Unit Availability Factor	0	0	59.8
21. Unit Capacity Factor (Using MDC Net)	0	0	52.0
22. Unit Capacity Factor (Using DER Net)	0	0	52.0
23. Unit Forced Outage Rate	0	0	27.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 3/12/83
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

DOCKET NO. 50-296
DATE 3-1-83
COMPLETED BY T. Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 3
2. Reporting Period: February 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672	1,416	52,584
12. Number Of Hours Reactor Was Critical	672	972.50	38,584.58
13. Reactor Reserve Shutdown Hours	0	443.70	3,815.85
14. Hours Generator On-Line	672	946.43	37,720.49
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2,163,518	2,962,356	112,298,460
17. Gross Electrical Energy Generated (MWH)	705,200	985,000	37,024,790
18. Net Electrical Energy Generated (MWH)	687,900	954,969	35,935,873
19. Unit Service Factor	100	66.8	71.7
20. Unit Availability Factor	100	66.8	71.7
21. Unit Capacity Factor (Using MDC Net)	96.1	63.3	64.2
22. Unit Capacity Factor (Using DER Net)	96.1	63.3	64.2
23. Unit Forced Outage Rate	0	33.2	17.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February

DOCKET NO. 50-259
UNIT NAME Browns Ferry 1
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
258	2/1/83	F	0.38	B					Turbine manually tripped for maintenance on bus duct damper fan. No reactor scram.
259	2/4/83	F		B					Derated for maintenance on "A" string high-pressure heater.
260	2/5/83	F	138.97	B	3				Reactor scram due to turbine stop valve closure during weekly turbine overspeed test. Unit remained down for maintenance on PCV-1-22 and RHR valve 1-74-52 and EHC system.
261	2/16/83	F		H					Derated due to "C" condensate booster pump and "B" reactor feedwater pump trip when unit preferred voltage began to decrease.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February

DOCKET NO. 50-260
UNIT NAME Browns Ferry 2
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
243	2/1/83	S	672	H	2				EOC-4 Refuel Outage Continues

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February

DOCKET NO. 50-296
UNIT NAME Browns Ferry 3
DATE 3-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
124	2/18/83	S		B					Derated for turbine control valve test and SI's and control rod pattern adjustment.

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

[4] Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit I - Same Source

(9/77)

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of February 19 83

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
2-9	Primary Containment	drywell door interlocks - personnel airlocks	replace interlocks	none	faulty interlocks	interlocks nonfunctional	replaced interlocks MR# 61784
2-18	EHC	nitrogen accumulators	low nitrogen pressure	none	low pressure	unit scram	recharge accumulato MR# 61962

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of February 19 83

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
2- 9	CRD	HCU Accumulator 38-23	rebuild accumulator	none	faulty o'rings	O'rings & accumulator nonfunctional	replaced o'rings MR# 61905
2-24	CRD	HCU 14-35	replace strainer screens	none	filters worn	manifold block filters nonfunctional	changed out all 3 manifold filters wit new ones MR# 62543
2-16	HPCI	FCV-73-35	replace key in keyway	none	faulty key in keyway	manual engagement lever not operable	replaced key in keyway MR# 61121
2-11	EECW	FCV-67-50	change strainer & adjust bleed-off valve	none	faulty strainer & bleedoff valve out of adjustment	valves stay open	changed strainer & adjusted bleedoff valve MR# 59083
2-11	Chem. Feed	HCV-70-503	repair valve	none	stem needs replacing	valve inoperable	removed stem from 3/4" vent valve off of chem-feed tank & placed in HCV-70-503 MR# 28328

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of February 19 83

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
1-30	RHR	FCV-74-12	replace set screw in gear	none	faulty set screw	valve inoperable	replaced set screw MR# 64451
2-16	LPCI	LPCI MG set 3EN	lubricant leak past seal ring in coupling	none	faulty seal rings	LPCI 3EN inoperable	changed seal rings MR# 28329
2-11	D/G	DG 3D air compressor B	replace head gasket	none	blown head gasket	air compressor inoperable	replaced head gasket MR# 64335
2-10	Fire Protection	fire pump B	replace shear pin	none	faulty shear pin	pump strainer will not rotate	replaced shear pin MR# 63797
2- 4	Fire Protection	RHR-EECW tunnel 3A	repair penetration with RTV	none	unknown	air leak	sealed air leak with RTV. MR# 68404
2- 2	RHR	FCV-74-52	stem leaking	none	stem has scarred places	water leak	tightened packing lub stem & stroked valve MR# 63751
1-31	HPCI	gland seal condenser	replace seal on condenser heat exchanger top gasket	none	blown seal	bad leak on H_2O side	replaced seal MR# 62227

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of February 19 83

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
2-12	Radwaste	low level radwaste cask BS-33-180	insert needs replacing	none	defected insert	insert not functioning properly	installed insert #2 by welding MR# 59168 MR# 60803

BROWNS FERRY NUCLEAR PLANT UNIT 1 and Common

BF EMSIL 30
Appendix B
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CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
2/2/83 through 2/15/83	Fire Protection	Smoke detectors. XS-39-72 BH XS-39-72 AG XS-39-66 XC XS-39-69 WE	During the performance of SI4.11.C.1&5 the listed smoke detectors sensitivity was found to be too low.	None	Normal aging.	The listed smoke detectors sensitivity was too low.	The smoke detectors were replaced and SI4.11.C.1&5 was successfully performed on each detector. MR #A-061453 MR #A-061400 MR #A-061460 MR #A-061466
2/5/83	RHR	Limit switch (LS12) on 1-FCV-74-48	While unit 1 was in cold shutdown with B RHR pump lined up to provide shutdown cooling, the B RHR pump failed to start.	None, All RHR pumps were still operable and primary containment isolation valves 74-47 and 74-48 would have gone closed on PCIS signal.	Water in the limitorque control box on FCV-74-48 corroded limit train contacts.	To go into shutdown cooling the RHR pumps take suction from the A recirculation loop through FCV's 74-47 and 74-48. These valves have an interlock that trips the pump if the valves do not show full open. When these valves were opened a interlock prevented B RHR pump from starting.	Replaced limit trains 1 and 2, 3 and 4, and the torque switch on FCV-74-48 and performed EMI 18. MR #A-061479

CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 1983

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
2/10/83	Neutron Monitoring	IRM "F" detector position relay.	IRM channel "F" would not travel.	None	Bad relay (7C-K9K) coil.	IRM "F" channel rods blocked.	Replaced the bad relay coil, IRM channel returned to normal. MR #A-064457
2/16/83	48 volt DC Power	48 volt DC annunciator battery charger A.	Battery charger inoperable.	None	Bad contactor closing coil.	Battery charger inoperable.	Replaced the bad contactor closing coil, charger operated properly. MR #A-063551

BROWNS FERRY NUCLEAR PLANT UNIT 2

BF EMSIL 30
Appendix B
9/29/82

CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 1983

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
1/31/83	CRD	Scram accumulator level switch in CRD hydraulic control unit 26-47.	Received a false water alarm.	None, unit in refueling outage	Bad level switch	Received several false water alarms.	Replaced the bad level switch. MR #A-061005
2/7/83	RWC and 4KV Shutdown Boards and Busses	Relay 16A-K59A and relay 52RU1B	Relay coil retainer (lexan) spool inspection.	None, relays were operable.	Failure of lexan coil retainer spools. Ref. HFA Relay Coils Service Advice PSM-721-152.2 FSR 366E8138.	Relay coil retainer spools cracked. NOTE: The relays were found to have cracked coil spools during inspection, relays were still operable.	The listed relays were replaced per SEMI 37 and retruned to service. MR #059078 MR #059079
2/9/83	CRD	Hand switch HS-85-48	Hand switch traveled past desired setting.	None	Broken stop plate.	Required care to be taken by operator to avoid turning hand switch past desired setting.	The broken stop plate was replaced in HS-85-48. The stop plate was broken and replaced again on 2/28/83. MR #A-061076 MR #A-063719

BROWNS FERRY NUCLEAR PLANT UNIT 2

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CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 1983

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
2/9/83	RBCCW	2B1 drywell blower motor feeder breaker.	Fan would not start.	None	Bad overload.	2B1 drywell blower inoperable.	Replaced bad overloads per EMI 7. MR #A-064266
2/ 0/83	Annunciator	Annunciator panel 2-XS-55-3B.	Panel failed to test properly.	None	Bad annunciator card.	2-XS-55-3B inoperable	Replaced the bad card, the annunciator panel operated properly. MR #A-061122
2/10/83	CRD	Rod select relay 3AK-32.	Could not select CRD 22-43 during withdraw cycle.	None	Relay 3AK-32 contacts were not making connection. Insulation was found under #7 terminal.	Operator could not select CRD 22-43.	Replaced the relay, the new relay operated properly. MR #061123
2/11/83	Fuel Pool Cooling	2-FCV-78-66 starter coil.	FCV-78-66 would not operate, breaker control circuit fuse blowing.	None	Bad starter coil.	FCV-78-66 inoperable.	Replaced the bad coil, valve operated properly. MR #A-061155

BROWNS FERRY NUCLEAR PLANT UNIT 2

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CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
2/13/83	Radiation Montioring	Radiation monitor 2-RM-90-250.	Monitor tripping electrically.	None	Bad motor.	2-RM-90-250 inoperable.	Replaced the bad motor, the new motor operated properly. MR #A-062699
2/17/83	RWCU	2-FCV-69-2	FCV-69-2 would not operate electrically.	None	Bad valve motor.	2-FCV-69-2 electrically inoperable.	Replaced the bad motor and performed EMI 18. MR #A-063606

BROWNS FERRY NUCLEAR PLANT UNIT 3

BF EMSIL 30
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CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 1983

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
2/-/83	Fire Protection	Smoke detector XS-39-88XA.	Received intermittent alarms from XS-39-88XA.	None	Bad smoke detector.	Received false alarms.	Replaced the smoke detector and performed SI4.11.C.1&5. MR #A-064330
2/5/83	RBCCW	RBCCW surge tank low level switch.	Surge tank low level annunciator inoperable.	None	Broken low level switch.	Surge tank low level switch inoperable.	Replaced the broken switch. MR #A-063959

BROWNS FERRY NUCLEAR PLANT UNIT 1 and Common

NON-CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Cause of Malfunction or Maintenance Action	Corrective Action
1/31/83	Generator Breaker	"A" air compressor check valve.	4th stage air pressure gauge shows pressure with compressor off.	Bad check valve.	Cleaned moisture separators and replaced the bad check valve. MR #A-061352
2/2/83	500KV Switchyard	PCB 5278 air system.	C phase losing air.	Air leak on #2 head. Rag stuck in discharge piston port.	Repaired air leak. Breaker air system returned to normal. MR #A-061476
2/4/83	Station Drainage	Condensate sludge pump A.	Excessive motor noise.	Bad motor bearing and end bell housing.	Replaced the pump motor. MR #A-064304
2/6/83	Off-Gas	SJAE "A" inlet valve.	No indicating lights for valve.	Bad limit switch due to water.	Replaced the limit switch, valve indicating lights operated properly. MR #A-064414
2/6/83	480 volt Turbine Bldg. Vent Boards	Normal feeder breaker for 1B 480V Turbine Bldg. vent board.	1B 480V Turbine Bldg. vent board normal feeder breaker located in 1B 480V unit Bd. compt. 6B tripping.	Bad overcurrent trip device.	Replaced the normal feeder breaker. MR #A-064360

BROWNS FERRY NUCLEAR PLANT UNIT 1 and Common

NON-CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Cause of Malfunction or Maintenance Action	Corrective Action
2/8/83	500KV Switchyard	PCB 5204 air system solenoid valve.	Air escaping from control cabinet.	Bad solenoid valve.	Replaced the bad valve, air system returned to normal. MR #A-061488
2/8/83	Generator Breaker	PCB 214 "A" compressor unloader valve.	Unloader valve inoperable.	Unloader valve stopped up.	Cleaned unloader valve and replaced piston in unloader cylinder. MR #A-059126
2/9/83	Building Heating	Secondary heat pump 1B motor starter coil.	Heat pump 1B inoperable.	Burned coil on heat pump motor starter.	Replaced starter coil, returned heat pump to service. MR #A-061493
2/10/83	Door Interlocks	Door interlock system, turbine to units 1 and 2 reactor building doors (221, 235, and 236).	During refueling outage on unit 2 and startup of unit 1, the turbine to units 1 and 2 reactor building doors (221, 235, and 236) were opened simultaneously breaching secondary containment. Approximately 22 hours later the personnel access doors (235 and 221) between the turbine bldg. and the unit 1 reactor bldg. were again opened at the same time.	Both events were apparently a result of the reactor building access doors closing enough to actuate the door position limit switches, but not closing enough to engage the electric strike.	A door watch was posted until the limit switches were adjusted and the doors returned to service. Upon re-occurrence the door watch was posted and the door interlock limit switches were adjusted to operate properly. MR #A-064424 LER#BFRO-50-259/83008

BROWNS FERRY NUCLEAR PLANT UNIT 2

NON-CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Cause of Malfunction or Maintenance Action	Corrective Action
2/1/83	Main Steam	2-FCV-1-155	Valve torque switch not operating properly.	Bad torque switch.	Replaced the torque switch, valve operated properly. MR #A-061553
/4/83	Heater Drains and Vents	2-FCV-6-133	2-FCV-6-133 inoperable.	Burned valve motor windings.	Replaced the valve motor, FCV operated properly. MR #A-061107
2/6/83	Off-Gas	Recombiner rm. cooling fan 2B.	Fan inoperable.	Bad control transformer in 480 volt Turbine MOV Bd. 2B, Compt. 10A.	Replaced the bad control transformer, the cooling fan operated properly. MR #A-061095
2/14/83	RWCU	2A Rx Water cleanup holding pump.	Pump motor making excessive noise and vibration.	Bad motor bearing.	Replaced pump motor bearings, pump operated properly. MR #A-061041

BROWNS FERRY NUCLEAR PLANT UNIT 3

NON-CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of February 19 83

Date	System	Component	Nature of Maintenance	Cause of Malfunction or Maintenance Action	Corrective Action
2/4/83	Condensate and Demin Water	3D condensate demin holding pump motor.	Pump motor making excessive noise and vibrating.	Bad pump motor bearings.	Replaced motor bearings, pump operated properly. MR #A-063765

FIELD SERVICES SUMMARY

February 1983

Major Work Areas

A. Refuel area - the fuel reload continued through the first week of February and was completed on February 8 at 0405 hours. An average of 48 bundles per day was loaded out of 764. Verification was completed on February 9, 1983.

B. Turbine - the generator air test was successfully performed during February. This concluded all turbine floor work except for the modification to the turbine floor crane rails and the control intercept valve handrail work. Both of these jobs are currently being worked and will be complete prior to unit 2 cycle 4 startup. Preliminary planning for the unit 1 cycle 5 was started during February.

C. Drywell - with fuel reload complete, major efforts were made to prepare the drywell for ILRT. The failure of an MSRV on unit 1 was solved by swapping MSRV and vacuum breakers from unit 2 to unit 1, and the failed valve sent to Wyle Labs for repair. A redesign for the vacuum breakers was submitted by FSG valve engineer and with approval by EN DES, a prototype was started by mid-February. The following work items were essential for ILRT:

1. Install blanks at missing MSRV
2. Replace fittings for RBCCW pumps
3. Tip tubing installation and handcranking
4. Set up for ILRT
5. LLRT's of 64-17, 18, 19 and 76-24, 74-57, 58, 59 and H2/02
6. Complete MSRV air supply system.

FIELD SERVICES SUMMARY (Continued)

February 1983

Major Work Areas (Continued)

C. Drywell (Continued)

All of the above was accomplished by February 14 when tensioning of drywell head was complete. Pressurization of drywell started at 0100 hours on February 15, but had to be aborted due to hose problems at 0600 hours. On February 17 stabilization phase was in progress and on February 20 at 2300 hours drywell ILRT was completed. With completion of ILRT the critical path became the refuel floor and drywell preparations for RPV hydro. On February 21 the refurbished MSRV was received from Wyle and installation began immediately while the RPV preparations for hydro were worked in parallel. On February 25 the RPV was ready for pressurization and hydro was completed on February 28 at 0700 hours.

D. Electrical - during February the primary work performed by the electrical group was as follows:

1. Completed STEAR 8301 on the recirculation riser instrumentation installation.
2. Completed L2115E modification, cable pulls and thermocouple installation and hookup.
3. Completed installing all emergency lighting on unit 2 (P0479). Final walk-thru inspection is still outstanding.
4. Completed all work on the torus vacuum breaker modification and completed surveillance instruction (SI).

FIELD SERVICES SUMMARY (Continued)

February 1983

Major Work Areas (Continued)

D. Electrical (Continued)

5. Resumed TIP work and installed all indexers and successfully completed all handcranking. Wattage test will be performed next month.

6. Performed maintenance on the 2C condensate pump motor which requires disassembly from pump and shipment to the power service shop.

7. Completed P0600 recirculation brush holder modification on recirculation MG sets 2A and 2B.

8. Completed all electrical work and SI's related to the H2/02 modification (P0361E).

9. Support other outage work with various maintenance requests that were added to the outage scope of work.

E. Mechanical -

1. Complete retest of high-pressure feedwater heater A1 and A2 and B1 and B2. Repaired A2 heater and released to plant.

2. Reworked drywell blower dampers A and B side.

3. Installed one SDIV platform and started prefabrication of second.

4. Completed CRD "O" ring change-out.

5. Completed main steam snubber installation in drywell.

6. Started fabrication of parts for modification of vacuum breaker and ordered material for installation.

FIELD SERVICES SUMMARY (Continued)

February 1983

Major Work Areas (Continued)

E. Mechanical (Continued)

7. Completed all ISI work in drywell.

8. Completed all MSIV and MSRV mechanical work.

9. Recived approved pipe hanger changes from EN DES for MSS-13 and H-5 on February 28 and started prefabrication.

10. Completed paint touch-up in drywell.

F. Torus

1. Unit 2

During the first two weeks of February the following remaining work was completed.

a. Installation of torus seismic lug shims

b. ECCS gussets

c. N, J and L lapplates

d. Installation of MKII for tie-downs and shimming

e. Installation of torus snubbers, except bolting at one bracket

f. Started cleanup work and decon at El. 519

2. Unit 1

During February setup to work at unit 1 started with setting of protective barrier, moving of compressor from unit 2 and unit 1, power supplies, tool cage and lubrication of shims at seismic lugs. By the end of the month N, J, and L lapplates were pre-

FIELD SERVICES SUMMARY (Continued)

February 1983

Major Work Areas (Continued)

F. Torus (Continued)

fabricated and preparations made for installation, nine positions had drilling complete for torus snubber wall bracket, lifting lugs were 80 percent complete and 80 percent of ECCS gussets were tacked into place, baseplates drilling and installation of ECCS ring header snubbers 50 percent complete and prefabricated approximately 52 pipe hangers in shop.

G. Administrative - the overtime percentage for the month of January was 15 percent, with 148,077 straight time hours and 26,479.5 overtime hours. As of January 31, 1983 year-to-date overtime percentage was 21 percent, 581,322.5 hours of straight time hours and 153,568 overtime hours. The overall goal of the overtime percentage is 17 percent.

The O&M budget for January was $4,557,896 and the expenditures were $2,654,076 with year-to-date budget being $12,788,935 and actual year-to-date expenditures being $10,233,344. The capital budget was $4,979,009 and the expenditures were $1,398,357 with year-to-date budget being $16,096,805 and actual year-to-date expenditures being $6,781,703. Overall budget was $9,556,905 and the overall expenditures were $4,043,433 with year-to-date budget being $28,885,740 and actual year-to-date expenditures being $17,015,047. The February overall budget is $3,859,909.

ERRATA

Make corrections to pages 16, 17, and 18 of the January Monthly Operating Report, as per the attached pages.

OPERATING DATA REPORT

DOCKET NO. 50-259
DATE 2-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 1
2. Reporting Period: January 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	74,546
12. Number Of Hours Reactor Was Critical	698.55	698.55	48,088.09
13. Reactor Reserve Shutdown Hours	45.45	45.45	5,782.76
14. Hours Generator On-Line	679.45	679.45	47,079.57
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2,066,830	2,066,830	133,839,834
17. Gross Electrical Energy Generated (MWH)	683,790	683,790	44,084,510
18. Net Electrical Energy Generated (MWH)	665,447	665,447	42,815,226
19. Unit Service Factor	91.3	91.3	63.3
20. Unit Availability Factor	91.3	91.3	63.3
21. Unit Capacity Factor (Using MDC Net)	84.0	84.0	53.9
22. Unit Capacity Factor (Using DER Net)	84.0	84.0	53.9
23. Unit Forced Outage Rate	8.7	8.7	24.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

DOCKET NO. 50-260
DATE 2-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 2
2. Reporting Period: January 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	69,487
12. Number Of Hours Reactor Was Critical	0	0	43,293.47
13. Reactor Reserve Shutdown Hours	0	0	13,684.82
14. Hours Generator On-Line	0	0	41,975.45
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	120,480,340
17. Gross Electrical Energy Generated (MWH)	0	0	40,024,908
18. Net Electrical Energy Generated (MWH)	0	0	38,873,075
19. Unit Service Factor	0	0	60.4
20. Unit Availability Factor	0	0	60.4
21. Unit Capacity Factor (Using MDC Net)	0	0	52.5
22. Unit Capacity Factor (Using DER Net)	0	0	52.5
23. Unit Forced Outage Rate	0	0	27.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 3/7/83
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-296
DATE 2-1-83
COMPLETED BY Ted Thom
TELEPHONE 205/729-0834

OPERATING STATUS

1. Unit Name: Browns Ferry - 3
2. Reporting Period: January 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098.4
7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	51,912
12. Number Of Hours Reactor Was Critical	300.30	300.30	37,912.58
13. Reactor Reserve Shutdown Hours	443.7	443.7	3,815.85
14. Hours Generator On-Line	274.43	274.43	37,048.49
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	798,838	798,838	110,134,942
17. Gross Electrical Energy Generated (MWH)	279,800	279,069	36,319,590
18. Net Electrical Energy Generated (MWH)	267,069	267,069	35,247,973
19. Unit Service Factor	36.9	36.9	71.4
20. Unit Availability Factor	36.9	36.9	71.4
21. Unit Capacity Factor (Using MDC Net)	33.7	33.7	63.8
22. Unit Capacity Factor (Using DER Net)	33.7	33.7	63.8
23. Unit Forced Outage Rate	63.1	63.1	18.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		