

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | V | A | S | P | S | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
8 9 14 15 25 26 57 CAT 58

01 | L | 0 | 5 | 0 | 0 | 0 | 2 | 8 | 1 | 0 | 1 | 1 | 8 | 8 | 2 | 0 | 2 | 1 | 7 | 8 | 2 | 9
8 60 61 66 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | With unit two operating at 90 percent power, PT-10 revealed an unidentified
03 | Leak Rate in excess of 1 gpm. This event is contrary to T.S.3.1.C and is
04 | reportable in accordance with T.S.6.6.2.b.(2). The Leaking flange was located
05 | beyond the containment isolation valve, therefore, the health and safety of the
06 | public were not affected.

09 | P | C | E | B | V | A | L | V | E | X | P | B | 8 | 2 | 0 | 0 | 7 | 0 | 3 | L | 0 | A | Z | Z | 0 | 0 | 0 | 0 | Y | N | A | C | 7 | 1 | 0
7 8 9 10 11 12 13 18 19 20 21 22 23 24 26 27 28 29 30 31 32 33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The cause of this event was a flange leak. The flange gasket was replaced and no
11 | further leakage was noted.
12 |
13 |
14 |

15 | E | 0 | 9 | 0 | N/A | B | Periodic Test
7 8 9 10 12 13 44 45 46
16 | Z | Z | N/A | N/A
7 8 9 10 11 44 45
17 | 0 | 0 | 0 | Z | N/A
7 8 9 11 12
18 | 0 | 0 | 0 | N/A
7 8 9 11 12
19 | Z | N/A
7 8 9 10
20 | N | N/A
7 8 9 10

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 2

DOCKET NO: 50-281

REPORT NO: 82-007/03L-0

EVENT DATE: 01-18-82

TITLE OF THE EVENT: REACTOR COOLANT UNIDENTIFIED LEAKAGE GREATER THAN ONE GALLON PER MINUTE

1. DESCRIPTION OF EVENT:

With Un. two operating at 90 percent power, Periodic Test 10 revealed an unidentified Leak Rate in excess of one gallon per minute. This event is contrary to Technical Specification 3.1.C and is reportable in accordance with Technical Specification 6.6.2.b.(2).

2. PROBABLE CONSEQUENCES AND STATUS OF REDUNDANT EQUIPMENT:

Leakage monitoring is performed to ensure timely detection of any loss of RCS integrity. The point of leakage was located beyond the containment trip valve and therefore would have been isolated under accident conditions. The health and safety of the public were not affected.

3. CAUSE:

The cause of this event was an inlet flange leak on RV-2209.

4. IMMEDIATE CORRECTIVE ACTION:

A search for the leakage was initiated, however the untimely loss of 'E' transfer bus and the subsequent turbine runback (LER 82-008) delayed these efforts. Relief valve RV-2209 was found to have an inlet flange leak in excess of 1 gallon/minute. The leakage was evaluated and it was determined that the plant could be operated safely.

5. SUBSEQUENT CORRECTIVE ACTION:

The leaking flange was repaired.

Reactor Coolant Unidentified Leakage Greater Than One Gallon Per Minute

6. ACTION TAKEN TO PREVENT RECURRENCE:

None deemed necessary at this time.

7. GENERIC IMPLICATIONS:

None.