



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 172 TO

FACILITY OPERATING LICENSE NO. DPR-51

ENERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 1

DOCKET NO. 50-313

1.0 INTRODUCTION

By letter dated March 19, 1993, Entergy Operations, Inc. (the licensee) submitted a request for changes to the Arkansas Nuclear One, Unit No. 1 (ANO-1) Technical Specifications (TSs). The requested changes would modify TS 4.18.6 and Table 4.18-2 to make the requirements for C-3 reports on steam generator (SG) tube inspections consistent with the Babcock & Wilcox Standard TS. This is done by deleting the current requirement for prior NRC approval of the remedial actions described in the C-3 report.

2.0 EVALUATION

As currently written TS 4.18.6, 6.12.5.d, and Table 4.18-2 for ANO-1 require that, if the results of an in-service inspection (ISI) of the tubes of either or both SGs is characterized as C-3 (that is, if more than 10% of the total tubes inspected are degraded or more than 1% of the inspected tubes are defective), a special report shall be submitted to the NRC prior to resumption of plant operation. This report is to contain the SG tube inspection results and a description of the remedial action taken. As the TS are currently written, the remedial action described must also receive NRC approval.

This requirement for approval of the remedial action was added to Table 4.18-2 by the NRC in Amendment No. 41, issued April 11, 1979. The basis for this addition was not addressed in the associated NRC safety evaluation. In the presently considered license amendment, it is requested that the Action Required statement of Table 4.18-2 be changed to read "Special Report to NRC pursuant to 6.12.5.d" and that Note 2 for the table be revised to read "For C-3 results in one or both steam generators, plug or sleeve defective tubes and provide a Special Report to NRC pursuant to 6.12.5.d." A statement reading "The written Special Report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence" shall be added to TS 4.18.6 to clarify the required contents of a C-3 report. Finally, the reference to TS 6.12.5 is changed to TS 6.12.5.d for greater specificity.

The change, then, would eliminate the requirement for the NRC to approve the remedial action described in the C-3 report. This administrative action (for the NRC to approve the remedial action) is not contained in the previous

Standard TS "Steam Generator Tube Inspection" table (NUREG-0103, Rev. 4), nor is NRC approval of any part of the C-3 report discussed in the recently issued Babcock & Wilcox Standard TS (NUREG-1430, Rev. 0). Category C-3 reports will of course still be submitted prior to resumption of operation and will describe any corrective measures taken.

Since reference to remedial action is being deleted from Table 4.18-2, a description of the contents of a Category C-3 report is being added to TS 4.18.6. This added sentence is consistent with the language used in the Babcock & Wilcox Standard Technical Specifications. A typographical error introduced in Amendment No. 118 is being corrected by properly referencing TS 6.12.5.d rather than a nonexistent TS 6.12.4.5.

The proposed changes are appropriate, are consistent with current regulatory guidance and requirements, and are therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes recordkeeping, reporting, or administrative procedures or requirements, and changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 39049). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: May 19, 1994