MEMORANDUM FOR:

Brian K. Grimes, Director

Division of Operating Reactor Support

FROM:

Alfred E. Chaffee, Chief Events Assessment Branch

Division of Operating Reactor Support

SUBJECT:

OPERATING REACTORS EVENTS BRIEFING

MAY 11, 1994 - BRIEFING 94-15

On May 11, 1994, we conducted an Operating Reactors Events Briefing (94-15) to inform senior managers from offices of the Commission, AEOD, EDO, NRR, and regional offices of selected events that occurred since our last briefing on April 20, 1994. Enclosure 1 lists the attendees. Enclosure 2 presents the significant elements of the discussed events.

Enclosure 3 contains reactor scram statistics for the weeks ending April 24, and May 1, 1994. There were no scrams reported for the week ending May 8, 1994. Four significant events were identified for input into the NRC Performance Indicator Program (Enclosure 4).

> [original signed by] Alfred E. Chaffee, Chief Events Assessment Branch Division of Operating Reactor Support

Enclosures: As stated

cc w/enclosures: See next page

> DISTRIBUTION: Central Files PDR LKilgore, SECY EAB R/F

KGray TGreene

9405310008 940513 PDR DRG

& Khay EAB/DORS KGray: atb 05/12/94

EAB/DORS TGreene | 05//2/94

RETURN TO REGULATORY CENTRAL FILES AChaffee 05/13/94

OFFICIAL RECORD COPY

DOCUMENT NAME: G:\KAG\ORTRANS

operating Experiences X- Dam- 6- meetings.

260032

W. Russell, NRR (12G18)

F. Miraglia, NRR (12G18)

F. Gillespie, NRR (12G18)

Acting ADPR, NRR (12G18)

S. Varga, NRR (14E4)

J. Calvo, NRR (14A4)

G. Lainas, NRR (14H3)

J. Roe, NRR (13E4)

J. Zwolinski, NRR (13H24)

E. Adensam, NRR (13E4)

A. Thadani, NRR (12G18)

B. Sheron, NRR (7D26)

M. Virgilio, NRR (8E2)

S. Rosenberg, NRR (10E4)

C. Rossi, NRR (9A2)

B. Boger, NRR (10H3)

F. Congel, NRR (10E2)

D. Crutchfield, NRR (11H21)

W. Travers, NRR (11B19)

D. Coe, ACRS (P-315)

E. Jordan, AEOD (MN-3701)

G. Holahan, AEOD (MN-9112)

L. Spessard, AEOD (MN-3701)

K. Brockman, AEOD (MN-3206)

S. Rubin, AEOD (MN-5219)

M. Harper, AEOD (MN-9112)

W. Bateman, EDO (17G21)

F. Ingram, PA (2G5)

E. Beckjord, RES (NLS-007)

A. Bates, SECY (16G15)

T. Martin, Region I

R. Cooper, Region I

S. Ebneter, Region II

J. Johnson, Region II

S. Vias, Region II

J. Martin, Region III

E. Greenman, Region III

L. Callan, Region IV

A. Beach, Region IV

K. Perkins, Region IV/WCFO

bcc: Mr. Sam Newton, Manager

Events Analysis Department

Institute of Nuclear Power Operations 700 Galleria Parkway Atlanta, GA 30339-5957

J. Stang (PDIII-2)

J. Dyer (PDIII-2)



# NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 13, 1994

MEMORANDUM FOR:

Brian K. Grimes, Director

Division of Operating Reactor Support

FROM:

Alfred E. Chaffee, Chief Events Assessment Branch

Division of Operating Reactor Support

SUBJECT:

OPERATING REACTORS EVENTS BRIEFING

MAY 11, 1994 - BRIEFING 94-15

On May 11, 1994, we conducted an Operating Reactors Events Briefing (94-15) to inform senior managers from offices of the Commission, AEOD, EDO, NRR, and regional offices of selected events that occurred since our last briefing on April 20, 1994. Enclosure 1 lists the attendees. Enclosure 2 presents the significant elements of the discussed events.

Enclosure 3 contains reactor scram statistics for the weeks ending April 24, and May 1, 1994. There were no scrams reported for the week ending May 8, 1994. Four significant events were identified for input into the NRC Performance Indicator Program (Enclosure 4).

Alfred E. Chaffee, Chief Events Assessment Branch Division of Operating Reactor Support

Enclosures: As stated

cc w/enclosures: See next page

### ENCLOSURE 1

### LIST OF ATTENDEES

# OPERATING REACTORS EVENTS FULL BRIEFING (94-15)

### MAY 11, 1994

OFFICE
NRR
NRP
NRR
OEDO
RIV
OCM/IS
AEOD
AEOD

### TELEPHONE ATTENDANCE (AT ROLL CALL)

Regions	Resident Inspectors
Region I	Brunswick
Region II	M. Leach (Dresden)
Region III	
Region IV	

IIT/AIT Team Leaders

Misc.

# OPERATING REACTORS EVENTS BRIEFING 94-15

LOCATION: 10 B11, WHITE FLINT WEDNESDAY, MAY 11, 1994 11:00 A.M.

DRESDEN, UNIT 3

CORE SHROUD CRACKS

PRESENTED BY: EVENTS ASSESSMENT BRANCH
DIVISION OF OPERATING REACTOR
SUPPORT, NRR

# DRESDEN, UNIT 3 CORE SHROUD CRACKS **APRIL 1994**

### PROBLEM

CIRCUMFERENTIAL CRACK IN THE CORE SHROUD. THE CRACK WAS IN THE LOWER HEAT AFFECTED ZONE (HAZ) OF WELD H5 IN CORE PLATE SUPPORT RING WHICH IS 18 INCHES BELOW BOTTOM OF FUEL. ALSO WELD H3 HAD NUMEROUS CRACK INDICATIONS.

### CAUSES

THE CAUSE OF THE CRACK IS UNKNOWN AT THIS TIME. THE LICENSEE IS INVESTIGATING INTERGRANULAR STRESS CORROSION CRACKING (IGSCC), IRRADIATION ASSISTED STRESS CORROSION CRACKING (IASCC), RESIDUAL STRESSES IN THE WELD AREAS, AND ENVIRONMENT AT THE WELD LOCATION.

### SAFETY SIGNIFICANCE

A 360° THROUGH WALL FAILURE AT THE H5 WELD LOCATION COULD RESULT IN SHROUD MOVEMENT DURING DESIGN BASIS ACCIDENT CONCURRENT WITH A DESIGN BASIS EARTHQUAKE. THIS COULD AFFECT CONTROL ROD INSERTION. ALSO OF CONCERN ARE THE 2/3 CORE COVER CRITERION AND THE AFFECT OF THE SKROUD MOVEMENT ON THE CORE SPRAY SYSTEM.

COMMONWEALTH EDISON HAS STATED IF THERE WAS A 360° THROUGH WALL FAILURE AT WELD LOCATION H5, THE SHROUD WOULD NOT LIFT DURING THE FOLLOWING CONDITIONS: NORMAL OPERATION, A RECIRCULATION LINE BREAK ACCIDENT, AND A MAIN STEAM LINE BREAK ACCIDENT.

CONTACT: T. GREENE, NRR/DORS/OEAB

AIT: NO REFERENCE: PN39432 SIGEVENT: NO

### DISCUSSION

- DURING THE WEEK OF APRIL 25, VISUAL INSPECTIONS
   IDENTIFIED CRACKING IN THE VICINITY OF THE HAZ OF THE
   H5 WELD IN THE CORE SHROUD. THE CRACK EXTENDED
   360° AROUND THE CIRCUMFERENCE OF THE SHROUD AND WAS
   LOCATED IN THE CORE PLATE SUPPORT RING. THE DEPTH OF
   THE CRACK, AS DETERMINED FROM ULTRASONIC TESTING,
   RANGED FROM 0.95 TO 1.55 INCHES.
- FURTHER VISUAL INSPECTIONS REVEALED INDICATIONS IN OTHER CORE SHROUD WELD AREAS.
- VISUAL INSPECTION OF SHROUD SEAM WELDS WAS RECOMMENDED BY GE IN SIL #572, DATED 10/01/93 AFTER CRACKING WAS FOUND AT BRUNSWICK IN 1993 AND A FOREIGN REACTOR IN 1990.
- GE OWNER'S GROUP GUIDELINES DID NOT HAVE SPECIFIC RECOMMENDATIONS TO INSPECT ALL WELDS.
- COMMONWEALTH EDISON INSPECTED QUAD CITIES, UNIT 1, SHROUD AND FOUND A SIMILAR CRACK AT WELD H5.

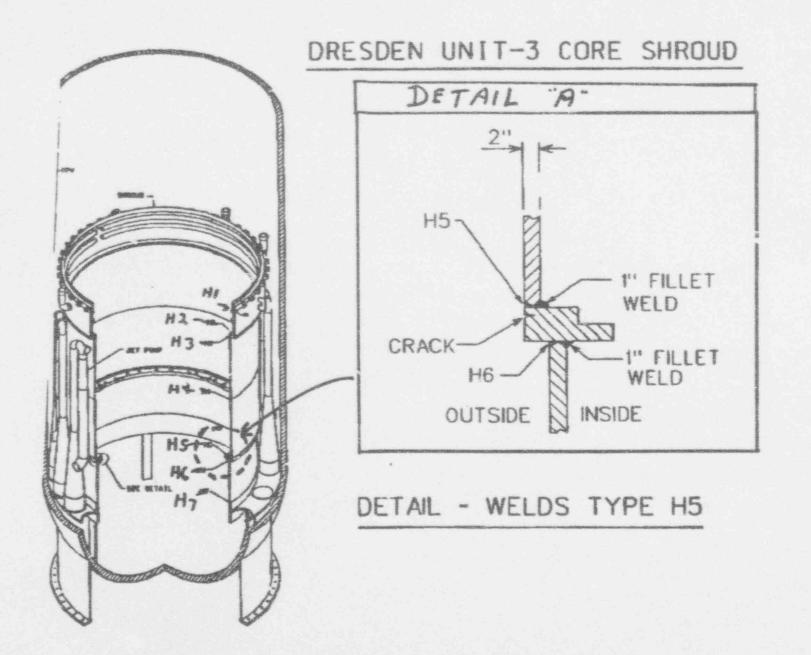
### **FOLLOWUP**

- COMMONWEALTH EDISON IS CONSIDERING THREE OPTIONS:
  - 1. DETERMINE BY INSPECTION SHROUD INTEGRITY AND CONTINUE TO OPERATE FOR 6 MONTHS.
  - PERFORM INTERIM SIMPLE REPAIR THAT WILL BE SAFE FOR ONE CYCLE.
  - PERFORM PERMANENT REPAIR.

- COMMONWEALTH EDISON IS PREPARING A JUSTIFICATION FOR CONTINUED OPERATION FOR DRESDEN, UNIT 2 AND QUAD CITIES, UNIT 2.
- NRC HAS SENT COMMONWEALTH EDISON A REQUEST FOR ADDITIONAL INFORMATION.
- NRC ISSUED INFORMATION NOTICE (IN) 93-79, "CORE SHROUD CRACKING AT BELTLINE REGION WELDS IN BOILING-WATER REACTORS," ON SEPTEMBER 30, 1993. AN IN IS BEING PREPARED AND ADDITIONAL GENERIC COMMUNICATION IS BEING CONSIDERED.
- LEAD PROJECT MANAGER (DON BRINKMAN) IS PROVIDING A LIST OF QUESTIONS FOR BWR PROJECT MANAGERS WHOSE PLANTS ARE SCHEDULED FOR RESTARTING PRIOR TO MID-JULY. THE QUESTIONS WILL DEAL WITH RECENT SHROUD INSPECTION.
- NRC IS SCHEDULING A MEETING WITH BWR OWNER'S GROUP EXECUTIVE OVERSIGHT COMMITTEE ON SHROUD CRACKING ISSUES FOR THE WEEK OF JUNE 13, 1994.

DRESDEN, UNIT 3 BRIEFING 94-15 DIFFERENTIAL PRESSURE AND LIQUID CONTROL LINE CORE PLATE, STANDBY LIQUID CONTROL INJECTION LINE DETAIL'A

Figure 13
Differential Pressure and Standby Liquid Control Line



#### REACTOR SCRAM

Reporting Period: 04/18/94 to 04/24/94

DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	ABOVE	SELOW 15%	YTD TOTAL
04/18/94	BROWNS FERRY 2	14	SA	Equipment failure	NO	1	1	2
04/22/94	COMANCHE PEAK 2	0	SA	Equipment Failure	NO	1	1	2
04/23/94	MILLSTONE 2	0	SM	Equipment Failure	MO	0	1	1
04/23/94	SAINT LUCIE 2	29	SA	Design or Installati	WO	1	0	1

#### REACTOR SCRAM

Reporting Period: 04/25/94 to 05/01/94

DATE	PLANT & UNIT	POWER	IYPE	CAUSE	COMPLICATIONS	ABOVE	BELOW 15%	YTD TOTAL
04/26/94	WATERFORD 3	70	SA	Equipment Failure	NO	1	0	1
04/26/94	WASHINGTON NUCLEAR 2	50	SM	Equipment Failure	NO	1	0	1
04/27/94	POINT BEACH 1	0	SA	Operating Error	NO	0	1	1
04/27/94	GINNA 1	45	SA	Equipment Failure	NO	1	0	1
04/30/94	DRESDEN 2	99	SM	Equipment Failure	NO	1	0	
05/01/94	SEQUOYAH 1	49	SA	Operating 1	NO	1	0	1

Note: Year To Date (YID) Totals Include Events Within The Calendar Year Indicated By The End Date Of The Specified Reporting Period

### COMPARISON OF WEEKLY SCRAM STATISTICS WITH INDUSTRY AVERAGES

### PERIOD ENDING 04/24/94

	NUMBER OF	1994 WEEKLY	1993 WEEKLY	1992 WEEKLY	1991* WEEKLY	1990* WEEKLY
SCRAM CAUSE	SCRAMS	AVERAGE	AVERAGE	AVERAGE	AVERAGE	AVERAGE
POWER GREATER THAN OR EQUAL TO	15%	(YTD)				
EQUIPMENT FAILURE*	0	1.60	1.83	2.62	2.88	3.38
DESIGN/INSTALLATION ERROR*	1	0.06	0.04			
OPERATING ERROR*	0	0.18	0.27	0.23	0.58	0.48
MAINTENANCE ERROR*	0	0.37	0.52	0.40		
EXTERNAL*	0	0.12	0.13			
OTHER*	0	0.00	0.02	0.23		
Subtotal		2.33	2.81	3.48	3.46	3.86
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	3	0.37	0.38	0.40	0.29	0.40
DESIGN/INSTALLATION ERROR*	0	0.06				
OPERATING ERROR*	0	0.12	0.13	0.13	0.15	0.08
MAINTENANCE ERROR*	0	0.00	0.02	0.06		
EXTERNAL*	0	0.00	0.04			
OTHER*	0	0.00		0.06		
Subtotal	3	0.55	0.57	0.65	0.44	0.48
TOTAL	4	2.88	3.38	4.13	3.90	4.34
		1994	1993	1992	1991	1990
	NO. OF	WEEKLY	WEEKLY	WEEKLY	WEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE (YID)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
TOTAL AUTOMATIC SCRAMS	3	2.21	2,44	3.06	3.25	3.21
TOTAL MINUAL SCRAMS	1	0.68	0.94	1.02	0.65	1,19

COTALS MAY DIFFER BECAUSE OF ROUNDING OFF

<sup>\*</sup> Detailed breakdown not in database for 1991 and earlier

<sup>-</sup> EXTERNAL cause included in EQUIPMENT FAILURE

<sup>-</sup> MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR

<sup>-</sup> OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

#### COMPARISON OF WEEKLY SCRAM STATISTICS WITH INDUSTRY AVERAGES

#### PERIOD ENDING 05/01/94

	NUMBER	1994	1993	1992	1991*	1990*
	OF	WEEKLY	WEEKLY	WEEKLY	WEEKLY	WEEKLY
SCRAM CAUSE	SCRAMS	AVERAGE	AVERAGE	AVERAGE	AVERAGE	AVERAGE
		(YTD)				
POWER GREATER THAN OR EQUAL TO	15%					
EQUIPMENT FAILURE*	4	1.74	1.83	2.62	2.88	3.38
DESIGN/INSTALLATION ERROR*	0	0.06	0.04			
OPERATING ERROR*	1	0.23	0.27	0.23	0.58	0.48
MAINTENANCE ERROR*	0	0.35	0.52	0.40		
EXTERNAL*	0	0.12	0.13			
OTHER*	0	0.00	0.02	0.23		
Subtotal	5	2.50	2.81	3.48	3.46	3.86
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.35	0.38	0.40	0.29	0.40
DESIGN/INSTALLATION ERROR*	0	0.06				
OPERATING ERROR*	1.1	0.17	0.13	0.13	0.15	0.08
MAINTENANCE ERROR*	0	0.00	0.02	0.06		
EXTERNAL*	0	0.00	0.04			
OTHER*	0	0.00		0.06		
Subtotal	-1	0.58	0.57	0.65	0.44	0.48
TOTAL	6	3.08	3.38	4.13	3.90	4.34
		1994	1993	1992	1991	1990
	NO. OF	MEEKLY	WEEKLY	WEEKLY	WEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
TOTAL AUTOMATIC SCRAMS	4	2.31	2.44	3.06	3.25	3,21
TOTAL MANUAL SCRAMS	2	0.75	0.94	1.02	0.65	1.19

TOTALS MAY DIFFER BECAUSE OF ROUNDING OFF

<sup>\*</sup> Detailed breakdown not in database for 1991 and earlier

<sup>-</sup> EXTERNAL cause included in EQUIPMENT FAILURE

<sup>-</sup> MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR

<sup>-</sup> DIHER cause included in EQUIPMENT FAILURE 1991 and 1990

#### NOTES

- 1. PLANT SPECIFIC DATA BASED ON INITIAL REVIEW OF 50.72 REPORTS FOR THE WEEK OF INTEREST. PERIOD IS MIDNIGHT SUNDAY THROUGH MIDNIGHT SUNDAY. SCRAMS ARE DEFINED AS REACTOR PPOTECTIVE ACTUATIONS WHICH RESULT IN ROD MOTION, AND EXCLUDE PLANNED TESTS OR SCRAMS AS PART OF PLANNED SHUTDOWN IN ACCORDANCE WITH A PLANT PROCEDURE. THERE ARE 111 REACTORS HOLDING AN OPERATING LICENSE.
- PERSONNEL RELATED PROBLEMS INCLUDE HUMAN ERROR, PROCEDURAL DEFICIENCIES, AND MANUAL STEAM GENERATOR LEVEL CONTROL PROBLEMS.
- 3. COMPLICATIONS: RECOVERY <u>COMPLICATED</u> BY EQUIPMENT FAILURES OR PERSONNEL ERRORS UNRELATED TO CAUSE OF SCRAM.
- 4. "OTHER" INCLUDES AUTOMATIC SCRAMS ATTRIBUTED TO ENVIRONMENTAL CAUSES (LIGHTNING), SYSTEM DESIGN, OR UNKNOWN CAUSE.

#### OEAB SCRAM DATA

Manual	and	Automatic	Scrams	for	1987	-	435
Manual	and	Automatic	Scrams	for	1988		291
Manual	and	Automatic	Scrams	for	1989		252
Manual	and	Automatic	Scrams	for	1990		226
Manual	and	Automatic	Scrams	for	1991	-	206
Manual	and	Automatic	Scrams	for	1992		212
Manual	and	Automatic	Scrams	for	1993		175
Manual	and	Automatic	Scrams	for	1994	(YTD 05/01/94)	53

### OPERATING REACTOR PLANTS SIGNIFICANT EVENTS

SORT> Event Date

QUERY> Event Type SIG & Close Out Date >= 04/01/94 & Close Out Date <= 05/06/94 & Event Type = "SIG"

PLANT & UNIT	DATE OF 50.72 EVENT NUMBER	DESCRIPTION OF EVENT	SIGNIFICANCE	BRIEFING	PRESENTER	CLOSEOUT
BEAVER VALLEY 2	10/17/93 0	FAILURE OF SPLINE COUPLING MAKES RECIRC SPRAY PUMP SUCTION VALVE INOPERABLE.	Safety-Related Cooling System	93-41	BENEDICT R.	HIGHLIGHT
BEAVER VALLEY 2	11/06/93 26326	EDG SEQUENCER FAILURE IN TWO TRAINS.	Safety-Related Cooling System	93-43	KOSHY T.	HIGHLIGHT
FERMI 2	12/25/93 26536	PLANT SCRAMMED FROM 93% POWER ON TURBINE TRIP WHEN THE TURBINE APPARENTLY THREW A BLADE.	OTHER - SCRAM WITH COMPLICATIONS	94-01	GREENE T.	HIGHLIGHT
MCGUIRE 2	12/27/93 26543	SCRAM WITH COMPLICATIONS CAUSED BY LOOP ENDING IN NATURAL CIRCULATION WITH ONE STEAM GENERATOR DRY.	Plant Power	94-01	BEHNER E.	HIGHLIGHT