NRC FORM 366 (7-77)		U. S. NUCLEAR REGULATORY COMMISSION
a thirt a the second	LICENSEE EVENT REPORT	
CONTROL BLOCK:		R TYPE ALL REQUIRED INFORMATION)
0 1 A L B R F 2 0 LICENSEE CODE 14	0 0 0 - 10 0 0 0 0 0 - 10 0 0 LICENSE NUMBER - 10 10 25	$ \boxed{3} \underbrace{4}_{26} \underbrace{1}_{\text{LICENSE TYPE}} \underbrace{1}_{30} \underbrace{4}_{67 \text{ CAT 58}} \underbrace{5}_{57} \underbrace{5}_{57 \text{ CAT 58}} \underbrace{5}_{57 $
CON'T REPORT SOURCE L 60 61 EVENT DESCRIPTION AND PROBA	0 1 0 1 0 1 2 1 6 1 0 0 1 1 1 1 7 DOCKET NUMBER 68 69 EVENT DATE ABLE CONSEQUENCES (10)	8 1 8 1 8 1 2 1 5 8 1 9 74 75 REPORT DATE 80
	ion, main steam line radiation mon	nitors 2-RM-90-137(C) and
0]3 [2-RM-90-139 (D) becan	me erratic and read lower than not	rmal. (T.S. table 3.1.A
0 4 Note 20) There was r	no danger to the health or safety	of the public. Redundancy
0 5 was reduced by one in	n each trip system. There were no	o previous similar events.
06		
07		
018 8 9		80
	AUSE CAUSE CODE SUBCODE COMPONENT CODE E 12 II 13 II N S T R 1 SEQUENTIAL OCCUR	U 18 RENCE REPORT VALVE SUBCODE SUBCODE U 10 18 REVISION
Image: Construction of the second	REPORT NO. COL 23 0 6 2 27 0 0 23 24 26 27 28 27 28 SF JTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED 23 21 0 0 0 1 Y 23 36 37 40 41 23 41 23	DE TYPE NO. 3 29 30 71 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
	se connectors. The connectors on	
	ned, SI 4.1.A-10 (main steam line	
12 [fully performed, and	2-RM-90-137(C) and 2-RM-90-139(D) were returned to service.
13 This is considered a	random failure and no further re	currence control is required.
1 4 7 8 9 FACILITY	(30) METHOD OF	80
T STATUS SPOWER T S S STATUS SPOWER T S S S S S S S S S S S S S S S S S S S	UTHER STATUS O DISCOVERT	or observed
7 8 9 PERSONNEL EXPOSURES	44 45	A
	NA NA	80
) NA	
1 8 9 11 12 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION		80
	NA	80
PUBLICITY ISSUED DESCRIPTION (45)	8112240257 811215 PDR ADDCK 05000260 S PDR	
8112240257 811215 PDR ADOCK 05000260 S PDR PDR PARER-	Gene Holder	PHONE (205) 729-6134

Tennessee Valley Authority Browns Ferry Nuclear Plant Form BF 17 BF 15.2 6/04/81

LER SUPPLEMENTAL INFORMATION

BFRO-50- <u>260</u> / <u>81062</u> Technical Specification Involved <u>T.S. Table 3.1.A Note 20</u> Reported Under Technical Specification <u>6.7.2.b(1)</u> * Date Due NRC <u>12/17/81</u> Date of Occurrence <u>11/17/81</u> Time of Occurrence <u>1600</u> Unit <u>2</u> Identification and Description of Occurrence:

Main steam line radiation monitors 2-RM-90-137(C) and 2-RM-90-139(D) became erratic and read lower than normal.

Conditions Prior to Occurrence: Unit 1 at 99%.

Unit 2 at 100%.

Unit 3 in refueling outage.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

None

Apparent Cause of Occurrence:

Loose-dirty connectors

Analysis of Occurrence:

There was no danger to the health or safety of the public, no release of activity, no damage to the plant or equipment, and no resulting significant chain of events.

Corrective Action:

The connectors were tightened, SI 4.1.A-10 was successfully performed, and the radiation monitors returned to service. This is considered a random failure and no further recurrence control is required.

Failure Data:

None Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: Ahaistnen