May 20, 1994 NG-94-1978

UTILITIES INC.

Mr. John B. Martin Regional Administrator Region III U. S. Nuclear Regulatory Commission 801 Warrenville Road Lisle, IL 60532

> Subject: Duane Arnold Energy Center Docket No: 50-331 Op. License DPR-49 Licensee Event Report #94-006

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the subject Licensee Event Report.

Very truly yours,

Julellerwoods

David L. Wilson Plant Superintendent - Nuclear

DLW/JWK/eah

cc: Director of Nuclear Reactor Regulation Document Control Desk U.S. Nuclear Regulatory Commission Mail Station P1-137 Washington, D. C. 20555

NRC Resident Inspector - DAEC

9405270142 940520 PDR ADOCK 05000331 PDR General Office • P.O. Box 351 • Cedar Rapids, Iowa 52406 • 319/398-4411

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| Duane Arnold Energy Center 05000 331 TITLE (4) Reactor Water Cleanup Isolation Due to Incomplete Valve Closure Caused by Position Indicator Obstruction EVENT DATE (5) LER NUMBER (6) REPORT NUMBER (7) O'HER FACILITIES INVOL | |
| TITLE (4) Reactor Water Cleanup Isolation Due to Incomplete Valve Closure Caused by Position Indicator Obstruction EVENT DATE (5) LER NUMBER (6) REPORT NUMBER (7) OTHER FACILITIES INVOL SEQUENTIAL REVISION | |
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| MONTH DAY YEAR YEAR NUMBER NUMBER MONTH DAY YEAR | DOCKET NUMBER |
| 04 26 94 94 006 00 05 20 94 FACILITY NAME | DOCKET NUMBER |
| OPERATING THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 1: (Check one or mo | ore) (11) |
| MODE (9) 1 20.402(b) 20.405(c) X 50.73(a)(2)(iv) | 73.71(b) |
| POWER 20.405(a)(1)(i) 50.36(c)(1) 50.73(a)(2)(v) | 73.71(c) |
| LEVEL (10) 100 20.405(a)(1)(ii) 50.36(c)(2) 50.73(a)(2)(vii) | OTHER |
| 20.405(a) (1) (iii) 50.73(a) (2) (viii) (A) (| Specify in Abstract below and in Text, NRC |
| 1 100 405(a)/4/64 | Form 366A) |
| 20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(x) | |
| LICENSEE CONTACT FOR THIS LER (12) | |
| VAME TELEPHONE NUMBER (Include Area | Code) |
| John W. Karrick, Technical Support Specialist (319) 851-7648 | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | REPORTABLE |
| CAUSE SYSTEM COMPONENT MANUFACTURER TO NPRDS CAUSE SYSTEM COMPONENT MANUFACTURE | TO NPRDS |
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| LAFECTED - | MONTH DAY YEA |
| YES SUBMISSION DATE) X NO DATE (15) | |
| ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16) | |
| At 0400 hours on April 26, 1994, with the plant operating at 100% power, a Gr V Primary Containment Isolation System (PCIS) signal was received and the appropriate valves closed as a result of a Reactor Water Cleanup (RWCU) hig differential flow signal. The flow differential occurred while an operator performing a system tagout during which the "A" RWCU pump discharge valve w required to be fully closed but was not. Subsequent system draining result in flow back through the partially open discharge valve resulting in the hi differential flow condition. The plant was in day 2 of a 7 day Limiting Condition for Operation (LCO) for the "A" Core Spray subsystem. | gh was vas ted |
| The cause of this event was that the position indicator on the discharge var reached the full closed position, stopping valve movement, prior to the val being full closed. Contributing factors were improper set up of the indica nut during previous work and the unavailability of communications in the RW pump room. | lve ator |
| | dure |

| NRC Form 366A (5-92) | U S. I | U.S. NUCLEAR REGULATORY COMMISSION | | | | | | | | APPROVED OMB NO 3150-0104 | | | | | | | | | | |
|--|--|------------------------------------|---|-----|----|---|---|-------------------|------------------|---|---|------------------|---|--------------------|---|----|---|--|--|--|
| LICENSEE EVENT REPORT (TEXT CONTINUATION | | (LER) | | | | | | | EXPIRE\$ 5/31/95 | | | | | | | | | | | |
| | | | | | | | | | | ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, J. 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503 | | | | | | | | | | |
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I. DESCRIPTION OF EVENT:

At approximately 0330 on April 26, 1994, a non-licensed operator began performing a tagout of the "A" Reactor Water Cleanup (RWCU) recirculation pump in support of maintenance on a valve on the pump discharge header. The "B" RWCU recirculation pump was in service and operating at the time. The "A" pump discharge valve was manually closed in the normal method by use of a handwheel attached to a Roto Hammer valve extension (Fig. 1). The operator used the remote position indicator in addition to feeling normal shutting torque on the handwheel in closing the valve for the tagout. As directed by the tagout, the operator began draining and venting the portion of the system isolated by the tagout. When the drain valves located between the pump discharge check valve and the pump discharge isolation valve were opened, flow commenced as expected through the drain valves into a closed drain system. The operator heard flow noise, but believed it was only from depressurizing the isolated portion of the system. It was not realized at that time that the drainage included back flow through the partially open pump discharge isolation valve. The control room noticed an increase in RWCU inlet flow of approximately 30 gpm at which time attempts were made to contact the operator in the RWCU pump room. Due to the lack of a plant page speaker in the room, the operator did not hear the control room operator's announcements to shut the drain valves. At 0400, after a 45 second time delay in the logic, a Group V Primary Containment Isolation (PCIS) signal was received and the appropriate RWCU isolation valves closed. The signal was from a high differential flow in the RWCU system. The setpoint for this actuation is 40 gpm.

A second operator was sent to the room who instructed the first operator to verify the valve positions of those valves on the tagout. Upon actions to locally check shut the "A" pump discharge valve, the position pointer on the Roto Hammer valve extension broke off, after which the valve was able to be moved 1/2 turn in the closed direction.

At the time of the isolation, the plant was operating at 100% power and was in day 2 of a 7 day Limiting Condition for Operation (LCO) for the "A" Core Spray subsystem. The isolation was reset and the RWCU system was returned to service at 0447.

| LICENSEE EVENT REPORT (TEXT CONTINUATION | | | U.S. NUCLEAR REGULATORY COMMISSION | | | | | | | | APPROVED OMB NO 3129-0104 EXPIRES: 5/31/95 | | | | | | | | | |
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XT (If more space is required, use additional NRC Form 366A) (17)

II. CAUSE OF THE EVENT:

The cause of this event was the remote position indicator on the Roto Hammer valve extension for the "A" RWCU pump discharge valve preventing full valve closure. The position indicator, when at the end of its travel, was able to draw the stem of the remote handwheel tight, simulating valve closing torque (See Fig. 1). This and the fact that it also indicated closed, misled the operator into believing the valve was closed.

A contributing factor to the indicator problem was an improper set up of the indicator nut during previous maintenance. The indicator had been previously broken and was replaced with a new pointer and indicator nut. This work was done with the system operating and the valve in the open position. Upon installation of the new indicator nut, it was adjusted to indicate open. Subsequent cycling of this valve has shown that the position indicator travel length is insufficient to allow for full valve travel. Also, because the critical function of this valve is to isolate the pump, the position indicator should have been set up with the valve in the fully closed position. These considerations were not understood at the time the maintenance was performed.

A contributing factor to the event was the lack of an immediate communication path between the control room and the operator in the RWCU pump room. Specifically, there was no plant page speaker in the room and the speaker outside the room could not be heard.

III. ANALYSIS OF EVENT:

There was no effect on plant safety, personnel safety or continued plant operation as a result of this event.

Though the RWCU system experienced fluid loss, the drains that were used are connected to a closed radioactive waste system such that there was no threat of leakage or contamination.

The tagout was written such that before maintenance could begin, the portion of the system that was isolated had to be drained and vented. In the event that the isolation had not occurred, there would not have been a significant threat to safety of maintenance personnel because of the drained and vented condition required by the tagout.

| U.S. NUC (5.92) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION | APPROVED DMB NO 3150-0104 EMPIRES 5/31/85 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MMBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503 LER NUMBER(6) PAGE(3) | | | | | | | | | | | |
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| The RWCU system responded as dest signal. The RWCU filter deminera mode of operation when the isolat returned to service at 0447. The of RWCU being isolated for 47 min | lizers automaticall tion valves closed. e effect on plant c | y switched to the Hold The system was chemistry as a result | | | | | | | | | | |
| IV. CORRECTIVE ACTIONS: | | | | | | | | | | | | |
| Interim guidance has been proverify valve positions when on System response will be used if local verification is pro- in place until permanent corr | operating valves wi when possible for hibited. This guid | th reach rods. this verification ance will remain | | | | | | | | | | |
| pump discharge valve. The mo possibility of the position | The position indicator will be repaired and modified on the "A" RWCU pump discharge valve. The modification will eliminate the possibility of the position indicator inhibiting full valve travel. This action will be completed by July 30, 1994. | | | | | | | | | | | |
| similar reach rods and indica generic corrective actions. for specific valve applicatio applicable actions will be co | An evaluation of the remaining values in the plant that have similar reach rods and indicators will be performed to determine generic corrective actions. The evaluation will specify actions for specific value applications and provide a schedule of when applicable actions will be completed. This evaluation will be completed by August 26, 1994. | | | | | | | | | | | |
| The tagout procedure will be when tagging out valves with means of determining valve pr action will be completed by | reach rods to ensu | ire a second | | | | | | | | | | |
| 5. A plant page speaker will be This action will be completed | | | | | | | | | | | | |
| A new maintenance procedure with the proper means for installar indicators. This action will | ation and set up of | the position | | | | | | | | | | |
| Training will be conducted for which will include the speci- the new maintenance procedure September 30, 1994. | fics of this event | and a review of | | | | | | | | | | |
| | | | | | | | | | | | | |

| NRC Form 365A (5-92) | U.S. NUC | LEAR | REGUL | ATORY | COM | MISSIC | NN. | | | | APPROVED O | MB | NO 3150-0104 | | | | | | |
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| | LICENSEE EVENT REPO | ORT (LER) | | | | | | | EXPIRES 5/31/95 | | | | | | | | | | |
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| TEXT (If more spa V | ice is required, use additional NRC form 30 . ADDITIONAL INFORMA | | | | | | | | | | den vergend. | | | | | adoust-ic rearran | | | |

A. Previous Similar Events:

LERs 85-013, 89-004, and 89-015 all involved RWCU isolations specifically from a high differential flow signal.

B. EIIS System and Component Codes:

PCIS - JM RWCU - CE Valve, isolation - ISV

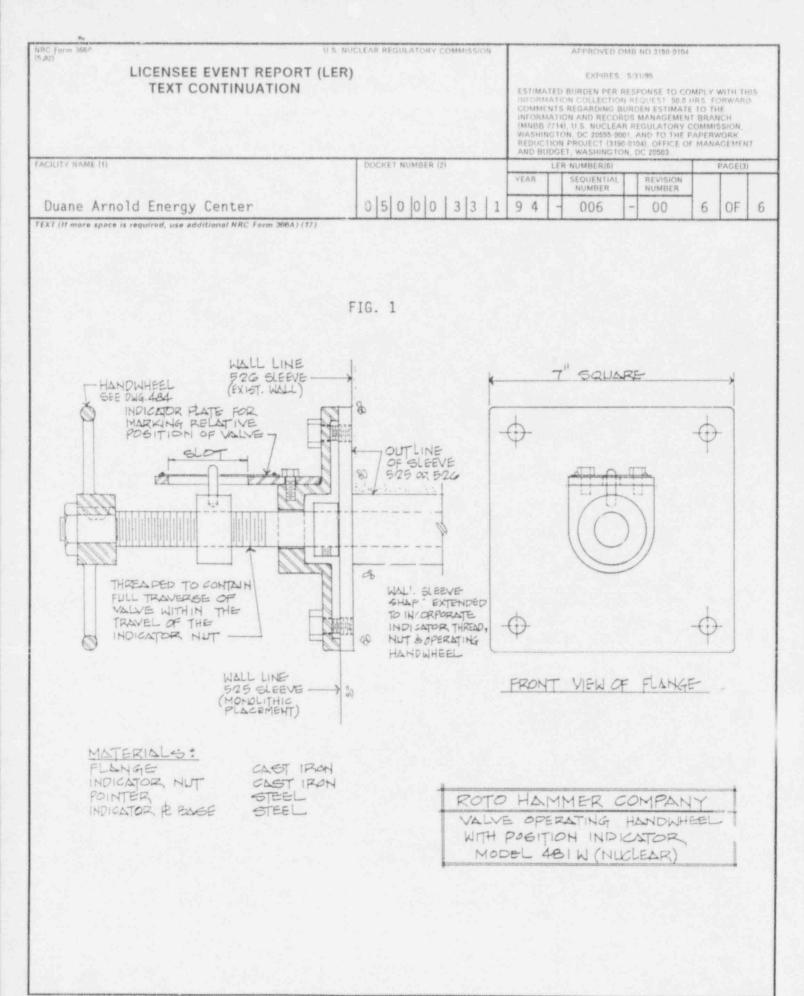
C. Equipment Information:

The handwheel and position indicator installed on the valve reach rod are Roto Hammer (R397) Model 482-B (drawing No. 481).

D. Reporting Criteria:

In reviewing this event for reportability, site personnel had initially determined that the signal causing the Engineered Safety Feature (ESF) actuation was invalid based on the guidance in the Federal Register dated September 10, 1992. The invalid signal was then applied to the exception criteria listed in 10CFR50.73(a)(2)(iv)(B)(3)(i). Subsequent evaluation determined that the signal was valid, and notification was made in accordance 10CFR50.72 but not within the required 4 hour time frame from when the actuation occurred.

This report is being submitted pursuant to 10CFR50.73(a)(2)(iv).



NRC FORM 386A (5-92)

DRM 366A (5-92)