NRC FOI (7-77)	ILICENSEE EVENT REPORT
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CON'T	REPORT L G 0 5 0 0 0 3 6 6 7 1 1 0 5 8 1 8 1 2 0 4 8 1 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
0 2	[On 11-5-81, with Unit 2 shutdown following a Rx scram, Bechtel personnel]
03	[were performing an insp. of electrical conduit when they discovered that]
04	RHRSW restraint 2E11-RSW-R26 located on the "B" RHRSW line in the torus
05	Larea was broken. The A/E's stress analysis revealed that the pipe would]
06	not be subjected to stress beyond its design capacity. RHRSW was not de-
07	clared inop. There were no effects upon public health & safety due to
08 78	this event. This is a non-repetitive occurrence for this system.
09	$ \begin{array}{c} \text{CODE} \\ S \\ 9 \\ 10 \\ \end{array} \begin{array}{c} \text{CODE} \\ 11 \\ 11 \\ 11 \\ 12 \\ 12 \\ 12 \\ 12 \\ 1$
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10	The cause of the support failure has been attributed to the tight fit
11	of the restraint U-bolt around the pipe line. The support welds were re-
12	paired and the U-bolt loosened to the proper clearance. A visual inspec-
13	tion of the support revealed no other defects. The A/E is performing a
14 78	Istructural analysis & upon completion an update report will be submitted.
15	FACILITY STATUS 9 10 12 13 0 0 0 0 0 0 0 0 0 0 0 0 0
	ACTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY 35 9 TO 10 TO 1
17	PERSONNEL EXPOSURES NUMBER 9 11 12 12 13 BO
19	LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION NA
7 8 2 0 7 8	PUBLICITY ISSUED DESCRIPTION (45) NA 5 PDR ADOCK 05000366 PDR 68 69 80 5
	NAME OF PREPARER C. L. Coggin - Supt. Plt. Eng. Serv. PHONE: 912-367-7851

LER #: 50-366/1981-103 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch Docket #: 50-366

Narrative Report for LER 50-366/1981-103

On 11-5-81, with the Unit 2 reactor shutdown following an MSIV closure SCRAM Bechtel engineering personnel were performing an inspection of electrical conduit within the Torus room area. During their inspection four defective welds were discovered on RHR Service water pipe support 2E11-RSW-R26, on the "B" RHR service water line downstream of the RHR heat exchanger. Deviation Report 2-81-190 was written, and is reportable under Tech Spec section 6.9.1.9.b. The preliminary recommendation by the A/E was to repair the defective welds, loosen the U-bolt to the proper clearance, and inspect the remaining support welds. These recommendations were implemented per Maintenance Request No. 2-81-4298. The A/E performed a stress analysis to determine the piping integrity. This analysis revealed that the piping was not subjected to any stress beyond its design capacity. Therefore, the "B" loop of the Unit 2 RHR Service water system was not declared inoperable. There were no effects upon public health and safety due to this event. This is a non-repetitive event for this system.

The cause of the support failure has been attributed to the tight fit of the support U-bolt around the pipe line. The U-bolt should have been installed with a clearance of 1/16" around the pipe line to allow axial movement. Due to an installation error, the U-bolt had been installed originally without any clearance. Following the repair an inspection c? the pipe support was performed and no other defects were found. To further ensure system integrity the A/E is performing a structural analysis to determine the exact loading effect the failed support would have upon the other pipe supports on this line. When this analysis is complete an update report will be submitted.