

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTSG. Structural Integrity

The structural integrity of the pressure boundaries shall be maintained at the level required by the original acceptance standard throughout the life of the plant.

G. Structural Integrity

1. In-service inspection of ASME Code Class I, Class II and Class III components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10CFR50, Section 50.55a(g)(6)(i).

~~a. The second 10-year interval for the in-service inspection program described above commenced on November 1, 1985.~~

2. In-Service testing of ASME Code Class I, Class II and Class III pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10CFR50, Section 50.55a(g)(6)(i).

- a. The second 10-year interval for the inservice testing program described above commenced on February 1, 1985.

~~The first 10 year interval for inservice inspections in accordance with the ASME Boiler and Pressure Vessel Code, Section XI commenced on February 1, 1975. This interval was extended for 9 months because of a 9 month outage for replacement of recirculation system inlet nozzle safe ends in 1978-79. Therefore, the first 10 year interval ended on October 31, 1985.~~

~~The second 10 year interval for inservice inspections commenced on November 1, 1985 and is scheduled to end on October 31, 1995. The second 10 year inspection program addresses the requirements of the ASME Code, Section XI, 1980 Edition with Addenda through Winter 1981, subject to the limitations and modifications as stated in 10 CFR 50.55a.~~

Visual ^{examinations} ~~inspections~~ for leaks will be made periodically on ^{ASME Section XI} ~~critical systems~~, Class 1, 2 and 3 systems. The inspection program specified encompasses the major areas of the vessel and piping systems within the ^{ASME Section XI} ~~drywell~~ boundaries. ~~The inspection period is based on the observed rate of growth of defects from fatigue studies sponsored by the NRC and is delineated by Section XI of the ASME Code. These studies show that it requires thousands of stress cycles at stresses beyond those expected to occur in a reactor system to propagate a crack. The test frequency established is at intervals such that in comparison to study results, only a small number of stress cycles, at values below limits will occur. On this basis, it is considered that the test frequencies are adequate.~~

The type of ^{examination} ~~inspection~~ planned for each component depends on location, accessibility, and type of ^{potential} ~~expected~~ defect. Direct visual examination is

proposed wherever possible since it is fast and reliable. Surface ~~inspections~~ ^{examinations} are planned where practical, and where added sensitivity is required. Ultrasonic ~~testing~~ ^{examination} or radiography shall be used where defects can occur in concealed surfaces. Section 5.2.4 of the Updated FSAR provides details of the inservice inspection program.

Starting with the Cycle 9/10 Refueling Outage, an augmented inspection program was implemented to address concerns relating to Intergranular Stress Corrosion Cracking (IGSCC) in reactor coolant piping made of austenitic stainless steel. The augmented inspection program conforms to the NRC staff's positions set forth in Generic Letter 88-01 and NUREG-0313, Revision 2 for inspection schedule, inspection methods and personnel, and inspection sample expansion.

The first 10-year interval for inservice testing of pumps and valves in accordance with the ASME Code, Section XI commenced on February 1, 1975 and ended on January 31, 1985. The second 10-year inservice testing interval commenced on February 1, 1985 and is scheduled to end on January 31, 1995. The second 10-year testing program addresses the requirements of the ASME Code, Section XI, 1980 Edition with Addenda through Winter 1981, subject to the limitations and modifications of 10 CFR 50.55a. Section 3.9.6 of the Updated FSAR describes the inservice testing program.

3.6.H & 4.6.H BASES:

Shock Suppressors (Snubbers)

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or other severe transient, while accommodating normal thermal motion during system startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of damage to piping as a result of a seismic or other event initiating dynamic loads or, in the case of a frozen snubber, exceeding allowable stress limits during system thermal transients. It is therefore required that all snubbers required to protect the primary coolant system or any other safety system or component be operable during reactor operation.

SAFETY ASSESSMENT1. INTRODUCTION

By letter dated May 6, 1994, IES Utilities Inc. submitted a request for revision to the Technical Specifications for the Duane Arnold Energy Center (DAEC). The proposed changes contained in this submittal which delete reference to the inservice inspection (ISI) program interval dates will allow DAEC's current ISI program interval to be extended in accordance with the ruling in Federal Register 57 FR 34666 dated August 6, 1992.

2. ASSESSMENT

The proposed revision deletes SR 4.6.G.1.a which refers to the ISI program interval. The proposed revision also adds clarification, deletes the inspection interval dates for the ISI program, deletes reference to the ASME edition and addenda requirements to be addressed by the ISI program and deletes information that is vague or no longer necessary from Technical Specification Bases 3.6.G and 4.6.G. The proposed revision does not involve a significant hazards consideration. The changes which delete reference to the ISI program interval dates are consistent with comparable specifications in the Improved Standard Technical Specifications (NUREG-1433). The ISI program will continue to conform with the ASME code and approved relief requests.

Based upon the above assessment, we conclude that this request is acceptable.

ENVIRONMENTAL CONSIDERATION

10 CFR 51.22(c)(9) identifies certain licensing and regulatory actions which are eligible for categorical exclusion from the requirement to perform an environmental assessment. A proposed amendment to an operating license for a facility requires no environmental assessment if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant hazards consideration; (2) result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite; and (3) result in a significant increase in individual or cumulative occupational radiation exposure. IES Utilities Inc. has reviewed this request and determined that the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment needs to be prepared in connection with the issuance of the amendment. The basis for this determination follows:

Basis

This revision meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) for the following reasons:

1. As demonstrated in Attachment 1, the proposed revision does not involve a significant hazards consideration.
2. No change in either plant design or operational strategies will be made as a result of this revision, therefore there will be no increase in either the types or amounts of effluents that may be released offsite.
3. No changes in either plant design or operational strategies will be made as a result of this revision, therefore there will be no significant increase in either individual or cumulative occupational radiation exposure.