EMERGENCY ROSTER

APPENDIX A

9 14 Barra 16-5-8)
Gaplant Manager / Date

DUTY CALL SUPERVISORS

MAINTENANCE DUTY CALL SUPERVISOR	OPERATIONS DUTY CALL SUPERVISOR	H. P. DUTY CALL SUPERVISOR
T. A. Dillard	D. A. Sager	G. M. Green
C. F. Leppla	J. A. Spodick	H. F. Buchanan
J. E. Bowers	B. W. Mikell	H. M. Mercer
M. B. Vincent	C. A. Pell	
	J. H. Barrow	
	P. L. Fincher	
	N. G. Roos	
	R. S. Glaze	
	M. D. Sheppard	
Beeper No.	Beeper No.	Beeper No.

E-PLAN IMPLEMENTING PROCEDURE NO. 3100023E, REV. 20 EMERGENCY ROSTER

APPENDIX C

DUTY CALL SUPERVISOR'S CALL LIST

TITLE	NAME	HOME OFFICE			
Emer. Recovery & Restoration team leader - Plant Manager	C. M. Wethy	Ex.			
Alternate: Ops. Superintendent	J. H. Barrow	Ex.			
Emergency Control Officer	A. D. Schmidt				
		Beeper			
ALTERNATES					
Manager Power ResNuclear	C. O. Woody	Beiper			
Asst. Vice Pres. Power Res.	J. R. Benson				
		Beeper			
Asst. Mgr. Pwr. ResNuclear	K. N. Harris	(Ft. Pierce-Weekends			
		(Miami-Weekdays			
		Beeper			
Mgr. of Pwr. Res.	H. N. Paduano				
Nuclear Services		Beeper			
Pwr. Res. Specialist	D. K.James	- American			
		Beeper			
Pwr. Res. Sec. Supv. Plant	R. J. Acosta	beeper			
		pseper			
Pwr. Res. Sec. Supv. Health	J. L. Danek	Baeper			
Systems Operations Power					
Coordinator					
TEAM LEADERS & ALTERNATES					
Emergency radiation team	H. F. Buchanan	Ex.			
Alternate emergency radiaion team leader	H. M. Mercer	Ex.			
Alternate emergency	G. M. Green or	Ex.			
radiation team leader	R. M. McCullers	Ex.			
Plant Fire Protection Coordinator	W. J. Barrow	Ex.			

FLORIDA POWER & LIGHT COMPANY ST. LUCIE UNIT #1 E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E REVISION 10

1.0 Title:

DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

2.0 Approval:

Reviewed by	Plant Nuclear Safety Com	mittee	July 25,	1975
Approved by	K. N. HarrisPlant Manage	r	July 29,	1975
Rev. 10 Rey	Sewed by F R G	Max	.28	1981
Approved by	Allow Vice Pres.	Power Resources	Luce 2	. 19 0/

3.0 Scope:

3.1 Purpose:

This procedure provides the instructions to be followed by the Emergency Coordinator when an emergency occurs that requires the initiation of the Emergency Plan.

3.2 Discussion:

The Nuclear Plant Supervisor becomes the Emergency Coordinator upon initiation of the Emergency Plans and, as such, directs the On-Site Emergency Organization to bring the emergency under control.

3.3 Authority:

This procedure implements the St. Lucie Plant Radiological Emergency Plan.

4.0 Precautions:

- 4.1 The Nuclear Plant Supervisor and the shift operating staff represent the first-line of response to any developing emergency condition. The primary responsibility of the Nuclear Plant Supervisor is to control the condition as well as possible. However, the success of the Emergency Plan and Procedures requires immediate classification of the emergency (in accordance with E-Plan Implementing Procedure 3100022E, Classifications of Emergencies) and notifications of designated off-site authorities as well as FPL's Off-Site Emergency Organization (see Figure 1 for notification flow).
- 4.2 The Emergency Coordinator can delegate his responsibilities at his discretion with the exception of the decision to notify state and local authorities, the content of that notification and of providing protective action recommendations to state and local authorities. When the TSC is operational, the Emergency Coordinator can delegate all off-site communications to the TSC Supervisor.

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

5.0 Responsibilities:

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- 5.1 If the Nuclear Plant Supervisor is incapacitated, the Emergency Coordinator shall be (in order of succession):
 - 5.1.1 Nuclear Watch Engineer
 - 5.1.2 Any other member of the plant staff with a Senior Reactor Operator License.
- 5.2 The Emergency Coordinator shall only grant permission for watch relief, including his own, when it is safe in his judgement to do so.
- 5.3 The Operations Superintendent, the Operations Supervisor or a Operations Duty Call Supervisor can assume the Emergency Coordinator function at his discretion following the proper turnover procedure.
- 5.4 The Technical Support Center Supervisor shall fill out the checklist for Notification of Significant Events (to NRC) as soon as possible after activating the TSC and preferably before calling the NRC.
- 5.5 The Emergency Coordinator is responsible for providing protective action recommendations to off-site authorities. This responsibility cannot be delegated.

6.0 References:

- 6.1 St. Lucie Plant Radiological Emergency Plan
- 6.2 E-Plan Implementing Procedures, as follows:

EPIP-3, Classification of Emergencies.

EPIP-5, Criteria for and Conduct of Evacuations.

EPIP-6, Radiation Release and Dose Projection.

7.0 Records:

All significant information, events, and actions taken during the emergency period shall be resorded as directed by the Emergency Coordinator.

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

8.0 Instructions:

8.1 Classification

Immediately upon becoming aware of an abnormal condition, the Nuclear Plant Supervisor shall classify the condition in accordance with the criteria given in E-Plan Implementing Procedure 3100022E, Classification of Emergencies. If the classification is, UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY, or GENERAL EMERGENCY, the Nuclear Plant Supervisor shall declare an emergency and become the Emergency Coordinator. The Emergency Coordinator shall, until relieved, remain in the Control Room at all times during emergency situations that require Fmergency Plant implementation unless, in his opinion, his personal evaluation of the situation is necessary to maintain plant safety.

8.2 The Emergency Coordinator shall mobilize the On-Site Emergency Organization to begin required corrective and protective actions.

8.3 UNUSUAL EVENT

If an UNUSUAL EVENT is declared, the Emergency Coordinator shall complete the actions specified in the attached UNUSUAL EVENT CHECKLIST. These actions include required off-site notifications and on-site protective actions.

8.4 ALEKT

If an ALERT is declared, the Emergency Coordinator shall complete the actions specified in the attached ALERT CHECKLIST. These actions include required off-site notifications, on-site protective actions, and activation of on-site support centers.

8.5 SITE AREA EMERGENCY

If a SITE AREA EMERGENCY is declared, the Emergency Coordinator shall complete the actions specified in the attached SITE AREA EMERGENCY CHECKLIST. These actions include required off-site notifications, on-site protective actions, and activation of on-site support centers.

8.6 GENERAL EMERGENCY

If a GENERAL EMERGENCY is declared, the Emergency Coordinator shall complete the actions specified in the attached GENERAL EMERGENCY CHECKLIST. These actions include required off-site notifications, on-site protective actions, and activation of on-site support centers.

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

8.0 Instructions: (continued)

8.7 MEDICAL EMERGENCIES

The attached MEDICAL EMERGENCY CHECKLIST shall be followed in the event of injury to plant personnel or visitors. The MEDICAL NOTIFICATION FORM attached to the Checklist shall be prepared for each victim and shall, if possible, accompany the victim to off-site emergency treatment. The MEDICAL EMERGENCY CHECKLIST may be followed independently of the Radiological Emergency Plan.

- 8.8 Responsibilities for off-site communications and coordination shall be relinquished first to the Technical Support Center Supervisor and then to the Emergency Control Officer when they establish contact and assume responsibilities.
- 8.9 OFF-SITE EVACUATION RECOMMENDATIONS

The Emergency Coordinator is responsible for providing protective action recommendations to off-site authorities in accordance with Table 1, "Guidelines for Protective Action Recommendations to Off-site Authorities".

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10
DUTIES AND PESPONSIBILITIES OF THE EMERGENCY COORDINATOR

UNUSUAL EVENT CHECKLIST

ACTIONS TO BE TAKEN BY EMERGENCY COORDINATOR OR HIS DESIGNEE IN THE EVENT OF AN UNUSUAL EVENT

1.	Order initial corrective action per Emergency Operating
	Fire/Explosion - see attached FIRE/EXPLOSION CHECKLIST
	Medical - see attached MEDICAL EMERGENCY CHECKLIST
2.	Mobilize Interim Emergency Teams to respond as necessary.
3.	Commence preparation of the attached EMERGENCY INFORMATION CHECKLIST excluding off-site dose projections.
4.	During current business day, notify the Plant Manager, during off hours and weekends relay information to the Duty Call Supervisor (see NPS Bulletin Board for scheduled supervisor and telephone number). Order him to notify the following:
	Plant Manager Operations Superintendent Operations Supervisor Technical Department Supervisor
5.	Within 15 minutes of classifying the emergency:
	1. Notify, by telephone, the State Warning Point at the Bureau of Disaster Preparedness in Tallahassee and communicate EMERGENCY INFORMATION CHECKLIST data. PHONE NO, Florida Marine Patrol-Alternate State Warning Point
	NOTE: Expect a message validation callback from B.D.P.
	 Notify, by telephone, the State Radiological Emergency Duty Officer at Radiological Health Services in Orlando and communicate FMERGENCY INFORMATION CHECKLIST data. PHONE No.
6.	Prepare attached SIGNIFICANT EVENT REPORT (if time permits) and notify NRC via hot-line (within one hour)
7.	Reassess the Emergency Classification and update the EMERGENCY INFORMATION CHECKLIST.

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

ALERT CHECKLIST

ACTIONS TO BE TAKEN BY EMERGENCY COORDINATOR OR HIS DESIGNEE IN THE EVENT OF AN ALERT

 1.	Order initial corrective action per Emergency Operating Procedures.
	Fire/Explosion - see attached FIRE/EXPLOSION CHECKLIST
	Medical - see attached MEDICAL EMERGENCY CHECKLIST
2.	Notify personnel of the emergency condition over the Unit #1 PA system, giving location, class, and type of emergency.
	Repeat announcement
3.	If the evacuation of an area is necessary, initiate a local evacuation in accordance with E-Plan Implementing Procedure 3100026E, Criteria for and Conduct of Evacuations. Announce the following:
	Area to be evacuated
	Areas to be avoided, if possible, during the
	Evacuation
	Assembly area
 4.	Mobilize Interim Emergency Teams to respond as necessary.
5.	Commence preparation of the attached EWMERGENCY INFORMATION CHECKLIST including off-site dose projections using E-Plan Implementing Procedure 3100033E, Radiation Release and Dose Projection.
 6.	Relay information to the Duty Call Supervisor (see NPS Bulletin Board for scheduled supervisor and telephone number). Order him to notify the following:
	71 W
	Plant Manager Operations Superintendent
	Operations Supervisor
	Technical Department Supervisor
	Emergency Control Officer (or Radiological Duty
	Officer) Other Department Supervisors (Primary Team
	Leaders)
	(ALTERNATE METHOD if Duty Call Supervisor is unavailable- Notify Emergency Control Officer.
 7.	Notify the following within 15 minutes of classifying the emergency:

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

ALERT CHECKLIST(Continued)

		7.1 Notify, by telephone the State Warning Point at the Bureau of Disaster Preparedness in Tallahassee and communicate EMERGENCY INFORMATIO CHECKLIST data. PHONE NO. or Florida Marine Patrol-Alternate State Warning Point
		NOTE: Expect a message validation callback from B.D.P.
	-	7.2 Notify, by telephone the St. Lucie County Disaster Preparedness Office in Fort Pierce, and communicate EMERGENCY INFORMATION CHECKLIST data. PHONE NO. 911
		7.3 Notify, by telephone the Martin County Disaster Preparedness Office in Stuart and communicate EMERGENCY INFORMATION CHECKLIST data. PHONE NO.
		7.4 Notify, by telephone the State Radiologhical Emergency duty Officer at Radiological Health Services in Orlando and communicate EMERGENCY INFORMATION CHECKLIST data. PHONE NO.
	8.	Prepare attached SIGNIFICANT EVENT REPORT (if time permits) and notify NRC via hot-line (within one hour). (To be done by TSC if properly staffed)
	9.	Verify from Security Team Leader that all personnel are accounted for (if local evacuation was conducted).
	10.	Reassess corrective and protective actions. Verify activities underway, reassign personnel and teams as necessary.
udenium ar Nahada sakas 1994	11.	Activate the Technical Support Center and the Operational Support Center. Appoint OSC Supervisor.
	12.	Brief the Technical Support Center Supervisor on events. Order him to provide state and county with periodic updates and to refine dose projections when the Chemistry Department representative arrives.
	13.	Reassess the Emergency Classification and update the EMERGENCY INFORMATION CHECKLIST with Technical Support Center Supervisor.
	14.	Relinquish responsibilities for communications with off- site support agencies when the Emergency Control Officer notifies the Emergency Coordinator that he accepts these responsibilities.

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

SITE AREA EMERGENCY CHECKLIST

ACTIONS TO BE TAKEN BY EMERGENCY COORDINATOR OR HIS DESIGNEE IN THE EVENT OF SITE AREA EMERGENCY.

 1.	Order initial corrective action per Emergency Operating Procedures.
2.	Notify personnel of the emergency condition over the PA system (connect all units), giving location, class, and type of emergency.
3.	Order all non-essential personnel to commence evacuation of the Owner Controlled Area in accordance with E-Plan Implementing Procedure 3100026E, Criteria for and Conduct of Evacuations.
 4.	Sound Site Evacuation Alarm.
5.	Repeat PA announcement.
6.	Oruer Security Team Leader to evacuate Owner Controlled Area and to report personnel accountability as soon as possible.
7.	Mobilize other Interim Emergency Teams to respond as necessary.
8.	Commence preparation of the attached EMERGENCY INFORMATION CHECKLIST, including off-site dose projection using E-Plan Implementing Procedure 3100033E, Radiation Release and Dose Projection. Fire/Explosion and Medical Checklist are attached if required.
9.	Relay information to the Duty Call Supervisor (see NPS Bulletin Board for scheduled supervisor and telephone number). Order him to notify the following:
	Plant Manager
	Operations Superintendent
	Operations Supervisor
	Technical Department Supervisor
	Emergency Control Officer
	Other Department Supervisors (Primary Team Leaders)
	(ALTERNATE METHOD If Duty Call Supervisor is unavailable-

(ALTERNATE METHOD If Duty Call Supervisor is unavailablenotify Emergency Control Officer. E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10
DUTIES AND RESPONSIBILITIES OF THE EMERGENTY COORDINATOR

SITE AREA EMERGENCY CHECKLIST(Con inued)

10.	NAWAS Announcement: (to be accomplished within 15 minutes after declaring the emergency condition).
	"State Warning Point Tallahassee, this is St. Lucie One (State Warning Point will give a go-anead) "State Warning Point Tallahassee, this is St. Lucie One, Ground Fog, SITE AREA EMERGENCY, repeat SITE AREA EMERGENCY." Additional data will be transmitted by Local Government Radio.
	Time
	"Acknowledge, over."
	(ALTERNATE METHOD if NAWAS is unavailable-telephone State Warning Point at PHONE NO. or Florida Marin Patrol-Alternate State Warning Point
11.	State Warning Point Message Validation Time (NAWAS and LGR announcement also serves to notify St. Luci and Martin Counties and the State Department of Health and Rehabilitative Services.)
12.	Relay EMERGENCY INFORMATION CHECKLIST data including off- site dose projections by LGR.
13.	Also, by LGR, contact St. Lucie County Disaster Preparedness, inform them that site evacuation has started location of assembly area(s), evacuation route(s). Notify them of any wind changes, and when evacuation is completed
14.	Prepare attached SIGNIFICANT EVENT REPORT (if time permits) and notify NRC via hot-line (within one hour). (To be done by TSC if properly staffed.)
15.	Verify from Security Team Leader that Owner Controlled Evacuation (including Unit #2 construction site) is complete and that all personnel are accounted for.
 16.	Reassess corrective and protective actions. Verify activities underway, reassign personnel and teams as necessary.
17.	Activate the Technical Support Center and the Operational Support Center.
18.	Brief the Technical Support Center Supervisor on events. Order him to update state and county periodically and to refine dose projections when the Chemistry

Department representative arrives.

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Page 10 of 20

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

SITE AREA EMERGENCY CHECKLIST(Continued)

	19.	Reassess	the	Cmergency	Classific	cation	and	update th	ne
		EMERGENCY	IN	FORMATION	CHECKLIST	with	the	Technical	Support
		Center Su	per	visor.					

20. Relinquish responsibilities for communications with offsite support agencies when the Emergency Control Officer notifies the Emergency Coordinator that he accepts the responsibilities.

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

GENERAL EMERGENCY CHECKLIST

ACTIONS TO BE TAKEN BY EMERGENCY COORDINATOR OR HIS DESIGNEE IN THE EVENT OF GENERAL EMERGENCY

 1.	Order initial corrective action per Emergency Operating Procedures.
 2.	Notify personnel of the emergency condition over the PA system (connect all units) giving location, class, and type of emergency.
3.	Order all non-essential personnel to commence evacuation of the Owner Controlled area in accordance with E-Plan Implementing Procedure 3100026E, Criteria for and Conduct of Evacuations.
4.	Sound Site Evacuation Alarm.
5.	Repeat PA Announcement.
6.	Order Security Team Leader to evacuate Owner Controlled Area and to report personnel accountability as soon as possible.
 7.	Mobilize other Interim Emergency Teams to respond as necessary.
8.	NAWAS Announcement: (to be accomplished within 15 minutes after declaring the emergency condition.) "State Warning Point Tallahassee, this is St. Lucie One" (State Warning Point will give a go-ahead.) "State Warning Point Tallahassee, this is St. Lucie One Ground Fog, we have a GENERAL EMERGENCY, repeat GENERAL EMERGENCY. Time Wind mph from degrees Meteorological conditions are (stable/unstable) Additional EMERGENCY INFORMATION will be forthcoming shortly. Acknowledge, over."
	(ALTERNATE METHOD If NAWAS is unavailable, telephone State Warning Point at PHONE NO. or Florida Marine Patrol-Alternate State Warning Point
 9.	State Warning Point Message validation Time
	(NAWAS announcement also serves to notify St. Lucie and Martin Counties and the State Department of Health and

Rehabilitative Services.)

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

GENERAL EMERGENCY CHECKLIST(Continued)

10.	Complete EMERGENCY INFORMATION CHECKLIST including off- site dose projections using E-Plan Implementing Procedure 3100033E, Radiation Release and Dose Projection.
 11.	Transmit EMERGENCY INFORMATION CHECKLIST data by LGR (NAWAS or telephone of LGR not operational).
 12.	State Warning Point Acknowledgement Time
13.	Also by LGR, contact St. Lucie County Disaster Preparedness, inform them that site evacuation has started, location of assembly area(s), evacuation route(s). Notify them of any wind changes, and when evacuation is completed.
14.	Relay information to the Duty Call Supervisor (see NPS Bulletin Board for scheduled supervisor and telephone number). Order him to notify the following:
	Plant Manager
	Operations Superintendent
	Operations Supervisor
	Technical Department Supervisor
	Emergency Control Officer
	Other Department Supervisors (Primary Team Leaders)
	ALTERNATE METHOD if Duty Call Supervisor is unavailable- Notify Emergency Control Officer.
15.	Prepare attached SIGNIFICANT EVENT REPORT (if time permits) and notify NRC via hot-line (within one hour). (To be done by TSC if properly staffed.)
 16.	Verify from Security Team Leader that Owner Controlled Evacuation (including Unit #2 construction site) is complete and that all personnel are accounted for.
 17.	Reassess corrective and protective actions. Verify activities underway, reassign personnel and teams as necessary.
 18.	Activate the Technical Support Center and the Operational Support Center.
19.	Brief the Technical Support Center Supervisor on events. Order him to update state and county periodically and to refine dose projections when the Chemistry Department representative arrives. (EOF will perform these tasks when operational).

Page 13 of 20

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10
DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

GENERAL EMERGENCY CHECKLIST(Continued)

- 20. Reassess the Emergency Classification and update the EMERGENCY INFORMATION CHECKLIST with the Technical Support Center Supervisor.
 - 21. kelinquish responsibilities for communications with offsite support agencies when the Emergency Control Officer notifies the Emergency Coordinator that he accepts the responsibilities.

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

EMERGENCY INFORMATION CHECKKLIST

1.	Description of incident	
2.	Emergency Classification	
3.	Location of Incident	
4.	Date/time of Incident	
5.	Assessment of the emergency (including) potential for esca-lating to higher class)	
6.	Personnel injuries and radiation exposures	
7.	Estimate of radioactive material released.	
3.	Areas potentially affected	
9.	Wind speed	
10.	Down wind direction and range over past nour	
11.	Assessment of potential radiation exposure to persons off-site (use E-Plan Implementing	
	Procedure 3100033E, Radiation Release and Dose Projection)	

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

FIRE OR EXPLOSION EMERGENCY CHECKLIST

TIME	
	P.A. Announcement of location and extent of fire.
	Sound Fire Alarm.
	Activate the Fire Brigade and other appropriate emergency teams in accordance with Fire Emergencies EPIP 3100025E - PA announcement.
	St. Lucie County - Fort Pierce Fire District PHONE No. 911.
	Return to appropriate UNUSUAL EVENT ALERT, SITE AREA EMERGENCY, or GENERAL EMERGENCY checklist.

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

MEDICAL EMERGENCY CHECKLIST

TIME					
	Determine:				
	Name of Victim				
	Employer (if not FPL)				
	Nature and Extent of Injury				
	Location Is victim contaminated?				
	Activate First Aid and Personnel Decontamination Team (Notify Chemistry Supervisor).				
	Notify:				
	Health Physics				
	Plant Manager				
	Operations Superintendent				
	When determined, notify Captain of Guard where to direct ambulance, etc.				
	Determine mode of transportation based on nature and extent of injuries. (Ensure victim's TLD, selfreader, ID badge and key card are retained on site).				
	a. Medical treatment for serious injury: AMBULANCE PHONE NO. 911				
	b. Medical treatment for minor injuries: FPL Venicle/Private Vehicle				
	Send all victims to LAWNWOOD MEDICAL CENTER PHONE NO.				
	Radioactively contaminated victims: Notify REEF PHONE NO.				
	If Plant Manager not available, notify Duty Call Supervisor and Health Physics Duty Call Supervisor-See Emergency Roster for Off-site notifications.				
	Complete attached Medical Notification form to accompany victim.				

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

MEDICAL NOTIFICATION FORM

In the event of a radiation emergency which requires the transportation of casualties to REEF, located within Mt. Sanai Hospital, Lawnwood Medical Center, or Radiology Associates, the Emergency Coordinator shall transmit the following information, if it is available:

1.	Name and security badge number of the casualty					
	NameNumber					
2.	Number of casualties being transported					
3.	Types of injuries involved and body part:					
	a. Fractures					
	b. Burns					
	c. Hemorrhaging					
	d. Other					
	e. Ambulatory: Yes No					
4.	Radiation contamination status:					
	a. Type of instrument used					
	b. BEFORE Decontamination AFTER Decontamination					
	Body Part C/M Distance C/M Distance					
	1					
	2					
	3					
	4.					
	c. Radioisotopes involved					
	i. Decontamination procedures used					
5.	Type of transporting vehicle					
6.	Time of departure from plant					

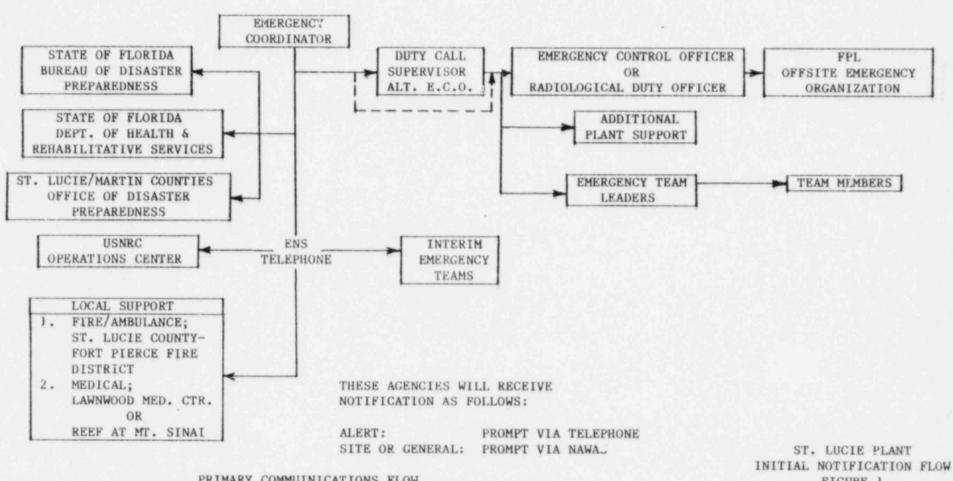
SIGNIFICANT EVENT REPORT

(for NRC Notification)

te	Time				
	Name Florida Power & Light St. Lucie Unit #1				
ller Na	ame				
1.	Description of Event				
	Classification of Event				
	Reactor Systems Status				
	Pressure Temperature Power level				
	Flow ECCS Operating/Operable (Pumps on)				
	PZR or RX Level Cooling Mode				
	Any Radioactive Release or Increased Release?				
	Path Stopped? Release Rate				
	Monitored?				
	Steam Plant Status: S/G Levels				
	Equip. Failures Feedwater Source/Flow				
	Electrical Dist. Status: Normal Off-site Power Sources				
	Available?				
	Major Busses/Loads Lost				
	Safeguards Busses Power Source				
	D/G Running? Loaded?				
	Personnel Casualties/Contamination?				
	Consequences of Event:				
	Actual and Potential Safety Hazards:				
	Tech. Spec. Violations?				
	State Notified? Press Release Planned?				
	Cause of Event:				
	Corrective Actions taken:				
	Dl anned:				
	Planned:				

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E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR



PRIMARY COMMUINICATIONS FLOW ALTERNATE COMMUNICATIONS FLOW FIGURE 1

E-PLAN IMPLEMENTING PROCEDURE NO. 3100021E, REV. 10 DUTIES AND RESPONSIBILITIES OF THE EMERGENCY COORDINATOR

Table 1 Guidelines for Protective Action Recommendations to Offsite Authorities

N=No Recommendation S=Shelter E=Evacuate

RADIUS FROM PLANT/(Sector Affected-Downwind)

Condition	2 Miles ·	5 Miles	10 Miles
Unusual Event	N	N	N
Alert	N	N	N
Site Area Emergency	N	N	N
General Emergency with the post Loca monitors reading 1000 MR or greater	E(360°)	E(90°)	N
General Emergency with the post Loca monitors reading 1000 MR or greater AND with actual containment failure	E(360°)	E(360°)	E(90°)
Actual releases (regardless of emergency class) with off-site doses estimated to be: a) Whole body: > 1 rem or Thyroid: < 5 rem	E(360°)	S(360°)	S(360°)
b) Whole body: > 1 rem to < 5 rem Thyroid: > 5 rem to < 25 rem	E(360°)	E(360°)	E(90°)
c) Whole body: > 5 tem Thyroid: > 25 rem	E(360°)	E(360°)	E(360°)

FLORIDA POWER & LIGHT COMPANY ST LUCIE UNIT NO. 1 EMERGENCY PLAN IMPLEMENTING PROCEDURE NO. 3100032E REVISION 1

1.0 Title:

ON-SITE SUPPORT CENTERS

2.0 Approval:

Reviewed by Facility Review Group		M	13rch 20+25	19 21
Approved by Facility Review Group	V. P.	Power	ResourcesMarch	16, 1981
Rev 1 Reviewed by Facility Review	ew Gro	up	May 28	19 2 (
Approved by Williams	V. P.	Power	Resources Lune	2, 1981

3.0 Scope:

3.1 Purpose

This procedure provides guidelines and responsibilities for activation and use of the on-site Technical and Operational Support Centers. This procedure describes interim facilities and shall be revised when permanent facilities are operational.

3.2 Discussion

The activities of plant management, technical, and engineering support personnel are an important part of the overall site response to an accident, and must be properly defined and logistically supported. The need for additional operational support personnel, other than those required and allowed in the Control Room, is also recognized as vitally important in properly responding to an emergency.

The intent of providing Technical and Operational Support Centers is to provide bases where post-accident emergency planning can be conducted and required operational support personnel can assemble for potential duty. Both centers will be in close communication with the Control Room via the PAX phone system, without disrupting, congesting, or confusing the activities in the Control Room by the physical presence of additional personnel.

3.0 Scope (cont):

3.3 Description

3.3.1 Technical Support Center (TSC)

The TSC is located in the training classroom area, adjacent to the Unit #1 Control Room. Emergency supplies available near the TSC are listed in Table 1. Documents and drawings available are listed in Table 2.

3.3.2 Operational Support Center (OSC)

The OSC is maintained in the first floor maintenance area of the Service Building.

3.3.3 Communications Equipment

The Control Room contains necessary telephone and public address systems for communications within and outside the plant. The Control Room also contains NAWAS and Local Government Radio for communications with state and county agencies. Use of these systems is described in the Appendix.

The TSC contains necessary telephones to communicate with the Control Room, OSC, the General Office Emergency Center, the Emergency Operations Facility and outside agencies. It also contains an ENS dedicated telephone to the NRC Operations Center and the NRC Region I office.

3.4 Authority

This procedure implements the St Lucie Plant Radiological Emergency Plan.

4.0 Precautions:

- 4.1 The TSC and OSC shall be activated upon the direction of the Emergency Coordinator or his designated alternate only.
- 4.2 Radiological conditions in the on-site Technical and Operational Support Center shall be monitored when required to be in use. Radiation levels in the TSC should be monitored using available (control room) portable instruments, in addition, Health Physics Department shall provide an NMC portable monitor to the Technical Support Center for radioiodine monitoring. This monitor can also be used for monitoring the control room.
- 4.3 The Emergency Coordinator shall recommend a suitable location other than the designated areas if radiological conditions warrant such actions.

5.0 Responsibilities:

- 5.1 The Emergency Coordinator is responsible for activating the TSC, activating the OSC, and arranging for staffing through the Duty Call Supervisor.
- 5.2 The TSC Supervisor will be the most senior person available to remain at the TSC of the following:

Primary:

Alternate 1: Operations Supervisor

Alternate 2: Operations Duty Call Supervisors

Alternate 3: Off-Duty Nuclear Plant Supervisors

He is responsible for supervising TSC activities, reporting to the Emergency Coordinator, and communicating with the Emergency Operations Facility.

- 5.3 Each department head or designated alternate is responsible for reporting to the TSC when called.
- 5.4 Those reporting to the Operational Support Center are responsible for being prepared to carry out support functions designated by the Emergency Coordinator.
- 5.5 The Emergency Coordinator has the overall responsibility for the conduct of emergency operations and activities and should work closely with the TSC to ensure all information is used in making accident recovery decisions.
- 5.6 The QC department is responsible for ensuring that records, documents, and prints are maintained in the TSC or are available for immediate use in the Document Control vault.

6.0 References:

- 6.1 St Lucie Plant Radiological Emergency Plan
- 6.2 E-Plan Implementing Procedure 3100021, Duties of the Emergency Coordiantor.

7.0 Records:

A record of actions taken at the TSC shall be maintained by the TSC Supervisor.

8.0 Instructions:

8.1 Activation

The Emergency Coordinator shall activate the TSC and the OSC for an emergency condition classified as Alert, Site Area Emergency, or General Emergency.

8.2 Staffing

The Emergency Coordinator shall arrange for staffing through the Duty Call Supervisor, Table 3 identifies the TSC staffing requirements.

8.3 Use of the Technical Support Center

The TSC shall be used to provide plant management and technical support to plant operations personnel during emergency conditions. A primary task shall be to relieve the Emergency Coordinator of off-site communications to the state and local agencies, NRC and the FPL Off-Site Emergency Organization. The TSC staff shall provide technical support as requested by the Emergency Coordinator and shall direct itself toward determining current and projected plant status and providing indepth diagnostic and engineering assistance, as required.

8.4 Use of the Operational Support Center

The Emergency Coordinator shall assign available plant staff not needed in the TSC or Control Room to the OSC. The OSC Supervisor (senior staff member present) shall maintain communications with the TSC and report as directed by the Emergency Coordinator or TSC Supervisor.

8.5 Deactivation

It is the responsibility of the Emergency Coordinator or his designee to deactivate and secure the TSC and OSC when the emergency condition no longer exists.

8.6 Alternate OSC

In the event that the OSC becomes untenable, the Emergency Coordinator shall designate an alternate location.

TABLE 1

RADIOLOGICAL EMERGENCY EQUIPMENT ST LUCIE PLANT CONTROL ROOM STORAGE LOCKER (ALSO FOR TSC USE)

- 1. Coveralls
- 2. Hood
- 3. Gloves
- 4. Shoe Covers
- 5. Self Contained Breathing Apparatus
- 6. Full Face Respirators and Filters
- 7. Pocket Dosimeters
- 8. Dosimeter Chargers
- 9. TLD's
- 10. Portable Count Rate Instrumentation
- 11. Portable Dose Rate Instrumentation
- 12. Contamination Smears
- 13. Envelopes
- 14. Radiation Tape/Rope
- 15. Radiological Signs
- 16. Industrial First Aid Kit
- 17. Step-Off Pads
- 18. Plastic Bags

TABLE 2

EMERGENCY DATA, DRAWINGS AND DOCUMENTS MAINTAINED AT THE TSC ST LUCIE PLANT

- 1. Plant Technical Specifications
- 2. Plant Operating Procedures
- 3. Emergency Operating Procedures
- 4. Emergency Plan and Implementing Procedures
- 5. Final Safety Analysis Report
- 6. Drawings, Schematics and Diagrams

TABLE 3

TECHNICAL SUPPORT CENTER STAFFING ST LUCIE PLANT

TSC Supervisor

Primary: Operations Superintendent Alternate 1: Operations Supervisor

Alternate 2: Operations Duty Call Supervisors
Alternate 3: Off-duty Nuclear Plant Supervisors

Technical Supervisor

Health Physics Supervisor

Chemistry Supervisor

Reactor Engineering Supervisor

Maintenance Superintendent

Quality Control Supervisor

Support Staff as determined by these supervisors

APPENDIX

COMMUNICATIONS EQUIPMENT

A. Use of the NAWAS System

- The NAWAS is used for announcing the initial warning to the State Warning Point at the Bureau of Disaster Preparedness (BDP) and St Lucie Disaster Preparedness Coordinator of a SITE AREA or GENERAL EMERGENCY.
- The NAWAS is a direct, protected telephone land line with the handset installed in the Control Room.
- 3. Picking up the handset from its cradle activates a response at the State Warning Point and St Lucie and Martin County Disaster Preparedness Offices. The Nuclear Plant Supervisor (Emergency Coordinator) will advise the personnel on the other end of the system, in a coded message, of the conditions at the plant. He then places the handset back on its cradle, as this is the only use for it during an emergency. His message will activate the required state and local emergency teams.
- 4. The procedure for reporting an emergency via NAWAS is summarized in E-Plan Implementing Procedure 3100021E, Duties of Emergency Coordinator, and described in detail below.
 - A. In the event of a SITE AREA EMERGENCY or GENERAL EMERGENCY incident, the following reporting procedure will be used:
 - (1) The initial report will be handled over NAWAS by code.
 - (2) The followup data will be handled by LER or telephone. Provide the data on Emergency Information Checklist, E-Plan Implementing Procedure 3100021E, Duties of Emergency Coordinator.

/R1

(3) The following code name will be used to identify the geographical location of the St Lucie Plant:

/Rl

St Lucie Plant

ST. LUCIE ONE

Page 9 of 10

EMERGENCY PLAN IMPLEMENTING PROCEDURE NO. 3100032E, REV 1 ON-SITE SUPPORT CENTERS

APPENDIX (cont)

A. Use of the NAWAS System (cont)

5. Classifications

The classes of Radiological Emergencies are described in E-Plan Implementing Procedure 3100022E, Classification of Emergencies.

6. Low Population Zone

- A. The Low Population Zone will extend outward from each nuclear power plant for a radius of five (5) miles, and shall be segmented into $22-1/2^{\circ}$ arcs and will be lettered A through R (excluding I and 0) for the 360° of the compass.
- B. Each area is divided into SECTORS beginning at the center and extending outward in mile increments One through Five. Contaminated area, extending outward from the nuclear plant, will be located and reported as discussed in E-Plan Implementing Procedure 6, Radiation Release and Dose Projection.

B. Use of the Local Government Radio (LGR)

- The LGR is used, unless inoperative or unavailable, for maintaining communications with various state and local disaster preparedness personnel. Messages are transmitted and received on frequencies allocated by the state Bureau of Disaster Preparedness. After initial NAWAS notice, detailed reports shall be by LGR.
- 2. The LGR control unit is installed in the Control Room.

C. Use of the ENS (OPX) Telephone Network

- 1. Within one hour of the time that the Emergency Plan or any part of the Emergency Plan is implemented the NRC shall be notified and an open, continuous communications channel established using the NRC ENS circuit (red phone). The Nuclear Plant Supervisor (Emergency Coordinator) shall be responsible for assuring that this notification is made.
- To initiate a signal to the NRC Operations Center, remove the handset from the receiver.
- 3. When the NRC Duty Officer answers, identify yourself as "Florida Power & Light, St Lucie Plant".

APPENDIX (cont)

- C. Use of the ENS (OPX) Telephone Network (cont)
 - 4. Make the notification.
 - 5. Maintain the line open and manned until allowed to terminate the call by the NRC Duty Officer.

FLORIDA POWER & LIGHT COMPANY ST. LUCIE PLANT UNIT NO. 1 EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

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RE-ENTRY

2. APPROVAL:

Reviewed	by Facility Review Group	July 25,	19 75
Approved	by K. N. Harris Plan	it Manager July 29,	19 75
Revision_ Approved	3 Reviewed by Facility Re by LLGum Mr. P. Powe	er Resource March 20+	25 19 81 19 81
	by Reviewed by Facility Re		

3.0 SCOPE:

3.1 Purpose

This procedure provides guidelines for the selection of the Re-entry Team members and provides instructions to follow during re-entry.

3.2 Discussion

- 3.2.1 Re-entry into an evacuated area shall be made only when authorized by the Emergency Coordinator as follows:
 - To ascertain that all personnel who were in the affected area have been evacuated and to search for unaccounted for personnel.
 - To rescue any injured or trapped personnel from the affected area.
 - To perform operations which may decrease the severity of the emergency.
 - 4. To determine the nature and extent of the emergency and the radiological conditions.
 - To establish definite personnel exclusion area boundaries.

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

3.0 SCOPE (Continued

3.2 (Continued)

- 3.2.2 The Emergency Coordinator should select individuals with appropriate qualifications in First Aid, Operations, Health Physics, and Maintenance for the Re-entry Team.
- 3.2.3 The initial entry of the Re-entry Team, and all subsequent entries, until radiation areas have been properly marked, shall take place under the supervision of the Radiation Team Leader.
- 3.2.4 The Re-entry Team Leaders should be fully briefed concerning the nature of the emergency. Information for these briefings will be obtained from the applicable available sources including current operating records, interviews with evacuated employees, and surveys conducted by the Radiation Team.

3.3 Authority

This procedure implements the St. Lucie Plant Radiological Emergency Plan.

4.0 Precautions:

- 4.1 There shall be no re-entry into the area affected by the emergency after an emergency evacuation unless authorized by the Emergency Coordinator.
- 4.2 All Re-entry Team members shall wear protective clothing, dosimeters, respiratory devices, and other protective devices as specified by the Radiation Team Leader.
- 4.3 The team members should not deviate from the planned route, unless required by un-anticipated conditions such as rescue, performing an operation which would minimize the emergency conditions, etc.
- 4.4 If the monitored dose rates encountered during the re-entry exceed the limits set by the Emergency Coordinator and the Radiation Team Leader, the Re-entry Team shall return to the area from which they were dispatched.

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

4.0 Precautions: (Continued)

- 4.5 The following guidelines for emergency exposure of personnel shall be followed during the re-entry operation:
 - 4.5.1 The Emergency Coordinator and the Re-entry Team
 Leaders shall make every effort to minimize re-entry
 personnel exposures, utilizing radiation survey and
 monitoring devices, protective clothing, breathing
 apparatus, and other special equipment as required.
 - 4.5.2 Under emergency conditions not requiring action to prevent serious injury or a catastrophic incident, Re-entry Team personnel exposure in excess of 10 CFR 20 limits may be authorized by the Emergency Coordinator, but shall not exceed 5 Rem whole body exposure.
 - 4.5.3 A planned emergency dose to prevent serious injury or to prevent destruction of equipment which could result in serious injury should not exceed 12 Rem whole body dose.
 - 4.5.4 Under emergency conditions where immediate action is necessary to prevent serious injury or a catastrophic incident, Re-entry Team members' exposures authorized by the Emergency Coordinator, with the consent of the individuals to be exposed, should not exceed 25 Rem whole body exposure, except for life saving actions.
 - 4.5.5 For life saving actions, an individual may receive a dose of up to 75 Rem whole body exposure.
- 4.6 A Re-entry Team shall consist of at least two persons.

5.0 Responsibilities:

- 5.1 The Emergency Coordinator has the responsibility for authorizing re-entry into an area after it has been evacuated. It shall be his responsibility to select qualified personnel for the Re-entry Team and make clear what their assigned duties are prior to re-entry.
- 5.2 The Radiation Team Leader shall be responsible for the supervision of the initial entry into an evacuated area, and for all subsequent entries until radiation areas have been properly marked and a safe route determined. He shall also make recommendations to the Emergency Coordinator as to the expected doses the Re-entry Team personnel will be subjected to and the feasibility of attempting re-entry into a given area.

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

5.0 Responsibilities: (Continued)

5.3 It shall be the responsibility of the person appointed Re-entry Team Leader to strictly follow the orders given to him by the Radiation Team Leader and at all times to protect the members of the Re-entry Team from possible injury and excessive radiation exposure.

6.0 References:

- 6.1 St. Lucie Plant Radiological Emergency Plan
- 6.2 St. Lucie Plant Radiation Protection Manual

7.0 Records and Notifications:

Detailed records of all significant actions of the Re-entry Team shall be recorded by the Emergency Coordinator.

Page 5 of 6

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

8.0 Instructions:

- 8.1 The Emergency Coordinator or the Radiation Team Leader or their designee shall:
 - 8.1.1 Select at least two qualified persons to form a Re-entry Team. One of the members shall be qualified in Health Physics procedures.
 - 8.1.2 Brief the team members concerning the nature of the emergency and the possibility of hazards being present.
 - 8.1.3 Instruct the team members as to what is required of them while in the emergency area. This may consist of the following:
 - Search for injured, trapped or unaccounted for personnel.
 - 2. Operate equipment as needed.
 - 3. Determine the nature and extent of the emergency and radiological conditions.
 - Establish definite personnel exclusion areas and access routes.

8.2 The Radiation Team Leader shall:

- 8.2.1 Evaluate the emergency to determine the possible dose rates and stay times. Refer to Step 4.5 of this procedure for guidelines.
- 8.2.2 Supervise the initial entry of the Re-entry Teams into the area of the emergency and any subsequent entries until readiation areas have been properly marked.
- 8.2.3 Based on the information available, select a route for the Re-entry Teams to follow into the affected area.
- 8.2.4 Assure that all Re-entry personnel are properly equipped with protective clothing, high range dosimeters, low range dosimeter, TLD, respiratory devices, and other protective equipment as required.
- 8.2.5 Specify self-monitoring and decontamination procedures for the Re-entry Teams.

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100027E, REV. 4

8.0 Instructions: (Continued)

- 8.3 The Re-entry Team members shall:
 - 8.3.1 Wear protective clothing and other protective devices as specified by the Radiation Team Leader.
 - 8.3.2 Unless unanticipated conditions are encountered, follow the pre-planned route into the affected area, and perform the assigned jobs quickly and safely.
 - 8.3.3 Monitor the dose rate along the route followed, and identify high radiation areas.
 - 8.3.4 Frequently observe their dosimeters and withdraw to a safe area before established dose limits are received.
 - 8.3.5 When re-entry is completed, follow the self-monitoring and personnel decontamination procedures as specified by the Radiation Team Leader.
 - 8.3.6 Record and report to the Emergency Coordinator or his designee the radiological conditions and extent of damage in the affected area.
- 8.4 The Emergency Coordinator and the Recovery and Restoration Team Leader shall evaluate the existing conditions and plan further actions as required.

1.0 TITLE:

CLASSIFICATION OF EMERGENCIES

2.0 APPROVAL:

Reviewed by Fa	cility Review Group	August 8,	1975
Approved by 1	C. N. Harris Plan	nt Manager August	19_75
Revision 8 R Approved by	William W V.P. P.	iew Group March 20+ wr. Res. March	26 1981
Revision 9 Red Approved by	Hewed by Facility Reviews	w Group May 28 wr. Res. June	1981

3.0 SCOPE:

3.1 Purpose

This procedure provides instructions on the classification of emergencies at St. Lucie Plant.

/R9

3.2 Discussion

In order of increasing seriousness, these are:

Unusual event Alert Site area emergency General emergency

A gradation is provided to assure fuller response preparations for more serious conditions.

3.3 Authority

This procedure implements the St. Lucie Plant Emergency Plan.

3.0 SCOPE: (Continued)

3.4 Definitions

- 3.4.1 Unusual Event This classification is represented by Offnormal events or conditions at the plant for which no significant degradation of the level of safety of the plant has
 occurred or is expected. Any releases of radioactive material
 which may have occurred or which may be expected are minor and
 constitute no appreciable health hazard.
- 3.4.2 Alert This classification is represented by events which involve an actual or potential substantial degradation of the level of safety of the plant combined with a potential for limited uncontrolled releases of radioactivity from the plant.
- 3.4.3 Site Area Emergency This classification is composed of events which involve actual or likely major failures of plant functions needed for protection of the public combined with a potential for significant uncontrolled releases of radioactivity from the plant.
- 3.4.4 General Emergency This classification is composed of events which involve actual or imminent substantial core degradation and potential loss of containment integrity combined with a likelihood of significant uncontrolled releases of radioactivity from the plant.

4.0 Precautions

4.1 Conflicting Information

When apparently conflicting information is available, the condition shall be classified at the most serious level indicated.

4.2 Judgmental Decision

If, in the judgment of the Nuclear Plant Supervisor (Emergency Coordinator), a situation is more serious than indicated by instrument readings or other parameters, the emergency condition shall be classified at the more serious level.

5.0 Responsibilities

5.1 Plant Personnel

All plant personnel are required to promptly report the existence of an emergency condition to the Nuclear Plant Supervisor by the fastest means possible.

5.2 Nuclear Plant Supervisor

- 5.2.1 The Nuclear Plant Supervisor shall promptly classify abnormal situations into one of the four defined categories.
- 5.2.2 If the diagnosis indicates that the condition is classified as an Alert, Site Area Emergency, or General Emergency the Nuclear Plant Supervisor shall declare an emergency.
- 5.2.3 If an emergency has been declared the Nuclear Plant Supervisor shall become the Emergency Coordinator and retain this position until relieved.

6.0 References:

- 6.1 St. Lucie Plant Radiological Emergency Plan
- 6.2 E-Plan Implementing Procedure 3100029E, Duties of Individual Discovering Emergency Condition
- 6.3 E-Plan Implementing Procedure 3100021E, Duties of Emergency Coodinator

7.0 Records and Notification

The basis for classifying an emergency condition shall be recorded.

/R9

8.0 Instructions:

- 3.1 The Nuclear Plant Supervisor shall initially classify a situation within 15 minutes of the time he has become aware of it. The initial classification shall be made on the basis of readily available observations and shall not rely on laboratory analyses, measurements, or calculations which would require more than 15 minutes to perform.
- 8.2 If subsequent information of a more detailed nature (e.g. sampling results) becomes available after the initial classification has been made, the event shall be reclassified by the Emergency Coordinator if appropriate.
- 8.3 The Nuclear Plant Supervisor shall classify events in accordance with the attached Classification Tables. The event shall be classified by matching the actual situation to the one most closely approximating it in the tables.
- 8.4 Classification Tables are provided for the following categories:
 - 1) Primary Depressurization

Miscellaneous Abnormal Primary Leak Rate Abnormal Primary/Secondary Leak Rate Loss of Secondary Coolant

4) Accident Involving Fuel

Fuel Element Failure Fuel Handling

2) Abnormal Radiation, Contamination of Effluent Release Values

Uncontrolled Effluent Release High Radiation Levels in Plant

- 3) Fires Lasting More Than 10 Minutes
- 8) Degradation of Control Capabilities

Engineered Safety Features Loss of Containment Integrity Loss of Alarms

8.0 Instructions:

8.4 (Continued)

9) Hazards to Station Operation

Aircraft Toxic or Flammable Gas Onsite Explosion Missile Turbine Failure

7) Electrical Malfunctions

Loss of Power

- 10) Security
- 5) Natural Emergencies

Earthquake Hurricane Tornado Water Level Abnormal

6) Miscellaneous Events

Contaminated Injury Abnormal Temperature Abnormal Shutdown

Page 6 of 20

PRIMARY DEPRESSURIZATION Page 1 of 4

EVENT	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
MISCELLANEOUS	A) Unplanned initiation of ECCS (Emergency Core Cooling System) 1. ECCS pumps running as indicated by motor amps, AND 2. ECCS header isolation valves open as indicated by valve position indication lights, OR B) Safety or relief valve fails to close 1. Reactor coolant system (ECS) a. Indication of flow through pressurizer relief valves as indicated on the acoustic valve flow monitor, AND b. ECS pressure drops to (1600 psia. 2. Main Steam System a. Unusual decrease in pressurizer pressure and level with decreasing Tave AND b. Abnormal drop in steam generator pressure to less than 500 psia, AND c. Visual or audible verification or steam relief lifting.			Loss of 2 of 3 fission product barrier with potential for loss of the third (Any two of the following conditions exist and the third is imminent.) 1. Confirmed fuel clad damage. 2. Confirmed LOCA 3. Confirmed containment integrity breached and cannot be restored.
CTION	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.			1. Complete actions listed on GENERAL EMERGENCY CHECKLIST.

Page 7 of 20

PRIMARY DEPRESSURIZATION Sheet 2 of 4

EVENT	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
ABNORMAL PRIMARY LEAK RATE	BCS leakage greater than allowed by Technical Specifications A. RCS water inventory balance indicates 1. greater 1 gpm unidentified leakage, OR 2. greater than 10 gpm identified leakage, OR B. Inspection reveals any RCS pressure boundary leakage,	RCS leak greater than 50 gpm - primary to atmosphere (1) Charging/letdown mismatch backed up by RCS water inventory balance indicating > 50 GPM and <132 gpm leakage, AND (2) Containment or plant vent radiation process monitor reading above normal.	LOCA greater than capacity of charging pumps (1) Unusual decrease in pressure vith constant T(ave), AND (2) Makeup rate greater than capacity of 1 charging pumps (132) gpm) AND (3) Containment pressure > 2 psig, or containment radiation monitors indicate above normal values.	A release has occurred or is in progress resulting in: LOCA Dose Calculation EPIP 3100033E Appendix B worksheet values in excess of 1 R/HR (whole body) or an integrated dose of 5R (thyroid)
ACTION	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.	1. Complete actions listed on ALERT CHECKLIST	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST	1. Complete actions listed on GENERAL EMERGENCY CHECKLIST

Page 8 of 20

PRIMARY DEPRESSURIZATION

GENERAL EMERGENCY		
SITE AREA EMERGENCY	Rapid failure of steam Renerator tubes with a loss of off-site power (> charging pp capacity) (1) Unusual decrease in pressure and level with constant T(ave), AND (2) Above normal readings on radiation process monitors for steam generator blowdown and condensate air elector, AND (3) Loss of the 6.9 kV and 4.16 kV busses (1A1, 1A2, 1B1, 1B2), AND (4) Simultaneous unusual increase in one steam generator's pressure and level.	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST.
ALEKT	Rapid gross failure of one steam generator tube with loss of offgite power (w/4 charging pp capacity) 1. Unusual decrease in pressurizer pressure and level with constant T aye., followed by; steam generator blowdown and condensate air ejector readings, steam generator blowdown and condensate air ejector readings, steam generator blowdown and condensate air ejector readings, steam generator from process monitor readings, and process (iAi, iA2, iBi, iB2) Rapid failure of steam generator tubes (charging pp capacity) 1. Unusual decrease in pressurizer pressure and level, with constant Taye, AND; constant Taye, AND; 2. Simultaneous unusual increase in ore steam generator's pressure and level followed by; a Above normal steam generator readings. 7. Above normal steam generator readings.	1. Complete actions listed on ALERI CHECKLIST
UNUSUAL EVENT	A. Greater than 1 GPM unidentified leakage AND and Condensate air elector process monitors and condensate air elector process monitor reading above normal or increasing.	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.
EVENT	TO SECONDARY LEAK RATE	ACTION

PRIMARY DEPRESSURIZATION Sheet 4 of 4

GENERAL EMERCENCY		
SITE AREA EMERGENCY	Major steam leak with greater than 50 gpm primary/secondary leak rate and fuel damage indidated 1. Unusual decrease in pressure and level with decreasing Tave, AND; 2. Abnormal drop in main steam or steam generator pressure to < 500 psia, AND; 3. Steam generator blowdown and condensate air ejector railation process monitors indicate above normal, (OR, known pri-sec leak of > 50 gpm). 4. Fuel damage as indicated by last known primary sample.	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST.
ALERT	Halor steam leak with greater than 10 gpm primary/secondary leak rate 1. Unusual decrease in pressurizer pressure and level, with decreasing Tave, AND; 2. Abnormal drop in main steam or steam generator pressure to < 500 psia, AND; 3. Steam generator blowdown and condensate air ejector radiation process monitors inducate above normal, (OR, known pri-sec leak of > 10 gpm).	ALERT CHECKLIST.
UNISUAL EVENT	Rapid depressurization of sacondary side with no primary to secondary leakage. 1. Unusual decrease in pressurize pressure and level with decreasing Tave AND 2. Simultaneous abnormal drop in Main Steam or steam generator pressure to less than 500 psia.	. Complete actions listed on the UNUSUAL EVENT CHECKLIST.
EVENT	COOLANT COOLANT	ACTION 1.

ACCIDENT INVOLVING FUEL

EVENT	unusual. Eyent	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
FUEL ELEMENT FAILURE	Fuel damage indication A. Letdown todine process monitor alarm followed by B. Confirmed RCS sample indicating 1. Coolant activity greater than the Tech Spec limit for fodine spike Figure 3.4-1. AND 2. Coolant activity greater than 100/E uci/gram specific act	Severe loss of fuel cladding including coolant pump failure leading to gross fuel failure (1) Letdown iodine process monitor alarms, AND (2) RC\$ 1-131 activity > 275 uC4/ml	Core damage with inadequate core cooling determined by: (1) RCS 1-131 activity > 275 uCi/m1, (2) RCS temperature within 20° of T(sat), AND (3) Loop AT >44°F	A release has occurred OR is in progress resulting in: Loca Bose calculation EPIP 3100033E Appendix B worksheet values in excess of 1 B/HR (wholebody) or an integrated dose of 5E (thyroid)
FUEL HANDLING ACCIDENT		Fuel handling accident which results in the release of radio-activity to containment or Fuel Handling Building: (1) Direct information from fuel handling personnel indicating that an irradiated fuel assembly has been damaged and gas bubbles are escaping, AND (2) Associated area or process monitor channels are alarming.	Major damage to spent fuel in containment or Fuel Handling Building (1) Step increase in the readings of radiation sonitors in the plant vent and/or in the fuel handling building, AND (2) Damage to more than one fuel assembly, OR (3) Uncovering of more than one spent fuel assembly in the spent fuel pit.	
ACTION	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.	1. Complete actions listed on ALERT CHECKLIST	1 Complete actions listed on SITE AREA EMERGENCY CHECKLIST	1. Complete actions listed on GENERAL EMERGENCY CHECKLIST.

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Page 11 of 20

ABNORMAL RADIATION, CONTAMINATION OR EFFLUENT RELEASE VALUES

EVENT	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
UNCONTROLLED EFFLUENT RELEASE	Radiological effluent Tech. Specs. Limits exceeded 1. Plant effluent monitor(s) exceed slarm setpoint(s) followed by 2. Confirmed analysis results for gaseous or liquid release which exceeds Technical Speci- fication limits.	A release has occurred or is in progress that is 10 times the T.S. limit (as shown by sample/ survey)	A release has occurred or is in progress resulting in: LOCA Dose Calculation EPIP 1100033E Appendix B worksheet values in excess of 50 ME/HE (whole body) 250 ME/HE (thyroid) for 1/2 hour OE 500 ME/HE (whole body) 2500 ME/HE (thyroid) for two min at the site boundry.	A release has occurred or is in progress resulting in: LOCA Dose Calculation EPIP 3100033E Appendix B worksheet values in excess of 1 R/HR (whole body) or an integrated dose of 5 E (thyroid)
HIGH RADIATION LEVELS IN PLANT		High radiation levels or high airborne contamination which indicates a severe degradation in the control of radioactive materials (1) Installed radiation monitoring stations indicate abnormally high radiation levels, OR (2) Installed airborne particulate or iodine activity monitors indicate abnormally high levels, AND (3) Levels > 1000 times normal are confirmed by area surveys and/or analysis of grab samples.		
ACTION	1. Complete actions listed on UNUSUAL EVENT CHECKLIST	1. Complete actions listed on ALERT CHECKLIST	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST	1. Complete actions listed on GENERAL EMERGENCY CHECKLIST

FINES LASTING MORE THAN 10 MINUTES

Page 12 of 20

alergy avolving Uncontrolled fire, potentially affecting safety systems and requiring off-site support. requiring off-site support.	red on 1. Complete actions listed 1. Complete actions listed on ALERT CHECKLIST on SITE AREA ENERGENCY
Uncontrolled fire not involving a safety system, but raquifing off-site support.	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST

DEGRADATION OF CONTROL CAPABILITIES Sheet 1 of 2

	54 5 <u>4</u>	2 2	
GENERAL EMERGENCY			
SITE AREA EMERGENCY	Loss of any function or system which precludes placing the plant in Hot Shutdown, OR Control Room is evacuated (for other than drill purposes) and shutdown control gannot be established locally at the Bot Shutdown Control Panel within 15 minutes.	All annunctator alarms lost 215 minutes with plant not in cold shutdown; OR Plant transfent occurs with all alarms lost.	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST.
ALERT	4. Loss of functions needed for cold shutdown, OR 2. Pailure of the Reactor Protection System to bring the reactor subcrittcal when needed, OR 3. Evacuation of control room (for other than drill purposes) with shutdown control established locally at the Hot Shutdown Control Panel.	All amunciators lost.	1. Complete actions listed on ALERI CHECKLIST.
HNUSHAL EVENT	1. A safety features actuation system functional unit shown in Technical Specification table 3.3-3 becomes inoperable per fechnical Specification 3.3.2.4 AND requires planf shuddown, OR 2. The fire suppression system or a portion thereof becomes inoperable per Technical Specification 3.7.11.4 AND requires plant shutdown.	Significant loss of effluent monitoring capability, meterolossical monitoring instrumentation communications, indication and alarm panels, etc., which impairs ability to perform accident or emergency assessment.	1. Complete actions listed on UNUSUAL EVENT CHECKLIST.
EVENT	ENCINEERED SAFETY FEATURES/REACTOR PROTECTION SYSTEM SYSTEMS TO PLACE PLANT IN COLD SHUT-DOWN/ CONTROL ROOM OPERATION AND FIRE PROTECTION SYSTEM	LUSS OF ALARHS	ACTION

Page 14 of 20

DECRADATION OF CONTROL CAPABILITIES

_		7
GENERAL EMERGENCY		
SITE AREA EMERGENCY		
ALERT		
UNUSUAL EVENT	Loss of containment integrity requiring shutdown by Tech. Speca. A. Penetrations required to be closed during accident conditions are NOT: 1. Capable of being closed by the ESFAS, OR 2. Closed by manual yalyes, blind flanges, or deactivated succent in their closed positions except as provided in Table 3.6-2 of Technical Specification B. An equipment latch is not close per Technical Specification 3.6.1.3; OR A scaling mechanism associated with a penetration (e.g., welds, believe, or ortings) becomes inoperable.	1. Complete actions listed on UNUSUAL EVENT CHECKLIST.
EVENT	MENT INTEGRITY	ACTION

Page 15 of 20

HAZARDS TO STATION OPERATION

	Visual sighting of direrule creek	Atteraft cresh onsite damaging	Aircraft crash on site damaging vital plant systems	GENERAL EMERCENCY
ONSITE	wettvity over facility, Visual or gudio indication of an explosion near or on-site,	Damage to facility by explosion which affects plant operation.	or atructures. Damage to safe shurdown equipment from missiles or explosion	
TOXIC OR FLAM- MABLE GAS	indication (visual or otherwise) of a near or on site toxic or flammable gas release.	Entry of toxte or flammable gas onto areas potentially affecting plant operation.	Alarm indication is received on Detecto-Chlor or toxic or flammable gas has diffused into vital areas.	
MISSILES		Visual or audible indication of missile impact on plant structures.	Damage to safe shutdown equipment from missiles or explosion.	
TURBINE FALLURE		Visual indication that the turbine casing has been penetrated by blading.		
J-100 G-100 G-100	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.	1. Complete actions listed on ALERT CHECKLIST.	1. Complete actions listed on SITE AREA EMERCENCY CHECKLIST.	

ELECTRICAL MALFUNCTION

EVENT	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
LOSS OF POWER	Loss of off-site power or loss of on-site AC power capability 1. Turbine generator trips with plant startup transformers unavailable for service: OR, 2. Loss of voltage on both 1A3 AND 1B3 4.16 KV busses for more than 15 seconds.	Loss of offsite power and loss of all onsite AC power 1. Turbine generator trip, with plant startup transformers non-functional, AND; 2. Failure of both emergency diesel generators to start or synchronize OR Loss of all onsite DC power Drop in A and B DC Bus voltages to § 70 volts.	Loss of offsite power and loss of onsite AC power > 15 minutes 1. Turbine generator trips with plant startup transformers unavailable for service, AND; 2. Sustained failure of both emergency diesel genera- tors to start or synchronize for > 15 minutes. OR Loss of all vital onsite DC power for > 15 minutes Sustained drop in A and B DC bus voltages to 70 vdc for > 15 minutes.	
CTION	1. Complete actions listed on UNUSUAL EVENT CHECKLIST.	1. Complete actions listed on ALERT CHECKLIST.	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST.	

SECURITY EMERCENCIES

Page 17 of 20

KERGENCY	A successful takeover of the plant including the Control Room has occurred.	CENERAL EMERGENCY CHECKLIST
GENERAL EMERGENCY	A successful to plant includis Room has occur	Complete ac
SITE AREA EMERGENCY	A security emergency has progressed to the point that takeover of the plant is probable	1. Complete actions listed on SITE AREA EMERGENCY CHECK- LIST.
ALEKT	A security emergency has been called by the security force as defined in the Safeguards Contingency Plan.	1. Complete actions listed on ALERT CHECKLIST.
UNUSUAL EVENT	A security alert has been called by the security force in response to one or more of the frequents of threat. 3. Bomb threat. 3. Civit disturbance. 4. Sabotage attempt. 5. Sabotage attempt. 6. Internal disturbance. 7. Vital area intruston. 8. Security force strike.	1. Complete actions listed on UNUSUAL EVENT CHECKLIST,
EVENT		ACTION

Page 18 of 20

NATURAL EMERGENCIES

ALEKT SITE AREA EMERGENCY	An earthquake occurs which registers >SSE (0.5g) on NOAA registers >SSE on NOAA seismic equipment	Notification by the Weather Bureau of the approach of a bureau of the approach of a hurricane with winds up to hurricane with winds y design design basis (120 mph) levels basis (120 mph) levels	Any tornado striking facility	Flood, low water, hurricane surge, OR other abnormal water level conditions cause the storm drainage system to be exceeded. Flood, low water, hurricane surge OR other abnormal water level conditions cause vital equipment to fail.	Complete actions listed 1. Complete actions listed on ALERT CHECKLIST CHECKLIST CHECKLIST
UNUSUAL EVENT	An earthquake has occurred as indicated on earthquake force represent sention SMI-42-11 (post accident sentence) and annunciated on RTGB 106 (\$36 and \$46).	Notification by NOAA (Weather N Bureau) that a hurricane watch ta in effect.	Notification by NOAA (Weather A Bureau) that a formado watch is in effect.	50-year flood or low water, buttlcane surge OR other abnormal awater level conditions 1. Notification by NOAA (Weather Bureau) that abnormal water level conditions are expected or are occuring, or 2. Visual siterings by station personnel that water levels are approaching storm drainage system capacity.	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.
EVENT	EARTHQUAKE	HURRICANE	TORNADO	MATER LEVEL ABNORMAL	ACTION

MISCELLANEOUS EVENTS

Page 19 of 20

SITE AREA EMERGENCY GENERAL EMERGENCY			
ALEKT			
UNUSUAL EVENT	Abnormal coolant temperature AND/OR pressure, abnormal fuel temperature 1. Subcocling mergin monitor indicates less than 20°g sub- cooling, OR 2. Highest hot leg temperature these thot leg temperature saturation temperature, OR 3. Plant incore thermocouples indicate abnormal fuel temperatures.	Any contaminated individual is transported off site to a hospital for treatment of injuries.	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.
EVENT	ABNORMAL TEMPERATURE/ PRESSURE	CONTAMINATED INJURY	ACTION

Page 20 of 20

Sheet 2 of 2

CLASS	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY					
OTHER CONDITIONS REQUIRING IN- REASED AWARE- RESS	1. A plant shutdown is required by Technical Specifications; OR 2. The plant is shut down under abnormal conditions (e.g., exceeding cooldown rates or primary system pipe reacks are found during operation);	The Technical Support Center AND/OR Near Site Emergency Operations Facility are activated for other than drill purposes.	1. The Emergency Centers are activated; AND. 2. Monitoring Teams are mobilized; AND. 3. A precautionary public notification is made concerning an shnormal plant condition for other than drill purposes.	An event resulting in escalation of the Emergency Classification to General Emergency with Imminent Substantial Gore Damage and potential for release of large amounts of radioactivity in a short period of time; such as: 1. LOCA with failure of ECCS. 2. Loss of Secondary Heat Sink. 3. Sustained station blackout with loss of secondary heat sink. 4. Failure of containment heat removal systems in the later stages of an accident resulting in loss of containment.					
ACTION	1. Complete actions listed on the UNUSUAL EVENT CHECKLIST.	1. Complete actions listed on ALERT CHECKLIST.	1. Complete actions listed on SITE AREA EMERGENCY CHECKLIST.	1. Complete actions listed on GENERAL EMERGENCY CHECKLIST.					

1.0 Title:

LOSS OF COOLANT ACCIDENT DOSE CALCULATIONS

2.0 Approval:

Approved by Millim W Vice	Pres-Pwr	March Resources	20 and 2	1981
Rev. 1 Reviewed by FRC Approved by				
Approved by Man W Vice	Pres-Pwr	Resources	June.	2, 1981
Rev. Reviewed by FRG				19
Approved by Vice	Pres-Pwr	Resources		19

3.0 Scope:

3.1 Purpose

This procedure provides a method for estimating thyroid and whole body doses for the area surrounding the plant out to ten miles. This procedure is intended for use by the Emergency Coordinator upon indications that a loss of coolant accident has occurred.

3.2 Discussion

3.2.1 The Emergency Coordinator shall estimate thyroid and whole body doses for off-site locations from one mile to ten miles if a loss of coolant accident has occurred.

3.3 Authority

This procedure implements the St. Lucie Plant Radiological Emergency Plan.

4.0 Precautions

None.

5.0 Responsibilities:

The Emergency Coordinator or his designee has the responsibility for making whole body dose rate estimates and calculating integrated whole body doses.

The Chemistry Department representative shall make refined estimates upon arriving at the scene at the direction of the Emergency Coordinator.

6.0 References:

- 6.1 St. Lucie Plant Radiological Emergency Plan
- 6.2 St. Lucie Plant Radiation Protection Manual.

7.0 Records and Notifications:

Detailed records of meteorological conditions used to estimate doses, and the estimated thyroid and whole body doses shall be kept on the attached worksheets. As deemed appropriate by the Emergency Coordinator, the off-site authorities shall be notified of:

- Meteorological conditions (wind speed, wind direction, stability and precipitation).
- 2) Projected thyroid and whole body doses at 1, 2, 5, and 10 miles, and
- Whether default values of actual measurements were used for dose estimates.

8.0 Instructions:

- 8.1 The Emergency Coordinator shall, upon the initiation of an event which is determined to be a loss of coolant accident, project the thyroid and whole body doses in areas surrounding the plant as follows:
 - 8.1.1 The dose rate as measured by the post-LOCA radiation monitor at the shield building wall shall be determined and recorded on App. A . The monitor reading shall be used to further categorize the event in accordance with the following:

Post-LOCA Monitor Reading	Category of LOCA
< 30 mr/hr	Case A
30-100 mr/hr	Case B
100 mr/hr - 1R/hr	Case C
> 1 R/hr	Case D

This category shall also be included on App. A . If it has been determined that a Case A LOCA has occurred, doses will be low and no further estimates are required.

8.1.2 Determine stability class from the temperature difference between the 200 foot temperature sensor reading and the 32.0 for temperature sensor reading from the meteorological tower (i.e. 200 foot reading minus 32.8 foot reading). This value should be an average for at least a fifteen minute period. Record this value on Appendix A.

8.1.3 Determine the stability class from the following Table based on the temperature difference measurement:

ΔT(from 8.1.2)	Stability Class				
< - 1.6	A and B				
-1.5 to - 0.5	C and D				
> - 0.4	E and F				

Record the stability class on Appendix A.

NOTE: If temperature difference measurements are not available, then a default value for stability class is required. During daytime assume C and D stability and during night-time assume E and F stability. Record the stability class on App. A and note that it represents a default value.

R1

R1

8.0 Instructions: (cont'd)

8.1 (cont'd)

- 8.1.4 Determine the average wind speed over at least the most recent fifteen minute period using the output of the wind speed sensor at the lower level on the meteorological tower. Record on App. A.
- NOTE: If the lower level sensor value is not available, use the upper level sensor value and note on App. A. If neither sensor output is available, assume a default value of 5 mph during daytime and 5 mph during nighttime. Record this value and note on Appendix A that it represents a default value.
- 8.1.5 Determine the average wind direction value over at least the most recent fifteen minute period from the wind direction sensor at the lower level of the meteorological tower. Record this value on App. A. If the lower level value is not available, use the upper level sensor value and note this on App. A. If neither sensor output is available, record "ALL" for wind direction on App. A.
- 8.1.6 Select the thyroid and whole body time-dose-distance tables corresponding to the meteorological conditions recorded on App. A from Tables D-1 through D-18. Record the dose factor on App. A based on the following:

LOCA Type	Dose Factor
Case B	1
Case C	6
Case D	60

Case B dose estimates are read directly from Tables D-1 through D-18. For Cases C and D the table values are multiplied by the dose factor.

8.1.7 Position the overlay, Figure D-1, with the "PLANT" reference point over the center of the reactor containment building on a map of the site area. Position the centerline of the overlay plume in the direction towards which the wind is blowing.

R1

- 8.0 Instructions: (cont'd)
 - 8.1 (cont'd)
 - 8.1.8 For each time period of interest (e.g. 7 hours 4 hours, etc.), record the dose information (Table values multiplied by the dose factor) on App. B. In order to account for wind shifts, the values should be recorded in the sectors adjacent to the indicated downwind sector. If "ALL" has been recorded for wind direction on App. A then it shall be assumed that the estimated doses could occur in any sector.
 - NOTE: To conservatively accommodate any plume stabilization effects that could conceivably result from a postulated potential stabilizing effect of the Indian River, those residents in sectors K, L, or M (between SW and WNW of the facility), should be considered to reside at 1.0 miles. Thus, Tables D-1 to D-18 should be used at a 1.0 mile distance for these environs.
 - 8.1.9 If there is a shift in wind direction of more than 45°, the procedure shall be repeated.

APPENDIX A

DOSE CALCULATION WORKSHEET

TIME	
INITIAL POST-LOCA MONITOR READING:	
CATEGORY OF LOCA: CASE	
TEMPERATURE DIFFERENCE:	oF
STABILITY CLASS:	
WIND SPEED:	mph
WIND DIRECTION:	
DOSE FACTOR:	

APPENDIX B

CUMULATIVE DOSE DISTRIBUTION

TIME RELEASE ASSUM	ED TO OCC	CUR:		
TIME THROUGH WHICH	THIS DIS	TRIBUTION A	PPLIES:	
Met Station Wind Direction (As Read)		wind ection	Thyroid Dose Downwind Distance (miles) 1 2 5 10	Whole Body Dose Downwind Distance (miles) 1 2 5 10
180	N	ALPHA		
202.5	NNE	BRAVO		
225	NE	CHARLIE		
247.5	ENE	DELTA		
270	E	ЕСНО		
292.5	ESE	FOXTROT		
315	SE	GOLF		
337.5	SSE	HOTEL		
000	S	JULIET		
022.5	SSW	KILO		
045	SW	LIMA		
067.5	WSW	MIKE		
090	W	NOVEMBER		
112.5	WNW	PAPA		
135	NW	QUEBEL		
157.5	NNW	ROMEO		

PASQUILL STARILITY CLASSES A & B

HINDSPEEDES MPH

DISTANCE (PILES)

		INE			1.99	2.00	3.00.	4.00	5.00	6.00	7,00	8.00	9.00	10.00
		2	5 HI	14	2.41-02	0.	υ,	0.	٥.	0.	0.	0.	0.	0.
	See - 10		D.WI	н	SE-02	0	0.	0.	0.	0.	0,	0.	6.	0.
		4	S HI	N	5.76-02	4.3E-03	0.	0.	0.	0.	0.	0.	4.	0.
1	HR		o HI	N	8.0E-02	7.2E-03	0.	0.	o.	0.	9.	0.	0.	0.
1	HR		HI.	H	1.0E-01	1,41-02	6.0E-03	2.5E-03	0.	0.	0.	0.	0.	0.
2	нк		n HI	н	1.25-01	1.76-02	9.76-03	5.98-03	2.88-03	0.	0.	٥.	0,	0.
5	нв	3	н1	н .	1,31-01	1.98-02	1,16-02	7.9E-03	5.7F-03	3.1E-03	1.56-03	0.	٥.	٥.
	HP	-	HI.	n	1.5E-01	2.2E+02	1,35-05	9.1E-03	6,7E-03	5.1E-03	3.5E-01	1.81-03	0.	0.
6	HR		HI	N	2.4E-01	3.58-02	2.1E-02	1.58-02	1,28-02	9.5E-03	7.06-03	6.51-03	5.66-03	4.8E-03
7	HR		, HÎ	н	10-30.5	3.86-02	2.35-02	1.7E-02	1.36-02	1.1E-02	6.9E-03	7.58-03	6.4E-03	5.5E-03
10	HR		. HI	N	3,26-01-	4.8E-02	2,9E-02	2.28-02	1.75-02	1.4E-02	1.26-05	1.06-02	8.98-03	7.06-03
15	HR		HI.	N	3,5E-01	5.28-02	3.2E-05	2.4E-02	1.96-02	1.66-02	1.36-02	1.26-02	1.01-02	9.0E-03
14	tift		HI	N	3.8E-01	5.58-02	3.4E-02	2. AE-02	2.0E-02	1.78-02	1.4E-02	1.26-02	1.18-02	9.86-03
10	HR		нТ	N	4.0E-01	5.9E-02	1.7E-02	2.7E-02	5.5E-05	1.86-02	1.5E-02	1.36-02	1.28-02	1.06-02
50	HR	(HI	14	8.5E-01	6.6E-02	4.1E-02	3.1E-02	2.56-02	2.0E-02	1.78-02	1.51-02	1,36-02	1.2E-02
24	HR		HI	N !	5.0E-01	7.3E-02	4.6E-02	3.4E-02	2.78-02	5.5E-05	1.98-02	1.78-02	1.56-02	1.36-05

. THYRDID DOSE (REH)

ST. LICIE EMERGENCY PLAN

DYERLAND DISPERSION

PASQUILL STABILITY CLASSES A & 8

MINDSPEEDES HPH

DISTANCE (HILES)

TABLE D-3

THYROTO COSE (REH)

ST. LUCIE ENERGENCY PLAN

OVERLAND DISPERSION

1	(many)						MINDSPEED=8 HPH	MINDSPEED=8 HPH					
							DISTANCE	(HILES)					
	1	3 1 1 1		00	2.00	3.00	4.00	5.00	6.00	7.00	00.00	00.6	10.00
		No.	r I r	1.7E-03		ó	.0	. 0			.0	. 0	
		30	N.	\$. 2E - 0.3	2,56-03	.0		.0	.0	. 0			
-		6.5	HIN	3.0E-02	3.06-03	2.0E-03	.0	.0	.0	.0	.0	.0	.0
	H		NIN	1,56-02	5,26-03	2.98-03	1.08-03	.0	.0	. 0	.0		. 0
1	==	1	30 HIN	4,15-02	6.25-03	3.05-03	2,8E-03	2,2E-03	1.8E-03	1.36-03	.0	.0	. 0
	2 HR		NIH O	9.75-02	7.18:03	4.4E-03	3,36-03	2.6E-03	2.15-03	1.06-03	1.56-03	1,36-03	. 0
	Z HR		NIH DE	5,36-02	8.0E-03	5,06-03	3.78-03	3.0E-03	2.46-03	2.16-03	1.66-03	1,58-03	3. 16-0
	3 HR		N1H 0	6.0E-02	9.08-03	5.68-03	4.28-03	3,36-03	8.76.03	2,36-03	2.06-03	1.05-03	1 . 56 - 0
	4 II 8		нін б	9.25-02	1.96-02	B. 6E-03	6.56-03	5,26-03	4.35-03	3.66-03	3,26-03	2.06-03	2.5E-0
	7 HH		NIHO	1.0E-01	1.5E-02	9.58-03	7,18-03	5,76-03	4.76-03	4.06-03	3.58-03	3.16-03	2.85-0
-	10 HR		NIH	1.26-01	1.85-02	1,16-02	8.68-03	6.96-03	5.86-03	4.96-03	4.35-03	1.96-03	1.56-0
-	12 118		NIH	1.3E-91	3.0E-02	1.25-02	9.36-03	7,58-03	6.25-03	5.4E-03	4.7E-03	4.25-03	3.86-0
-	14 118		NIH O	1.46-01	2.16-02	1.36-02	1.05-02	8.06-03	\$.7E-03	\$.7E-03	5.06-03	4.SE-03	8.0E-0
-	16 HR	1	N1H 0	1.58-01	2.2E-02	1.46-02	1.16-02	8,5E-03	7.16-03	\$.1E-03	5.36-03	4.76-03	4.36-0
~	20 IIB	- 1	HIH 0	1.76-01	\$.5E-02	1.66-02 1.26-02	1.26-02	9.45-03	\$. 0 E - 0 \$	\$. BE = 03	6.06-03	5,36-03	4.86-0
~	24 HR		N1H 0	1.96-01	2.86-02	1.76-02	1.36-02	1,16-02	8.05-03	7.56-03	6.68-03	\$.98-03	5.36.0
				Street or other Persons	AND DESCRIPTION OF REAL PROPERTY.	Married Street, or other Persons and	N. S.	大田田 二四 七年五一日					

THYFOLD DOSE (REH)

ST, LUCIE ENERGENCY PLAN

OVERTAND DISPERSION

· PASOUILL STAHILITY CLASSES C . D

WINDSPEED # 1 PH

	3171		1.00	2.00	3,00	00.00	5.00	00:	7.00	00.0	00.6	10.00
		HIH 22	2.7E-01	* 0	.0	.0	. 0	.0	.0	•	.0	.0
1		NIH OF	3.9E-01	.0	*0	.0	.0	.0	.0	.0	.0	.0
1		45 MIN	6.3E-01	1.16-01	.0	.0	.0		.0	.0	.0	.0
	H H8	NIH 0	8.8E-01	1.05-01	.0	.0	.0	.0	.0	•		.0
-	1 118 3	NIH OF	1,15,00	3.5E-01	1.2E-01	4.36-02	*0	.0	.0	.0	.0	.0
~	HR	NJH 0	1.36+00	4.16-01	2.0E-01	10-30-1	4.2E-02	.0	•	•	,0	.0
~	HB 3	30 HIN	1.58+00	4.7E-01	2.36-01	10-30"1	8.98-02	4.36-02	1.98-02	.0	.0	.0
ada.	3 H.R	NIH O	1.6E+00	5.1E-01	2.75-01	1.66-01	10-30-1	T. 2E=02	4.56-02	2.1E-02	0.	.0
9	нВ	NIN O	2,1E+00	6.8E-01	3, 6E-01	2,26-01	1,66-01	1.2E-01	8.9E-02	7.1E-02	5.85-02	4.98-02
-	ня	2 2	2,25+00	7.36-01	3.86-01	2.0E-01	1.76-01	1.28-01	9.6E-02	7.7E-02 .		5.36-02
10 /HR	8	N. H. O	2, 6E+00	8.4E=01	4.45-01	10-38-2	Z.0E-01	10-35.1	10-31.1	9.26-02	7.6E-02	6.4E-02
12 HR	R	NIH 0	2.7E+00	8.98-01	4.7E-01	3.0E-01	2.1E-01	1.66-01	1.28-01	10-30.1	6,35-02	7,06-02
14 HR		NIH O	3.98+00	9.48-91	4.98-01	3.16-01	2.2E-01	1.66-01	1.35-01	1.18-01	8.7E-02	7.46-02
16-118		NIH D	1.0E +00	9.8E=01	5.1E-01	3.28-01	2,36-01	10-31.1	10-36-1	10-31.1	9.2E-02	7.86-02
20 H	HR	NIH 0	3,3£+00	1.1E+00	5.6E-01	1,56-01	2.58-01	10-36-1	1.56-01	1.26-01	1.05-01	8.5E-02
24 H	HR	HIH 0	3.6E+00	1.2E+00	6.1E-01	3.85-01	2.7E-01	3 OF -01	1 45-01	1 15=01	4 4 5 - 0 1	0 25-03

TAISLE D-5 IHYROID DOSF (REH)

ST. LUCIE EMERGENCY PLAN

OVERLAND DISPERSION

PASQUILL STABILITY CLASSES C & D

HINDSPEEDS HPH

2 2 2 3 3 4 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5		3,00 A.00 4,1E-02 0. 4,1E-02 0. 1,2E-01 6,7E-02 1,4E-01 6,7E-02 1,6E-01 9,7E-02 1,6E-01 1,9E-01 2,2E-01 1,9E-01 2,8E-01 1,9E-01 3,0E-01 1,9E-01 3,1E-01 2,0E-01	0.	3,00 A.00 4,1E-02 0. 4,1E-02 0. 1,2E-01 6,7E-02 1,4E-01 8,4E-02 1,5E-01 1,1E-01 2,2E-01 1,8E-01 2,8E-01 1,9E-01 3,0E-01 1,9E-01 3,4E-01 2,6E-01	(*11.E5)	5.00 6.00 7.00 6.00 9.00 10.00	0. 0. 0. 0.	0. 0. 0. 0.	0, 0, 0, 0, 0,	.0 .0 .0 .0	3,5E-02 1.9E-02 1.0E-02 0. 0.	5.76-02 a.16-02 2.56-02 1,56-02 6,56-03 0,	6.6E-02 4.8E-02 3.6E-02 2.8E-02 2.0E-02 1.3E-02	7.56-02 5.56-02 4.26-02 3.36-02 2.6E-02 2.1E-02	9.9E-02 7.5E-02 5.9E-02 4.7E-02 3.9E-02 3.3E-02	1.16-01 7.96-02 6,36-02 5.16-02 4,26-02 3.06-02	1, 2E-01 4, 1E-02 7, 2E-02 5, 9E-02 4, 9E-02 4, 2E-02	1,3E-01 9,7E-02 7,7E-02 6,2E-02 5,2E-02 4,4E-02	1.36-01 1.0E-01 6.0E-02 6.6E-02 5.5E-02 4.7E-02	1.4E-01 1.1E-01 8.4E-02 6.8E-02 5.7E-02 4.9E-02	1.9E-01 1.2E-01 9.1E-02 7.9E-02 6,2E-02 5,3E-02
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THYAU	n	DUSE	(REM)
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BT. LUCIE EHERGENCY PLAN

DYERLAND DISPERSION

PARQUILL STARILITY CLASSES C & D

HIHOSPEEDEA HPH

DISTANCE (HILE	5)
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-	-	one excession			Carlo Sales State Company State Company				-	-			
	11			1.00	2.00	3,00	4.00	5,00	6.00	7.00	8.00	9.00	10.00
		22	. HIH	1,98-01	4.6E-02	0.	0.	0.	0.	7- ,	0.	0,	0.
		.30	-HIH-	1,45-91	6.3E-02	2.5E-02	3.5E-04	0.	٥,	0,	٥.	٥,	٥.
		45	HIH	3,38-01	9.56-02	4.1E-02	2.1E-02	1.1E-02	2.9E-04	0,	0,	0,	0,
1	HR	0	HIH (3.9E-01	1.38-01	5.9E-02	3.26-02	1.9E-02	1.1E-02	6.88-03	2.68-04	0.	0.
+ 1	HR	30	HIN	4,6E-01	1.5E-01	7.98-02	4.9E-02	3,4E-02	\$.5E-02	1.76-02	1.1E-02	7.76-03	5.2E-01
5	HR	0	HIN	5,2E-01	1.7E-01	9.2E-02	5.8E-02	4.0E-02	3.0E-02	2,3E-02	1.8E-02	1,58-02	1.28-02
5	HR	30	HIN	5.7E-01	1,98-01	1.0E-01	6.58-02	4.6E-02	3.4E-02	5.6E-05	2.18-02	1.78-02	1.4E-02
- 3	HR	- 0	-HIH-	0.1E-01	2.1E-01	1.18-01	7.0E-02	4,98-02	1.7E-02	2.9E-02	2.46-05	2,08-02	1.66-02
6	HR	0	HIN	8.02-01	2.7E-01	1,4E-01	9.0E-02	6.4E-02	4.8E-02	3.8E-02	3.16-02	2,6E-02	2.2E-03
7	HR	0	нін	8.5E-01	2.8E-01	1,56-01	9.46-02	6.8E-02	5.16-02	4.1E-02	3,36-02	2.86-02	2.4E-02
10	HR-	0	HIN	9.75-01	1.5E=01	1.7E-01	1.18-01	7.7E-02	5.8E-02	4.66-02	3.86-02	3.28-05	2.7E-02
15	HR	0	нін	1.02+00	3.48-01	1.8E-01.	1.1E-01	8,18-02	6.1E-02	4.98-02	4,0E-02	3,36-02	2.9E-02
14	HR	٥	нін	1,1E+00	3,68-01	1,9E-01	1.25-01	8.5E-02	4.4E-02	5.18-02	4.2E-02	3,58-02	3.06-02
16	HR-	- 0	HIN	1.12+00	3.7E-01	10-30.S	1.5E-01	B.9E-02	6.7E-02	5.3E-02	4.3E-02	3,66-02	3.1E-02
20	HR	0	HIH	1.22+00	4.1E-01	2.16-01	1.08-01	9.78-02	7.3E-02	5.88-02	4.76-02	4.08-02	3.4E-02
24	HR	. 0	нін	1.3E+00	4.45-01	2.3E-01	1.58-01	1.02-01	7.98-02	6.2E-04	5,18-02	4.36-02	3.66-02

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ST, LUCIE ENERGENCY PLAN

OVERTAND DISPERSION

PASQUILL STABILITY CLASSES E . F

HINDSPEEDS HPH

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1	-	HIH OF	1,5€+00	.0	*0	.0	.0	.0	.0	.0	.0	.0
	4	NIH SE	2,4E+00	5.16-01	.0	.0	. 0	• 0	.0	•	.0	.0
HH		NIH 0	3.46+00	10-30.0	.0	. 0		0.0		•		
E	-	NIH OF	9.4E+00	3.75+06	6. bE-01	2.56-01	.0	.0	.0	.0	.0	
S HR		O HIN	5.0E+00	2.0E+00	1.1E+00	8.96-01	2.56-01	.0	• 0	. 0	.0	.0
* HB		30 HIM	5.5E+00	2,3£100	1,26100	7.86-01	5.26-01	2.6E-01	1.25-01	.0	.0	.0
3 HR		NIH D	5.8E+00	2.45+00	1 . 4E+00	9.0E-01	6.18-01	10-35.0	2.8E-01	1,46-01	.0	.0
P HIS		NIH	7,25,00	2.9E+00	1.75+00	1 . 1E + 00	6,36-01	6.58-01	5.28-01	4.36-01	3.76-01	3.26-01
Œ		NIH O	7.5E+00	3.08+00	1.76+00	1.28400	8.66-01	6.75-01	5.46-01	4	3,96-01	3.38-0
10 HR	-	HIH 0	00+36+00	3,36+00	1.96+06	1.36+00	10-35.6	10-31-4	10-30.9	5.0E-01	4,36-01	3.76-0
12 HB		N1H 0	0.7E+00	3.45+00	2.0E+00	1.3E+00	9.86-01	7.7E-01	6.2E-01	5.28-01	4.56-01	3.96-0
14 HR		N1H 0	9.0E+00	3.66+00	2.06+00	1.46+04	1.01+00	10-30.4	10-35.9	5.46-01	4.68-01	4.0E-0
16 HR	1	NIH 0	00+36+60	3.76+00	2,16+00	1.45+00	\$. 0E+00	A.2E.01	10-19.9	10-39.8	4.86-01	4.16-0
20 HR		NIH O	1,04 30,1	3.98+00	2.2E+00	1.56.00	1,16,00	8.76-01	7,06-01	10-36.5	5,06-01	9,46=0
24 HR		NIH O	1.15+01	4.16.40	2.4E+00	1.68+00	1.28+00	15-01	7 05-01	A 35 = 0.1	F 15-01	45.00

TABLE 0-8

THYAGID DUSE (REM)

ST, LUCIF EMERGENCY PLAN

EYERLAND DISPERSION

PASSULL STARLITY CLASSES E & F

0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 0	2.00 3.00 0.00 5.00
0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 0, 2, 16-01 1,2E-01 1,5E-01 1,6E-01 1,5E-02 5,6E-02 3,9E-01 2,9E-01 1,9E-01 1,9E	3.36-01 0. 0.
3.4E-01 1.2E-01 1.6E-01 9.5E-02 5.6E-02 3.9E-01 2.5E-01 1.6E-01 9.5E-02 5.6E-02 3.9E-01 3.9E-01 2.5E-01 1.8E-01 1.3E-01 4.4E-01 3.9E-01 2.6E-01 2.1E-01 1.7E-01 5.2E-01 4.7E-01 3.3E-01 2.9E-01 2.5E-01 6.0E-01 4.9E-01 3.9E-01 3.2E-01 2.9E-01 6.2E-01 4.9E-01 3.9E-01 3.3E-01 2.9E-01 7.1E-01 5.0E-01 4.3E-01 3.0E-01 3.1E-01 7.1E-01 5.0E-01 4.5E-01 3.0E-01 3.1E-01	5.6E-01 2.2E-01 0.
3.4E-01 1.2E-01 1.6E-01 9.5E-02 5.6E-02 3.9E-01 2.9E-01 1.6E-01 1.8E-01 1.3E-01 4.4E-01 3.9E-01 2.5E-01 1.8E-01 1.3E-01 5.2E-01 4.1E-01 3.4E-01 2.9E-01 2.9E-01 5.4E-01 4.9E-01 3.7E-01 3.3E-01 2.9E-01 6.2E-01 4.9E-01 4.0E-01 3.3E-01 2.9E-01 6.4E-01 5.0E-01 4.3E-01 3.0E-01 3.6E-01 7.1E-01 5.0E-01 4.5E-01 3.0E-01 3.1E-01	0.2F-01 3.5E-01 1.6E-01
3, 9E-01 2.5E-01 1.6E-01 9.5E-02 5.6E-02 4, 4E-01 3.9E-01 2.6E-01 2.1E-01 1.3E-01 5, 2E-01 4.1E-01 3.3E-01 2.9E-01 2.9E-01 5, 8E-01 4.6E-01 3.7E-01 2.9E-01 2.5E-01 6, 6E-01 4.9E-01 3.9E-01 3.3E-01 2.9E-01 6, 4E-01 5.0E-01 4.3E-01 3.9E-01 3.6E-01 7.1E-01 5.6E-01 4.5E-01 3.6E-01 3.1E-01	1.1E+00 6.3E-01 3.8E-01
3,9E-01 2,9E-01 2,3E-01 1,0E-01 1,3E-01 4,4E-01 3,4E-01 2,6E-01 2,1E-01 1,7E-01 5,2E-01 4,1E-01 3,7E-01 2,9E-01 2,5E-01 6,0E-01 4,9E-01 3,9E-01 3,2E-01 2,0E-01 6,4E-01 5,0E-01 4,0E-01 3,3E-01 2,9E-01 7,1E-01 5,6E-01 4,5E-01 3,6E-01 3,1E-01 7,1E-01 5,6E-01 4,5E-01 3,6E-01 3,1E-01	1.3E+00 7.3E-01 4.8E-01
5.2E-01 3.4E-01 2.6E-01 2.1E-01 1.7E-01 5.4E-01 4.2E-01 3.4E-01 2.9E-01 2.9E-01 5.6E-01 4.6E-01 3.7E-01 3.2E-01 2.8E-01 6.2E-01 4.9E-01 4.0E-01 3.2E-01 2.9E-01 6.4E-01 5.0E-01 4.5E-01 3.6E-01 3.1E-01 7.1E-01 5.6E-01 4.5E-01 3.6E-01 3.1E-01	1.4E+00 8.3E-01 5.5E-01
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5.4E-01 4.5E-01 3.4E-01 2.9E-01 2.5E-01 6.0E-01 4.7E-01 3.9E-01 3.2E-01 2.6E-01 6.2E-01 4.9E-01 4.0E-01 3.3E-01 2.9E-01 6.4E-01 5.0E-01 4.3E-01 3.0E-01 3.1E-01 7.1E-01 5.0E-01 4.5E-01 3.0E-01 3.1E-01	1.8E+00 1.0E+00 7.0E-01
5.8E-01 4.6E-01 3.7E-01 3.1E-01 2.7E-01 6.2E-01 4.9E-01 4.0E-01 3.3E-01 2.9E-01 6.4E-01 5.0E-01 4.3E-01 3.0E-01 3.6E-01 7.1E-01 5.0E-01 4.5E-01 3.0E-01 3.1E-01	1.8E+00 1.1E+00 7.3E-01
6.2E-01 9.9E-01 9.0E-01 3.2E-01 2.8E-01 6.4E-01 8.0E-01 4.0E-01 3.3E-01 2.9E-01 6.4E-01 8.3E-01 4.3E-01 3.6E-01 3.1E-01 7.1E-01 8.6E-01 4.5E-01 3.8E-01 3.3E-01	2.0E+00 1.2E+00 7.RE-01
6.2E-01 9.9E-01 4.0E-01 3.3E-01 2.9E-01. 6.4E-01 5.0E-01 4.3E-01 3.0E-01 3.0E-01 7.1E-01 5.6E-01 4.5E-01 3.0E-01 3.3E-01	2.1E+00 1.2E+00 8.1E-01
6.4E-61 5.0E-01 4.1E-01 3.4E-01 3.0E-01 7.0E-01 7.1E-01 7.1E-01 5.6E-01 4.5E-01 3.8E-01 3.3E-01	2.2E+00 1.2E+00 8.4E-01
7.1E-01 5.6E-01 4.5E-01 3.0E-01 3.1E-01	2.2E+00 1.1E+00 8.6E-01
7.1E-01 5.6E-01 4.5E-01 3.8E-01 3.1E-01	2.4E+00 1.4E+00 9.2E-01
	2.5E+00 1.4E+00 9,7E-01

TABLE D-9
THYROTO BOSE (REH)

ST. LUCIF EMERGENCY PLAN	
I. LUCIF EMERGENCY PL	
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HINDSPEEDER HPH

30 HIN 7,3E-01 2.2E-01 30 HIN 1,5E+00 4,6E-01 30 HIN 1,5E+00 6,1E-01 30 HIN 2,0E+00 7,6E-01 30 HIN 2,0E+00 7,6E-01 0 HIN 2,1E+00 8,4E-01 0 HIN 2,1E+00 8,1E-01 0 HIN 3,1E+00 1,2E+00 0 HIN 3,1E+00 1,2E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,6E+00 1,3E+00		DISTANCE (HILES)	(H1LEB)					
22 HIN 7, 3E-01 2, 2E-01 30 HIN 1, 3E+00 4, 6E-01 00 HIN 1, 8E+00 7, 6E-01 00 HIN 2, 0E+00 8, 4E-01 00 HIN 2, 2E+00 8, 4E-01 00 HIN 2, 2E+00 9, 1E-01 00 HIN 2, 8E+00 1, 2E+00 00 HIN 3, 3E+00 1, 3E+00	3.00	00:	5.00	01	7.00	001	00:	10.00
30 HIN 1.3E+00 4.6E-01 0 HIN 1.5E+00 6.1E-01 0 HIN 2.0E+00 8.4E-01 0 HIN 2.2E+00 8.4E-01 0 HIN 2.2E+00 9.1E-01 0 HIN 2.8E+00 1.2E+00 0 HIN 3.8E+00 1.3E+00 0 HIN 3.8E+00 1.3E+00 0 HIN 3.8E+00 1.3E+00 0 HIN 3.6E+00 1.3E+00					.0	• 0	•	
95 HIN 1.3E+00 9.6E-01 9 HIN 1.5E+00 6.1E-01 9 HIN 2.0E+00 8.4E-01 9 HIN 2.7E+00 9.1E-01 9 HIN 2.7E+00 1.2E+00 9 HIN 3.8E+00 1.3E+00 0 HIN 3.8E+00 1.3E+00 0 HIN 3.8E+00 1.3E+00	1,36-01	\$0-30.5	.0	.0	.0	.0	.0	.0
0 HIN 1,5E+00 6,1E-01 30 HIN 2,0E+00 7,6E-01 30 HIN 2,1E+00 8,4E-01 0 HIN 2,2E+00 9,1E-04 0 HIN 2,8E+00 1,2E+00 0 HIN 3,1E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00	2.25-01	1.26-01	6,66-02	1.86-03	.0	.0	.0	.0
30 HIN 2,0E+00 7,0E-01 30 HIN 2,0E+00 8,4E-01 9 HIN 2,2E+00 9,1E-04 0 HIN 2,2E+00 9,5E-01 0 HIN 2,8E+00 1,2E+00 0 HIN 3,1E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,6E+00 1,3E+00	10-22.5	10-38-1	1.16-01	6.85-02	4.36-02	1,76-03	.0	
30 HIN 2,0E+00 8,4E-01 30 HIN 2,1E+00 9,1E-04 0 HIN 2,2E+00 9,5E-01 0 HIN 2,8E+00 1,2E+00 0 HIN 3,1E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00	10-32.E	Z.8E-01	2.0E-01	1.5E-01	10-30.1	1.36-02	5.1E-02	3.56-02
30 HIN 2,2E+00 9,1E-01 0 HIN 2,2E+00 9,5E-01 0 HIN 2,8E+00 1,2E+00 0 HIN 3,1E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,4E+00 1,3E+00 0 HIN 3,6E+00 1,5E+00	4.96-01	3.76-01	2,46-01	1.85-01	10-37-1	1,25-01	9.72-02	50-30 g
0 HIN 2,2E+00 9,5E-01 0 HIN 2,8E+00 1,2E+00 0 HIN 3,1E+00 1,3E+00 0 HIN 3,9E+00 1,3E+00 0 HIN 3,9E+00 1,3E+00 0 HIN 3,9E+00 1,5E+00	10-35.5	3.76-01	2.7E-01	7.16-01	10-32.1	10- 46-01	1.16-01	20-39.6
0 HIN 2.8E+00 1.1E+00 0 HIN 3.8E+00 1.3E+00 0 HIN 3.4E+00 1.3E+00 0 HIN 3.4E+00 1.3E+00 0 HIN 3.8E+00 1.5E+00	10+39.8	3.8E-01	2,9E-01	\$.25-01	1,86-01	1.56-01	1,38-01	1.16-0
0 HIN 3,8E+00 1,2E+00 0 HIN 3,9E+00 1,3E+00 0 HIN 3,9E+00 1,3E+00 0 HIN 3,9E+00 1,5E+00	10-35.9	4.01-01	3,36-01	7.6E-01	2.16-01	1.85-01	1,58-01	1.36-01
0 HIN 3, 15,00 1,35,00 0 HIN 3,45,00 1,35,00 0 HIN 3,45,00 1,45,00 0 HIN 3,55,00 1,55,00	6.7E-01	4.65-01	3.4E-01	2.7E-01	2,2E-01	1.05-01	1.68-01	1.48-0
НВ 0 НІИ 3,4E+00 1.3E+00 НВ 0 НІИ 3,4E+00 1.3E+00 НВ 0 НІИ 3.6E+00 1.5E+00	7.2E-01	10-36.0	3.7E-01	10-36.2	10-36-2	2.08-01	10.31.1	1.56-0
HR 0 HIN 3,9E+00 1.3E+00 HR 0 HIN 3,8E+00 1.5E+00	1,56-01	5.16-01	3. BE-01	1.0E-01	3.4E-01	2.18-01	1,81-01	1.58-0
HR 0 HIN 3.0E+00 1.5E+00	7.8E-01	5.36-01	10-36.8	1,16-01	\$. 5E-01	10-31.5	10-30.1	1,61-0
0 HIN 3.8E+00 1.5E+00	8.0E.01	8.0E-01	4.0F-01	1.25-01	2.68-01	3,25-01	1.9E-01	1.65-0
	8.5E-01	5,86-01	4.38-01	1.36-01	2.7E-01	2.38-01	2.0E-01	1.76-0
24 HR O MIN 4.1E+00 1.6E+00 9.	9.0E-01	6.15-01	4.56-01	3.56-01	2.96-01	2.48-01	2.16-01	1.86-01

HHOLE BODY DOSE CON

ST, LUCIF EMENGENCY PLAN

OVERLAND DISPERSION

PASMITTLE STANILITY CLASSES A & B

HINDSPEED#3 HPH

DISTANCE (HILES)

		IHE			1,00	2.00	3.00	4.00	5.00	6.00	7.00	8.00	9.00	10.00
		35	Н	IN	9.8E-00	0.	٥.	0.	0.	0.	٥.	0,	٥.	0.
		- 30	7	IH	1.75-05	Q.	0.	0.	0,	0.	0.	0.	0.	0.
		45	5 H	IH	3.6E-05	2.6E-06	0.	0.	0.	0.	٥.	٥.	0,	0.
1	I HR		0 1	114	6.0E-05	5.68-06	0,	0.	0.	ρ.	٥,	0.	0.	٥.
	1 1118		0 1	IIN -	9,56-05	1,58-05-	5. CE-06	1.98-06	0.	0.	0.	C.	0.	0.
	2 HR		0 +	IIN	1.26-04	2.15-05	1.3E-05	h. 8E-06	2,5F=06	0.	0.	0.	0.	0.
	2 HR	3	0 1	IN	1.5E-04	2.6E-05	1.6E-05	1.18-05	7,38-06	3.2E-06	1.18-00	0,	0.	0,
	3 HR		0 1	IN	1.7E-04	3.16-05	2.0E+05	1.4E-05	9.88-06	6.9E-06	3.9E-00	1,51-06	0.	0.
	6 HR		0 1	HIN	2.8E-04	5.2F-05	3.5E-05	2.66-05	2,06-05	1.6E-05	1.38-05	1.06-05	8.4E-06	7.08-06
	7 116	1	0 1	нін	3,26-04	5.8E-05	3.9E-05	3.0E-05	2.36-05	1.9E-05	1.56-05	1.28-05	9.98-06	8.25-06
	0 HF			HIN	4.1E-04	7.5E=05	5.1E-05	3.9E-05	3,16-05	2.58-05	2,16-05	1,78-05	1.48-05	1.2E-05
	2 118			нін	4.6E-04				3.5E-05	2.98-05	2.4E-05	2.0E-05	1.7E-05	1.48-05
-	4 116		-	HIN	5.16-04		6,3E-05	100					1.48-05	1.68-05
	6 H			nIN	5.50-04					3.58-05	2.9E-05	2.4E-05	2.1E-05	1.8E-05
	0 11			HIN	6.3E-04				4.9E-05		3.38-05	2.8E-05	2.41-05	2.18-05
*****	4 11			HIN	7.18-04		AND THE RESIDENCE OF THE PARTY	6.7E-05	5.48-05	4.5E-05	3.7E-05	3.2E-05	2.86-05	2.4E-05

. . .

HIH

THELE D-11 HHOLE BODY DOSE (HEM)

ST. LUCIE EMERGENCY PLAN

OVERLAND DISPERSION

PASQUILL STABILITY CLASSES A & 8

HINDSPEED . S HPH

DISTANCE (HILES)

	11			1.00	2.00	3,00	4.00	5.00	6.00	7.00	8.00	9.00	10.00
			нін	1.1E-05	٥.	9.	٥.	0.	9,	ρ,	0.	٥.	٥.
		34	HIN	1.8E-05	1.8E-06	0.	0.	0.	٠,	٥.	0.	٥.	0.
		45	HIN	3.1E-05	3.9E-04.	1.58-06	.0.	0,	0.	0.	0.	٥.	0,
1	118	9	HIN	4.6E-05	6.88-06	3.16-06	1.4E-06	0.	0.	٥.	٥.	0.	0.
1	ня	30	нін	6,4E-05	1,2E-05	7.58-06	4.6E-04	2.6E+06	1.38-06	6.28-07	0.	0,	0,
2	HR.	0	MIN	8.08-05	1.5E-05	1.0E-05	7.4E-06	5,5E-06	4.0E-06	2.38-06	1.28-06	6.3E-07	0.
2	HR	30	нін	9.5E-05	1.8E-05	1.26-05	9.4E-06	7.1E-06	5,58-06	4.2E-06	3.2E-06	2.28-06	1.56-00
3	ня	0	нін	1.16-04	2.1E-05	1.4E-05	1.1E-05	8.8E-06	6,98-06	5.4E-06	4.28-06	3,36-06	2.7E-06
6	HR .	0	HIN	1.85-04.	3.4E-05	2.3E-05	1.88-05	1,56-05	1.28-05	9.98-06	40-32.4	7.16-06	6.1E-06
7	ня	0	нии	2.0E-04	3.86-05	2.6E-05	2.18-05	1,78-05	1.48-05	1.16-05	9.46-06	8.0E-06	7.05-06
10	HR	0	нін	2.58-04	4.7E-05	3.3E-05	2.6E-05	2.1E-05	1.08-05	1,56-05	1.36-05	1.18-05	4,56-06
12	ня	. 0	нін	2.8E-04	5.38-05	3.7E-05	2.98-05	2.4E-05	2.08-05	1.6E-05	1.46-05	1.26-05	1.16-05
14	нн		ніи	3.18-04	5.88-05	4,0E-05	3.28-05	2,6E-05	2.28-05	1.86-05	1.56-05	1.36-05	1.26-05
16	HB	0	H531	3.4E-04	6.3E-05	4.4E-05	3.48-05	2,81-05	2.36-05	2,0E-05	1.7E-05	1.56-05	1.38-05
20	.HB.	0	HIN	3.9E-09	7.2E-05	A.9E-05	1.9E-05	3.26-05	2.7E-05	2.2E-05	1.98-05	1.76-05	1.58-05
24	HR	0	HIN	4.38-04	7.9E-05	5.58-05	4.3E-05	3.5E=05	2.98-05	2.56-05	2.16-05	1.4E-05	1.7E-05

.. WHOLE BODY DOSE (REH)

ST. LUCTE EMERGENCY PLAN

DVERLAND DISPERSION

PASQUILL STABILITY CLASSES A & B

HINDSPEED O HPH

DISTANCE (HILES)

		11	18		1.00	2.00	3,00	4.00	5,00	6.00	7.00	8.00	9.00	10.00
			55	нін	1.0E-05	٥.	٥,	0.	٥,	0.	٥,	٥.	٥,	0.
10 m (m)	14.00		30	HIN	1,4E-05	2.0E-06	0,	0.	0.	0.	0.	0.	0.	0.
			. 45	HIH.	2.3E-05	3.08-04	2.0E=06	.0,	. 0	0,	0,	0	0.	0.
	, ,	HR	0	HIN	3.1E-05	5.82-06	3,58+06	2.28-06	0,	Q.	0,	0.	0.	0.
	1	HR	30	нін	4,26-05	8.2E-06	5.78-06	4.3E-06	3,3E-04	2.6E-06	1.7E-06	0.	0.	0,
	2 1	HR .	0	HIH.	5.2E-05	1.08-05	7.3E-06	5.7E-06.	4.58-06	3.6E-06	2.98-06	5.3E-06	1.98-06	0.
-	2 1	HR	30	HIN	6.1E-05	1.2E-05	8,7E-06	7.0E-06	5.6E-06	4.66-06	3,78-06	3.08-06	2.58-06	2.1E-06
	3 1	HR	0	HIN	7.0E-05	1.4E-05	9.8E-06	7.98-06	6,4E-06	5,3E-06	4.4E-06	3,88-06	3.28-06	2.7E-06
	6	HR	9	нін_	1.18-04	2.26-05	1.66-05	1.26-05.	1.08-05	8.68-06	7.2E-06	6.1E-06	5.38-06	4.7E-06
	7 1	HR	0	нін	1.3E-04	2.4E-05	1,7E-05	1.4E-05	1,16-05	9,58-06	8.0E-06	6.88-06	5.9E-06	5.28-06
1	0 1	ня	0	нін	1.6E-04	3.0E-05	2,18-05	1.7E-05	1.4E-05	1.26-05	1.0E-05	8.7E-06	7.68-06	6.78-06
1	2	ня	0	нін_	1.8E-09	3.4E-05	2.4E-05	1.98-05	1.6E-05	1.36-05	1,1E-05	9.7E-06	8.5E-06	7.51-06
1	4 1	HR	0	HIN	2.0E-04	3.7E-05	2.68-05	2.1E-05	L.7E-05	1.4E-05	1,26-05	1.18-05	9.38-06	9.55-06
1	6	HR	0	нін	2.1E-04	4.0E-05	2,86-05	2.28-05	1.88-05	1.66-05	1.3E-05	1.1E-05	1.0E-05	6.98-06
2	Q I	HR	Q	HIH.	2.96-99	4.5E=05	3.28-05	2.5E-05	2.16-05	1.8E-05	1.5E-05	1.3E-05	1.18-05	1.08-05
2	4	ня	0	нін	2.76-04	5.0E-05	3,58-05	2,88-05	2,38-05	1,98-05	1.6E-05	1.46-05	1.38-05	1.1E-05

WHOLE BODY DONE (REII)

. ST. LUCIE EMERGENCY PLAN

TOVERLAND DISPERSION

PASQUILL STARILITY CLASSES C & D

MINDSPEED+3 HPH

DISTANCE (HILES)

	1111			1.00	2.00	3.00	4,00	5.00	6.00	7.00	8.00	9.00	10.00
		22	HIH	1.1E-04	0.	0,	0.	0.	0.	0,	0.	٥,	0.
		30	нін	1.98-04	0.	0,	0.	٥.	٥,	٥.	0.	0.	٥.
		45	HIN	3,9E-04	6.3E-05	0.	0.	0.	0.	.0.	0.	0,	0.
1	нв	0	HIN	6.6E-04	1.46-04	0,	0.	٥.	0.	0.	0.	٥,	0.
1	HR-	-30	нін	1.01-03	3.8E=04	1.26-04	3.38-05	0.	٥,	٥.	0.	٥.	0.
2	HR	0	нін	1.46-03	5.2E-04	2.6E-04	1.26-04	3,8E-05	0.	0,	0,	0,	0,
3	HR	30	нін	1.6E-03	6.5E-04	3.48-04	1,98-04	1.1E-04	4.4E-05	1.4E-05	0.	0.	0.
	HR	0	нін	1.76-01	7.3E-04	4.2E-04	2.5E-04	1.5E-04	9.7E-05	5.0E-05	1.8E-05	٥.	0.
	HR	0	HIN	2.4E-03	1.0E-03	5.8E-04	1. AE-04	2.6E-04	1.9E-04	1.4E-04	1.1E-04	8,7E-05	7.0E-0\$
1	7 HR	Q	нін	2.6E-03	1.1E-03	6.3E-09	4.1E-04	2,98-04	2.1E-04	1.65-04	1.26-04	9.68-05	7.8E-05
10	HR G	0	HIN	3,16-03	1.38-03	7.4E-04	4.9E-04	3,4E-04	2.5E-04	1.9E-04	1.5E-04	1.26-04	9.7E-05
1,	2 HR	0	нін	3,46-03	1.4E-03	8.1E-04	5.3E-04	3,7E-04	2.8E-04	2,15-04	1,68-04	1,38-04	1.18-04
11	4 HR	0	нін	3.7E-03	1.5E-03	8.7E-04	5.7E-04	4.0E-04	3.0E-04	2.36-04	1.86-04	1.48-04	1.28-04
	6 HR	0	нін	4,06-03	1.66-03	9.2E=04	6.1E-04	4.38-04	3.2E-04	2.46-04	1.9E-04	1,56-04	1.36-04
2	0 HR	0	нін	4.4E-03	1.66-03	1.0E-03	6.7E-04	4.8E-04	3.6E-04	2.76-04	2.28-04	1,75-04	1.4E-04
2	9 HB	0	HIN	4,9E-03	1.96-03	1.1E-01	7.3E-04	5,2E-04	3.9E-04	3.0E-04	2.46-04	1.98-04	1.6E-04

PHOLE GOOY DOSE (REH)

ST, LUCIF EMERGENT PLAN

OVERLAND DISP' . LOH

PASHUILL STARILLTY CLASSES C & D

HINDSPEED . S HPH

							DISTANCE	(HILES)					
		HE		1.00	2.00	3,00	4.00	5.00	6.00	7.00	8.00	9.00	10,00
		22	нін	1,32-09	0.	٥,	0.	٥,	0,	0.	0.	٥.	٥.
		30	нін_	1,9E-04	4.4E-05	0,	۹,	0.	0.	0.	0,	٥,	٥.
	-	45	HIN	3.46-04	9.6E-05	3.2E-05	0.	0.	0.	٥,	0.	0.	0.
1	HR	0	нін	5,0E-04	1.7E-04	6,58-05	2,56-05	0.	0.	0.	0.	0.	0.
- 1	HR	-30	HIN-	7.08-04	2.9E-04	1.6E-04	8.5E-05	4,0E-05	1.8E-05	8.0E-06	0.	0,	ŏ.
5	HR	0	нін	8.8E-04	3.7E-04	2.1E-04	1.3E-04	e.4E-05	5.68-05	3.0E-05	1.58-65	7,1E-06	0.
5	HR	30	нін	1.0E-03	4.4E-04	2,6E-04	1,6E-04	1.18-04	7.76-05	5.4E-05	3,88-05	2,4E-05	1.38-05
- 3	HR-	0	HIH.	1.1E-03	4.8F-04	2,9E+04	1.98-04	1.4E-04	9.7E-05	6.9E-05	5.0E-05	3.76-05	2.86-05
6	HR	0	HIN	1.5E-01	6.5E-04	3,8E-04	2.66-04	1.8F-04	1.4E-04	1.1E-04	8.48-05	6,88-05	5.6E-05
7	HR	0	нтн	1,68-01	6.9E-04	4.1E-04	2.8E-04	2.0E-94	1.5E-04	1,18-04	9.0E-05	7.3E-05	6.1E-05
10	HR-	0	HIN-	1,98-03	B. 1E-04	4,82-04	3.2E-04	2.3E-04	1.7E-04	1.3E-04	1.18-04	8.7E-05	7.3E-05
12	HR	0	HIN	2.1E-03	8.8E-04	5.28-04	3.55-04	2.5E-04	1.98-04	1,5E-04	1,26-04	9,58-05	7,98-05
14	HR	0	HIN	2.38-03	9.4E-04	5,51-04	3.7E-04	2,76-04		1.6E-04			
-10	HR-	0	HIH.	5.4E-01	1.08-03	5,91-04	3.4E-04	2.88-04	2.1E-04	1.7E-04	1.3E-04	1.18-04	9.1E-05
20	HR	0	HIN	2,76-03	1.16-03	6,5E-04	4.3E-04	3.1E-04	2.4E-0A	1.05-04	1.56-04	1.25-04	1.02-04
24	HR	0	нін	3.02-03	1.26-03	7.0E-04	4,78-04	3.42-04	2.66-04	2.0E-04	1.6E-04	1,31-04	1.18-04

HHOLE BODY DOSE (KEH)

ST. LUCIF EMERGENCY PLAN

OVERLAND DISPERSION

PASQUILL STARILITY CLASSES C & D

HINDSPEEDER MPH

						DISTANCE	(MILES)				was a second a second	
111			1,00	2.00	3,00	4.00	5.00	6.00	7,00	4.00	9.00	16.00
	22	нін	1.1E-04	3,18-05	ο.	٥,	0,	ο,	0.	0.	٥,	0.
	30	HIN-	1.6E+04	5.0E+05	2.01-05	2.56-07	0.	0,	٥.	0.	0.	0.
		HIN	2.58-04	9.26-05	4,21-05	2.08-05	9.98-06	2.21-07	0.	0.	0,	0.
1 HR		111	3,48-09	1,48-04	7.25-05	3.88-05	2,1E-05	1.1E-05	6.0E-06	1.91-07	٥.	0.
1 HR-	30	HIH -	4.6E-04	2,0E=04	1.2E-04	7, AE-05	5,21-05	3.6E-05	2.25-05	1.38-05	6,0E-06	4.9E-06
2 HR	0	нін	5.8E-04	2.5E-04	1.56-04	1.0E-04	6.9E-05	5.0E-05	3.76-05	2.8E-05	2,1E-05	1.66-05
2 HR	30	нін	6.5E-04	2.9E-04	1.8E-04	1.28-04	8.61-05	6.4E-05	4.88-05	3.6E-05	2.0E-05	2.3E-05
3 HR		HIN	7,9E-04	3.2E-04	1.9E-04	1.3E-04	9.5E-05	7,3E-05	5.08-05	4.5E-05	3,56-05	2.98-05
6 HR	0	HIN	9.6E-04	4.2E-04	2,5E-04	1.75-04	1.3E-04	9.6E-05	7.5E-05	6.0E-05	4,91-05	4.1E-05
7 HR	0	HIN	1.0E-03	4.5E-04	2.76-04	1.0E-09	1.36-04	1.0E-04	7.9E-05	6.4E-05	5,2E-05	9.4E-05
10 HR	0	HIN-	1,28-03	5.2E-04	3.1E-04	2.1E-04	1.5E-04	1.2E-04	9.25-05	7.4E-05	6.16-05	5.1E-05
12 HR	0	HIN	1.36-03	5.6E-04	3.4E-04		1.7E-04		9,9E-05	8.0E-05		5,58-05
14 HR		HIN	1.46-03	6.0E-04	3.6E-04	2.4E-04	1.8E-04	1.4E=04	1.1E-04	8.5E-05	7.0E-05	5.98-05
16 HR		HIN		6.4E-04	3.8E-04	2.6E-04	1.96-04	1.4E-04	1.1E-04	9.0E-05	7.48-05	6.28-05
20 HR		нін	1.78-03		4.26-00	2.8E-04	2.0E-04	1.6E-04	1.2E-04	9.8E-05	0.16-05	6.98-05
24 HR	-	HIN	1.96-01	7.7E-04		1,0E+04	2.28-04	1.78-04	1.3E-04	1.1E-04	8,8E-05	7.4E-05

Page 23 of 26

22 HIN 4.1E-09 0. 2.00 3.00 30 HIN 7.3E-09 0. 0. 30 HIN 2.5E-03 3.1E-09 0. 30 HIN 2.5E-03 1.8E-03 6.5E-09 0 HIN 5.2E-03 2.2E-03 1.8E-03 0 HIN 6.1E-03 3.2E-03 1.8E-03 0 HIN 8.5E-03 3.2E-03 3.1E-03 0 HIN 9.6E-03 4.2E-03 3.1E-03 0 HIN 1.1E-02 5.2E-03 3.5E-03 0 HIN 1.1E-02 5.2E-03 3.5E-03 0 HIN 1.1E-02 5.2E-03 3.5E-03 0 HIN 1.1E-02 5.2E-03 3.9E-03	\$1, LUCIE EMERGENCY PLAN	TENGENCY P	LAH				
22 HIN 4.1E-09 0. 2.00 30 HIN 7.3E-09 0. 30 HIN 2.5E-03 3.1E-09 0 0 HIN 2.5E-03 1.8E-09 0 0 HIN 6.1E-03 3.2E-03 1 0 HIN 6.5E-03 3.2E-03 2 0 HIN 6.5E-03 3.5E-03 2 0 HIN 9.6E-03 4.2E-03 3 0 HIN 1.1E-02 5.2E-03 3 0 HIN 1.1E-02 5.2E-03 3 0 HIN 1.1E-02 5.2E-03 3	OVERLAND DISPERSION	HOISPERSION			-		
30 HIN 4.1E-09 0. 30 HIN 7.3E-09 0. 30 HIN 7.5E-03 3.1E-09 0 0 HIN 8.2E-03 1.8E-09 0 0 HIN 8.2E-03 2.5E-03 1 0 HIN 8.2E-03 3.2E-03 1 0 HIN 8.2E-03 4.2E-03 2 0 HIN 8.6E-03 4.2E-03 2 0 HIN 9.6E-03 4.4E-03 2 0 HIN 9.6E-03 4.9E-03 3 0 HIN 1.1E-02 5.2E-03 3 0 HIN 1.1E-02 5.2E-03 3	PASQUILL STARILITY	ITY CLASSES					
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THBLE D-17

WHOLE BODY DOSE (REII)

SI, LUCIE EMERGENCY PLAN

PASOUILL STARILITY CLASSES E & F

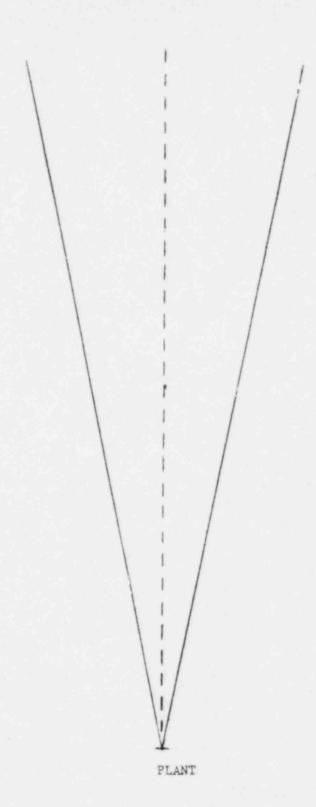
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2	20 HK	0	H1H 0	5.0E-01	- 90	1,65-03	1.35-03	2,5E-03 1,6E-03 1.2E-03 0,7E-04 7.0E-04 5,6E-04 4,6E-04 3,9E-04 3,4E-04	7.0E-04	\$ 05-04	A, 6E-04	1,96-04	3,46-04
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OVERLAY - 22 1/2° SECTOR

Figure D-1

FLORIDA POWER & LIGHT COMPANY ST. LUCIE PLANT UNIT NO. 1 EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100026E, REV. 6

1.0 TITLE:

CRITERIA FOR AND CONDUCT OF EVACUATIONS

2.0 APPROVAL:

Reviewed by Facility Review Group_ Approved by K. N. Harris	
Revision 5 Reviewed by Facility Approved by Clarker 2011	Review Group March 20 & 25,1981 V.P. Pwr. Res. March 1977
Revision & Reviewed by Recipion Re	view Group May 28 1981 V.P. Pwr. Res. June 2, 1987

3.0 SCOPE:

3.1 Purpose

This procedure provides criteria for determining if evacuation of a local area on-site or the Owner Controlled Area should be carried out. It provides instructions for effecting an ordered, rapid and safe evacuation of a local area or the Owner Controlled Area in order to prevent of minimize radiation exposure to personnel. It also provides instructions for personnel accountability.

3 2 Discussion

3.2.1 Accorate classification of emergencies that may occur at the Plant is necessary to enable the Nuclear Plant Supervisor to take the proper corrective action and to know when to initiate the Emergency Plans. For those emergencies involving high radiation and/or the release of radioactive material to the environment, it is necessary to provide the Nuclear Plant Supervisor (Emergency Coordinator) with guidelines that wall enable him to know when to order a local area evacuation or evacuation of the Owner Controlled Area.

3.0 SCOPE: (Continued)

3.2 Discussion (Continued

3.2.2 A local area evacuation or evacuation of the Owner Controlled Area may be necessitated by the Occurrence of any of several events where evacuation of personnel would be necessary to minimize their exposure to radiation. An Owner Controlled Area evacuation will be conducted for any Site Area Emergency or General Emergency.

3.3 Authority:

This procedure implements the St. Lucie Unit #1 Emergency Plan.

3.4 Definitions:

3.4.1 Assembly Area

The location to which evacuation personnel report. In a local evacuation, personnel evacuate and assemble in a location designated by the Emergency Coordinator. In an evacuation of the Owner Controlled Area, evacuating FPL personnel from the Protected Area go to the Site Assembly Station on Highway AlA near the north entrance to plant property, while non-FPL personnel and FPL personnel from outside the Protected Area assemble at J.C. Public Park on Highway AlA, 7 1/2 miles north of the plant. If necessary, other assembly areas can be specified by the Emergency Coordinator.

3.4.2 Escort

An individual specifically assigned to accompany other persons who are required by the Security Plan or Health Physics Manual to be escorted.

3.4.3 Loral Area Evacuation

An evacuation of some portion of, but not all of, the Owner Controlled Area. It may include evacuation of a room, building, group of buildings, generating station area, or other portion of the Owner Controlled Area.

3.0 SCOPE: (Continued)

3.4 Definitions: (Continued)

3.4.4 Owner Controlled Area

That portion of FPL property surrounding and including the plant which is subject to limited access and control as deemed appropriate by FPL (indicated by the dotted line on the attached Florida Power & Light property plan, Figure 1 of this procedure).

3.4.5 Owner Controlled Area Evacuation

The evacuation from the Owner Controlled Area of all personnel except those required to place the plant in a safe condition, the emergency teams, and the guards necessary to fulfill their evacuation responsibilities.

3.4.6 Radiation Control Point

Located in the Auxiliary Building, it is likely to be used as an assembly area during a local evacuation.

4.0 Precautions:

- 4.1 Every effort shall be made to minimize personnel exposure to radiation.
- 4.2 Personnel who have been in the area of the emergency shall remain in a group, and shall not mix with the other personnel in the Radiation Control Point or other assembly area until they have been monitored for possible contamination, unless they are injured. Injured personnel shall be treated as discussed in E-Plan Implementing Procedure 3100021E, Duties of the Emergency Coordinator.
- 4.3 Except as discussed in 4.2 above, personnel reporting to the Radiation Control Point or other assembly area shall group according to departments in order to facilitate personnel accountability and monitoring.
- 4.4 When reporting to the assembly area, take the most direct route possible, but do not use any route or area that has been declared part of the emergency area, or that could result in excessive individual exposure or injury.
- 4.5 If an area or building is evacuated, any records regarding the persons in the area shall be removed and taken to the assembly area.

4.0 Precautions: (Continued)

4.6 Health Physics personnel exiting the RAB as necessitated by an emergency condition of possible radiological nature shall pick up suitable portable radiation monitoring equipment which would be required for re-entry monitoring.

5.0 Responsibilities:

- 5.1 The Nuclear Plant Supervisor is responsible for:
 - 5.1.1 Classifying any emergency that may occur at Unit #1.
 - 5.1.2 Initiating the Emergency Plans when appropriate.
 - 5.1.3 Initiating the evacuation of any area in which the criteria for evacuation, as expressed in steps 8.1 8.2 are met.
 - 5.1.4 Providing protective action recommendations to off-site /R6 authorities per Table 1 in 3100021E(Duties of Emer. Coordinator).
- 5.2 The Nuclear Plant Supervisor shall act as Emergency Coordinator.
- 5.3 The Emergency Coordinator is responsible for advising the Security Team Leader of a local evacuation or an Owner Controlled Area evacuation. This notification will be by the alarm or the PA system. Information needed by the Security Force to properly fulfill their responsibilities during the evacuation shall be provided to the Security Team Leader by the Emergency Coordinator. (See E-Plan Implementing Procedure 3100021E, Duties of Emergency Coordinator).
- 5.4 The Security Guard Force is under the supervision of the Plant Security Supervisor. He serves as the Primary Security Team Leader. The Security Force Shift Supervisor is the Interim Security Team Leader.
- 5.5 The responsibilities of individuals acting as escorts are contained in Sections 5.12 and 8.4.5 of this procedure.
- 5.6 All personnel permanently or temporarily assigned to St. Lucie plant shall familiarize themselves with:
 - 5.6.1 The instructions given in this procedure.
 - 5.6.2 The access routes to the work location to which they are assigned.

5.0 Responsibilities:

5.6 (Continued)

- 5.6.3 The locations of the evacuation assembly areas:
 - a. The Radiation Control Point in the Auxiliary Building.
 - b. The Site Assembly Station.
 - c. The J. C. Public Park on Route AlA, north of the plant site.
- 5.7 It shall be the responsibility of each supervisor and department foreman not involved in the emergency response to report to the assembly area to assist in accounting for all personnel under his supervision.
- 5.8 Security Department Responsibilities are defined in Security Procedure 0006123.
- 5.9 The Control Point Guard shall maintain accurate records of all personnel entering or leaving the controlled area and make them available when required.
- 5.10 The Main Gate Guard shall maintain accurate records of all personnel entering or leaving the Generating Station Area and make them available when required.
- 5.11 The Assembly Area Supervisor shall be responsible for personnel accounting at assembly area and liason between these personnel and the Emergency Coordinator.
- 5.12 Tour Guides are assigned to escort groups of visitors during tours of the plant site. In an emergency, Tour Guides are responsible for personnel accountability of their respective groups and the evacuation of their group to the Site Assembly Station in cooperation with the Security Force.
- 5.13 Unit 2 Responsibilities are defined in ASP 5.0.

6.0 References:

- 6.1 St. Lucie Plant Radiological Emergency Plan.
- 6.2 St. Lucie Plant Security Plan.
- 6.3 E-Plan Implementing Procedure 3100021E, Duties of Emergency Coordinator.

7.0 Records and Notifications:

- 7.1 A list of all persons assembled at J.C. Public Park shall be prepared by a member of the Plant Staff who will travel to that location with evacuees.
- 7.2 Records showing that all personnel in the area affected by the emergency were accounted for shall be kept as part of the records of the emergency and recorded by the Emergency Coordinator.

8.0 Instructions:

8.1 Criteria for Local Area Evacuation

If an incident involves a high radiation field or has caused the release of radioactive material to the environment, the need to evacuate the affected area shall be determined in accordance with the following criteria:

Evacuate the affected <u>local area</u> in which any of the following conditions occur:

- 8.1.1 Area Radiation Monitor Alarm.
- 8.1.2 Containment Evacuation Alarm.
- 8.1.3 Unevaluated direct radiation dose rate reading in excess of 100 mRem/hour.
- 8.1.4 Unevaluated airborne radioactivity concentration in excess of 1 X 10⁻⁹ micro Ci/cc.
- 8.1.5 Removable radioactive surface contamination in an unposted area in excess of 1000 dpm/100 cm² beta-gamma over an area of 100 ft².
- 8.1.6 Removable radioactive surface contamination in an unposted area in excess of 50 dpm/100cm² alpha over an area of 100 ft².
- 8.1.7 The Emergency Coordinator determines that a situation exists for which local evacuation is appropriate.

8.2 Criteria for Owner Controlled Area Evacuation

The Owner Controlled Area shall be evacuated in the following circumstances:

- Site Area Emergency,
- 2. General Emergency,
- If the Emergency Coordinator determines that the entire Protected Area should be evacuated,
- 4. Any other situation in which the Emergency Coordinator determines that evacuation is appropriate.

8.0 Instructions: (Continued)

8.3 Evacuation Preparedness

- 8.3.1 All visitors shall have adequate transportation available on-site to evacuate all members of their respective groups.
- 8.3.2 Escorts shall maintain control of groups or individuals to which they are assigned to enable all members of the groups to be located, notified and evacuated.

8.4 Local Area Evacuation Implementation

- 8.4.1 When the Emergency Coordinator (Nuclear Plant Supervisor) determines that a local evacuation is necessary, he shall, in following Emergency Procedure Duties of the Emergency Coordinator, announce an evacuation using the PA system, giving the area affected, the assembly area and other instructions as required.
- 8.4.2 All personnel in the evacuated area shall stop work, turn off potentially hazardous equipment, such as cutting torches, and leave the area by the same route taken to enter the area, unless otherwise instructed by the Emergency Coordinator.
- 8.4.3 All personnel in the evacuated area shall report to the Radiation Control Point in the Auxiliary Building for monitoring and accountability.
- 8.4.4 The Radiation Monitor Team shall monitor evacuated personnel for contamination, (for areas where contamination may be present).
- 8.4.5 Injured personnel shall be treated as discussed in E-Plan Implementing Procedure 3100021E, Duties of Emergency Coordinator. If an injured person is evacuated from the plant site, the person's TLD, selfreading dosimeter, identification badge, and keycard should be removed and given to the First Aid and Personnel Decontamination Team Leader.
- 8.4.6 The Control Point Guard shall make available to the Emergency Coordinator his records showing the personnel who are in the Security Control ed Area.

8.0 Instructions: (Continued)

8.4 Local Area Evacuation Implementation (Continued)

- 8.4.7 The Emergency Coordinator and department supervisors and foremen having personnel working in the evacuated area shall verify that all personnel in the evacuated area are accounted for.
- 8.4.8 The Emergency Coordinator shall initiate a search for the personnel who have not been accounted for.

8.5 Owner Controlled Area Evacuation Implementation

- 8.5.1 When the Emergency Coordinator (Nuclear Plant Supervisor) determines that an evacuation of the Owner Controlled Area is necessary, he will, in the following Emergency Procedure Duties of the Emergency Coordinator, order an evacuation using the PA system and the evacuation alarm.
- 8.5.2 All personnel in the evacuated area shall stop work, turn off potentially hazardous equipment, such as cutting torches, and leave the area. All personnel not needed to place Unit #1 in a safe condition and not on an emergency team or otherwise required on site, shall evacuate to the appropriate Assembly Area. Emergency Teams shall report to the administrative Building for assignment.
- 8.5.3 Injured personnel shall be treated as discussed in E-Plan Implementing Procedure 3100021E, Duties of Emergency Coodinator. If an injured person is evacuated from the plant site, the person's TLD, selfreading dosimeter, identification badge, and keycard should be removed and given to the First Air and Personnel Decontamination Team Leader.
- 8.5.4 Personnel shall remove protective clothing prior to leaving the Radiation Controlled Area.
- 8.5.5 Proceed to the Main Guard Station by the most direct route, unless an alternate route is specified by the Emergency Coordinator.
- 8.5.6 Each person shall then proceed to the Site Assembly Area (see Figure 2).

Page 10 of 13

FLORIDA POWER & LIGHT COMPANY EMERGENCY PLAN IMPLEMENTING PROCEDURE #3100026E, REV. 6

8.0 Instructions: (Continued)

8.5 (Continued)

- 8.5.7 Assemble by department for accounting and monitoring, and remain at the assembly area until given instructions by the Assembly Area Supervisor.
- 8.5.8 If the conditions associated with the emergency are such that the site assembly station area is untenable, the Emergency Coordinator shall direct all personnel to report to the J. C. Public Park and specify the route to be followed.
- 8.5.9 The department supervisors and foremen shall account for their personnel and report the results to the Assembly Area Supervisor.
- 8.5.10 The Assembly Area Supervisor shall assure that all personnel are monitored for contamination prior to leaving the Assembly Area.
- 8.5.11 The Assembly Area Supervisor, the Security Force Shift Supervisor and the department supervisors and foreman shall examine all the personnel records available to determine if any personnel are unaccounted for.
- 8.5.12 The Emergency Coordinator shall initiate a search for the personnel who have not been accounted for.

8.6 Security Force Evacuation Implementation

- 8.6.1 Upon notification of an evacuation, the Security Force Shift Supervisor (Interim Security Team Leader) shall advise by radio all other guards on duty that an evacuation is in progress. The Security Force will proceed with the evacuation of persons and with personnel accountability activities.
- 8.6.2 The Control Point Guard and the Main Gate Guard shall immediately review logs and badge records to determine what persons are in the affected area. These guards will be prepared to brief the Emergency Coordinator. The Control Point Guard shall verify that all personnel have evacuated the area and report to the Security Team Leader for further instructions. The Control Point Guard shall

8.0 Instructions: (Continued)

8.6 (Continued)

- 8.6.2 go to the Main Gate with his personnel records and await instructions from the Security Team Leader.
- 8.6.3 The Security Team shall:
 - Sweep the plant site to verify that all personnel have evacuated.
 - 2. Notify any Special Guards on duty.
 - Perform additional duties as specified in the Security Guard Force Post and Force Orders.
- 8.6.4 The Security Team Leader, after completion of sweeps by members of the Security Team shall instruct guards concerning special assignments which may be required as a result of the evacuation. He shall advise the Off-Site Emergency Security Manager of any circumstances which require his attention.

FIGURE 1

St. Lucie Plant Owner Controlled Area

FIGURE 2

Site Assembly Area