

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	M	E	M	Y	P	1	0	0	-	0	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33
LICENSEE CODE														LICENSE NUMBER						LICENSE TYPE						CAT 58

01	L	0	5	0	0	0	0	3	0	9	0	2	1	8	8	3	0	3	0	1	8	3	9		
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
CON'T			REPORT SOURCE			DOCKET NUMBER						EVENT DATE						REPORT DATE							

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | SEE ATTACHED

03 |

04 |

05 |

06 |

07 |

08 |

09	X	X	A	F	X	X	X	X	X	X	Z	Z				
7	8	9	10	11	12	13	14	15	16	17	18	19	20			
SYSTEM CODE			CAUSE CODE		CAUSE SUBCODE			COMPONENT CODE					COMP. SUBCODE		VALVE SUBCODE	
17	8	3	—	0	0	5	—	0	1	T	—	1				
7	8	9	10	11	12	13	14	15	16	17	18	19	20			
LER/RO REPORT NUMBER			EVENT YEAR		SEQUENTIAL REPORT NO.			OCCURRENCE CODE		REPORT TYPE		REVISION NO.				
X	Z	Z	Z	0	0	0	Y	N	X	6	6	1	0			
7	8	9	10	11	12	13	14	15	16	17	18	19	20			
ACTION TAKEN			EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER			
FUTURE ACTION			EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | SEE ATTACHED

11 |

12 |

13 |

14 |

15	6	0	0	0	N/A	D	Notification from Vendor
7	8	9	10	11	12	13	14
FACILITY STATUS			% POWER			METHOD OF DISCOVERY	
ACTIVITY CONTENT			AMOUNT OF ACTIVITY			LOCATION OF RELEASE	
16	Z	Z	N/A	N/A	N/A	N/A	N/A
7	8	9	10	11	12	13	14
PERSONNEL EXPOSURES			PERSONNEL INJURIES				
17	0	0	0	Z	N/A	N/A	N/A
7	8	9	10	11	12	13	14
LOSS OF OR DAMAGE TO FACILITY			PUBLICITY				
18	0	0	0	N/A	N/A	N/A	N/A
7	8	9	10	11	12	13	14
LOSS OF OR DAMAGE TO FACILITY			ISSUED DESCRIPTION				
19	Z	N/A	N/A	N/A	N/A	N/A	N/A
7	8	9	10	11	12	13	14

20	N	N/A	8303160356	830301	NRC USE ONLY
7	8	9	10	11	12
PUBLICITY			NRC USE ONLY		
ISSUED DESCRIPTION			PDR ADQCK 05000309		
ISSUED DESCRIPTION			PDR		

GPO 917-926

LER #83-005/01T-1

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

Control Data Corporation discovered an inaccuracy in the version of the PDQ Computer code used by Maine Yankee to calculate coefficients for the INCA incore analysis code. This code version is provided and maintained by Control Data Corporation under contract to the Yankee Nuclear Services Division.

The error affected coefficients relating maximum one pin power to assembly power, which are used to calculate total radial peaking factors and linear heat generation rates from incore detector signals. The inaccurate coefficients caused the calculations to be slightly less conservative, however, the error did not result in operations beyond specified acceptable fuel design limits. There was no effect on public health and safety.

CAUSE DESCRIPTION AND CORRECTION ACTIONS (27)

The inaccuracy was due to a coding error in the Control Data Corporation code.

A corrected version of the PDQ code was obtained, and the calculation of the coefficients for the INCA code was repeated. The corrected coefficients were provided to the plant and promptly incorporated into the INCA code. Comparisons have shown that the maximum error resulting from the plant specific application did not exceed 0.6% for any core location. Applicable Technical Specification LCO's were not violated at any time during Cycle 7.

The allowances for uncertainty built into the codes are approximately ten times greater than the differences observed between the corrected and uncorrected versions. These allowances for uncertainty have been retained in the final corrected version.