U.S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) UPDATE REPORT LICENSEE EVENT REPORT PREVIOUS REPORT DATE: 02-07-83 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 3 0 0 0 0 0 0 0 0 0 (2)0 D C C N LICENSE NUMBER LICENSEE CODE CON'T REPORT (7)[0]1 1 (8)01 3 0 011 8 L (6) 0 31 0 5 0 0 SOURCE EVENT DATE REPORT DATE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During startup operations at 1145, while attempting to feed the steam 0 2 generators (SGs) it was discovered that #22 Auxiliary Feedwater Pump 0 3 (AFW) pump could not deliver any flow to the SGs, and was declared 0 4 inoperable (T.S. 3.7.1.2.b). Repairs were conducted and the pump was 0 5 returned to operability at 0330 on 1-13-83. #21 Auxiliary Feedwater 0 6 Pump remained operable during this event. Similar events: none. COMP SYSTEM CAUSE CAUSE VALVE SUBCODE COMPONENT CODE SUBCODE B (13) Z (15 Z (16) (12 C F U N (14) E WE E 18 REVISION OCCURRENCE SEQUENTIAL REPORT REPORT NO. CODE NO EVENT YEAR TYPE LER/RO 01 REPORT 0 0 4 3 Х 3 8 NUMBER COMPONENT EFFECT ON PLANT SHUTDOWN METHOD ATTACHMENT SUBMITTED NPRD-4 PRIME COMP FUTURE ACTION. (22) HOURS FORM SUB SUPPLIER 9 10 0 0 Y (24) A (25 (26)(23 18 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Testing of the AFW pump at rated flow, discharge pressure and steam conditions to the turbine can not be performed until the RCS reaches 532°F. Consequently, testing conducted prior to entering Mode 3 (< 300°F) may not be an accurate indication of the pumps performance at normal steam generator conditions. 4 80 METHOD OF DISCOVERY FACILITY (30) DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER (31 Operator Observation ACTIVITY CONTENT 80 LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED_OF RELEASE Z 33 Z 34 N/A 44 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 0 (37) Z (38) 01 01 N/A 86 PERSONNEL INJURIES DESCRIPTION (41 0 (40) N/A 8303160184 830307 PDR ADOCK 05000318 PDR 80 LOSS OF OR DAMAGE TO FACILITY DESCRIPTION Z (42) N/A 80 PUBLICITY NRC USE ONLY DESCRIPTION (45 UED N (44) N/A 60 80 PHONE (301) 269-4850/4504 NAME OF PREPARER G. S. Pavis/E. V. Farrell

LER NO. 83-04/3X, Rev. 1 DOCKET NO. 50-318 LICENSE NO. DPR 69 EVENT DATE 01-12-83 REPORT DATE 03-07-83 ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

During the refueling outage #22 Auxiliary Feedwater Pump Turbine governor was replaced because of a leaking seal oil ring. Following the governor replacement its linkage was not properly adjusted. Although the pump was subsequently started on 150 psig, auxiliary steam and utilized to fill the depressurized steam generators, insufficient steam pressure was available to demonstrate that the pump was capable of delivering the required flow to the steam generators at their 900 psig normal operating pressure. Following plant heatup, sufficient steam pressure was available to conduct a full operational test. At that time it was discovered that #22 auxiliary feedwater pump failed to develop sufficient discharge pressure to feed the steam generators. Our inability to adequately test all facets of AFW pump operation at low temperature and pressure is recognized by the Technical Specifications which only requires performance of such tests prior to entry into Mode 1. However, we will continue our current practice of testing the pumps to the extent possible on low pressure steam prior to entry into Mode 3. Although such testing failed to disclose the improperly adjusted governor linkage in this instance, it was sufficient to disclose most problems which could impair pump operability. Additionally, since this event, a third motor driven auxiliary feedwater train has been declared operational. The motor driven pump can be proven fully operable prior to entering Mode 3.