

54

# Woodward-Clyde Consultants

March 25, 1980

Dr. K. N. Jabbour  
SEP Equipment and Piping  
Systematic Evaluation Program Branch  
Division of Operating Reactors  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Dr. Jabbour:

I have reviewed the "Seismic Analysis of Reactor Internals for Dresden II" dated December 24, 1968 which was transmitted by Mr. Janecek's letter dated 20 February 1980. While the report goes into considerable detail concerning the dynamic analysis procedures and assumptions used in analyzing the reactor vessel internals and supports, the only seismic input to the analysis is N-S El Centro 1940 time history normalized to 0.1g zero period ground acceleration. In addition, while the model seems to indicate that the building structure-reactor vessel interaction was considered, no mass coefficient for the building structure was defined in the input properties to the analysis.

The vessel skirt moment of 12,800 K-ft. and shear of 728 Kips determined in the subject report compare to the values of 25, 200 K-ft. and 821 Kip determined in a John A. Blume report dated February 2, 1966 which was used by B&W in their Stress Analysis evaluation of the Support Skirt. Of particular interest in the further review of the design adequacy of the reactor vessel internals and support would be John Blume's report where the basis for the 25,800 K-ft. and 821 Kip loads used in the stress analysis could be evaluated. Unfortunately, the material supplied in the subject report while useful in defining the mass and stiffness characteristics of the vessel internals and supports does little to answer the design adequacy questions raised in the current NUREG/CR-0891 Section 6.3.1.8 report.

Sincerely,

*John D. Stevenson*  
John D. Stevenson  
Vice President & General Manager

X003  
3  
1/0

cc: R. Murray  
JDS/1

800 40/0318 XA