U.S. NUCLEAR REGULATORY COMMISSION
REGION I

DOCKET/REPORT NO: 50-219/94-08 (OL)

LICENSEE: GPU Nuclear Corporation

FACILITY: Oyster Creek Nuclear Power Station

DATES: April 5-7, 1994

EXAMINERS: A. Burritt, Operations Engineer
R. Orton, Examiner (PNL)

CHIEF EXAMINER: Arthur L. Burritt 4-26-94

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PWR & BWR Section
Division of Reactor Safety

Date

APPROVED BY: Glenn W. Meyer 5-17-94

Glenn W. Meyer, Chief
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Date

EXAMINATION SUMMARY

Report No. 50-219/94-08

Operations

Initial examinations were administered at Oyster Creek to one Senior Reactor Operator (SRO) instant applicant and one SRO upgrade applicant. Both applicants passed all portions of the examination. Generic strengths and weaknesses were not identified due to the small number of applicants.

The simulator cause and effect documentation, part of the reference material submitted for examination preparation, was poor (Section 3.0).

Maintenance

The corrective actions on a previous violation regarding GE 4160 volt breaker prop spring failure were sufficient to close the item; however, corrective actions to prevent recurrence were weak (Section 4.2). In addition, the inspector concluded that the 1992 GPUN violation response letter was inaccurate, but that the inaccuracy had minimal regulatory significance.

Engineering

The corrective actions on a previous inspection finding regarding maintaining configuration control of equipment modified or overhauled by a vendor were sufficient to close the item (Section 4.3).

DETAILS

1.0 INTRODUCTION

The NRC staff administered initial examinations to one Senior Reactor Operator (SRO) instant applicant and one SRO upgrade applicant. The examinations were administered in accordance with NUREG-1021, "Operator Licensing Examiner Standards," Revision 7.

2.0 PREEXAMINATION ACTIVITIES

The facility reviewed the written examinations during the week of March 21, 1994. The simulator scenarios and job performance measures (JPMs) were also validated during the week of March 21, 1994, on the facility's simulator and in the plant. The facility staff who were involved with these reviews signed security agreements to ensure that the initial examinations were not compromised.

3.0 EXAMINATION RESULTS AND RELATED FINDINGS.

3.1 Examination Results

The results of the examinations are summarized below:

Category	SRO Pass/Fail
Written	2/0
Operating	2/0
Overall	2/0

3.2 Facility Generic Strengths and Weaknesses

A summary of generic strengths and weaknesses was not identified due to the small number of applicants. No conclusive training program feedback could be derived with the limited amount of observations and examination data.

3.3 Reference Material

The examiner identified that the simulator cause and effect documentation, part of the reference material submitted for examination preparation, was poor. This document did not have the attributes described in the guidelines of ES-201, Attachment 2 of NUREG-1021, "Operator Licensing Examiner Standards," Rev. 7. Specifically, the information provided did not contain a concise description of the expected result or range of results that will

occur upon initiation of a malfunction (i.e., integrated response). Additionally, the malfunction codes and document index did not allow direct reference of applicable cause and effect documentation.

The operations training supervisor agreed to review their simulator cause and effect documentation and incorporate the attributes listed above as appropriate.

3.4 Post-Examination Facility Comments

Oyster Creek provided no post-examination comments on the written examination.

4.0 PREVIOUS INSPECTION FINDINGS

4.1 (Closed) Violation (219/92-07-01): GE 4160 Volt Breaker Prop Spring Failure.

The inspector reviewed corrective actions for a failed prop spring in a 4160 volt Magna-Blast circuit breaker. GPUN stated that all 4160 volt Magna-Blast circuit breakers with more than 1000 cycles, in a safety-related application, have been overhauled in General Electric shops. The inspector reviewed the "4160V Circuit Breaker PM Matrix," dated October 28, 1993, an excerpt from GPUN purchase order 0375157, and verified that the overhaul included replacement of the old style prop spring. GPUN also stated that 40 of the 53 total 4160 volt Magna-Blast circuit breakers at Oyster Creek have had the old style prop springs replaced, and the remaining breakers are scheduled for overhaul by the end of outage 15R.

The inspector also reviewed corrective steps taken to prevent further violations and concluded some corrective actions were weak. GPUN revised Procedure 125, "Conduct of Engineering," which specifies that an annual review of open backlog action items be performed. There is no specified documentation of this review; however, the inspector reviewed memorandum 5910-94-008, "Review of Open Work Assignments," which provided evidence that the reviews were being done. The inspector also discussed revision of action item requirements and due dates with the GPUN staff, and determined that there was no procedurally specified concurrence of the assigned manager when changes are made in opposition to the position stated in the 1992 GPUN Notice of Violation response. The licensing manager stated that the concurrences were done in practice; however, he stated that the response letter was inaccurate and committed to revise the GPUN response. The inspector concluded that the term, "procedurally require," had been inappropriately used to describe existing practices and that the inaccuracy had minimal regulatory significance.

The inspector concluded that GPUN has taken adequate action to address the prop spring failures, and this item is closed based on the overhaul and replacement of the old style prop springs in all breakers in a safety-related application with more than 1000 cycles. However, the inspector also concluded that the corrective actions to prevent recurrence were weak based on not procedurally specifying concurrence of the assigned manager, when changes are made to action items and documentation of the annual review of open backlog action items.

4.2 (Closed) Unresolved Item (219/92-07-02): Configuration Control of Vendor Equipment Work.

The inspector reviewed the revised procedural controls in place to ensure that configuration control is maintained when equipment is overhauled by a vendor. GPUN specifies in Procedure 125.2, "Conduct of Procurement Engineering," that for items requiring repair or overhaul, the procurement engineer shall specify that no modifications or changes be made without prior approval of GPUN. Additionally, the inspector reviewed a purchase order, and verified that the approved modifications were clearly identified, and that the purchase order also stated any additional changes needed to be approved by GPUN prior to performing the work. Further, Procedure 125.2 specified that if equipment modifications or changes are authorized, the equipment is to be placed on engineering hold to ensure appropriate documentation is prepared prior to release of the equipment for installation in the plant.

The inspector concluded that the revised procedure provided effective control for equipment modifications or overhauls performed by vendors. This item is closed.

5.0 EXIT MEETING

An exit meeting was conducted on April 7, 1994. The NRC examiners noted that plant housekeeping conditions were good and there were no problems with plant access. One problem with the examination reference material provided was discussed.

Key GPUN persons contacted and attendees at the exit meeting are listed below:

Licensee Personnel

* G. Busch, Manager, Licensing
* G. Cropper, Manager, Operations Training
* J. Kowalski, Manager, Plant Training
* S. Levin, Director, Operations & Maintenance
* P. Scallon, Manager, Plant Operations
* P. Thompson, Site Audit Manager
* H. Tritt, Lead Instructor, Operations

* Denotes those present for the exit meeting on April 7, 1994.

ATTACHMENT 1

SRO EXAMINATION AND ANSWER KEY

MASTER

Nuclear Regulatory Commission
Operator Licensing
Examination

This document is removed from
Official Use Only category on
date of examination.

NRC RULES AND GUIDELINES FOR LICENSE EXAMINATIONS

During the administration of this examination the following rules apply:

1. Cheating on the examination will result in a denial of your application and could result in more severe penalties.

2. After you complete the examination, sign the statement on the cover sheet indicating that the work is your own and you have not received or given assistance in completing the examination.

3. To pass the examination, you must achieve a grade of 80 percent or greater.

4. The point value for each question is indicated in parentheses after the question number.

5. There is a time limit of 4 hours for completing the examination.

6. Use only black ink or dark pencil to ensure legible copies.

7. Print your name in the blank provided on the examination cover sheet and the answer sheet.

8. Mark your answers on the answer sheet provided and do not leave any question blank.

9. If the intent of a question is unclear, ask questions of the examiner only.

10. Restroom trips are permitted, but only one applicant at a time will be allowed to leave. Avoid all contact with anyone outside the examination room to eliminate even the appearance or possibility of cheating.

11. When you complete the examination, assemble a package including the examination questions, examination aids, and answer sheets and give it to the examiner or proctor. Remember to sign the statement on the examination cover sheet.

12. After you have turned in your examination, leave the examination area as defined by the examiner.

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QUESTION: 001 (1.00)

The reactor is operating at 100% power. The Feedwater Hydrogen Injection system has been operating for 30 minutes. WHICH ONE of the following is the result of initiating Isolation Condenser "A"?

a. High radiation in the Isolation Condenser area.

b. Explosive concentration of hydrogen in the Isolation Condenser.

c. Water hammer in the Isolation Condenser U-tubes.

d. Decrease in heat removal capacity of the Isolation Condenser.

QUESTION: 002 (1.00)

Technical Specifications require that the shell side of each Isolation Condenser contain a minimum volume of 22,730 gallons. WHICH ONE of the following levels corresponds to this volume?

a. 4.8 feet

b. 6.3 feet

c. 7.2 feet

d. 7.7 feet

QUESTION: 003 (1.00)

During a LOCA the Automatic Depressurization System has actuated and all EMRVs are open. WHICH ONE of the following will close the EMRVs?

a. The Core Spray Booster Pumps trip.

b. Reactor water level increases to 97 inches above TAF.

c. Placing the Drywell High Pressure Reset Switches to RESET.

d. Placing the Timer Reset switches to RESET.

QUESTION: 004 (1.00)

125 VDC power has been lost to Panels D and F. WHICH ONE of the following describes the operational status of the ADS?

a. Prevents automatic initiation of ADS and remote manual operation of all EMRVs.

b. Prevents automatic initiation of ADS but remote manual operation of the EMRVs is still possible.

c. Automatic initiation of ADS is still possible but prevents remote manual operation of the EMRVs.

d. Remote manual operation of the EMRVs and automatic initiation of ADS will operate normally without position indication.

QUESTION: 005 (1.00)

WHICH ONE of the following will initiate the Standby Gas Treatment System? (Assume all time delays have timed out.)

a. Drywell pressure 2.75 psig.

b. Reactor water level 96 inches above TAF.

c. Reactor Building Vent radiation level 10 mrem/hr.

d. Refuel Floor radiation level 75 mrem/hr.

QUESTION: 006 (1.00)

WHICH ONE of the following Standby Gas Treatment System (SBGTS) components is adversely affected if the SBGTS is operated while the reactor building is being painted?

a. Orifice Purge Valves

b. Electric Heating Coils

c. Charcoal Adsorbers

d. HEPA Filters

QUESTION: 007 (1.00)

WHICH ONE of the following could result in exceeding the transient MCPR limits?

a. Mode switch in RUN, IRM range 10, recirculation flow rate is 32 mlb/hr.

b. Mode switch in RUN, IRM range 9, recirculation flow rate is 41 mlb/hr.

c. Mode switch in STARTUP, IRM range 10, recirculation flow rate is 36 mlb/hr.

d. Mode switch in STARTUP, IRM range 9, recirculation flow rate is 42 mlb/hr.

QUESTION: 008 (1.00)

During refueling operations, WHICH ONE of the following is the reason for maintaining the core plate differential pressure below 4 psid while changing recirculation flow?

a. To avoid recirculation pump cavitation.

b. To eliminate nuclear instrumentation vibration.

c. To prevent blade guide lift.

d. To reduce flow perturbations.

QUESTION: 009 (1.00)

During an APRM surveillance test, APRM channel 2 is bypassed and the Input Selector Switch is placed in the COUNT position. The meter reads 50%. WHICH ONE of the following is the number of LPRMs that are bypassed for that APRM channel?

a. 2

b. 3

c. 4

d. 5

QUESTION: 010 (1.00)

During reactor startup all APRMs are on scale except for APRM channel 4 which has been bypassed. The mode switch is in the STARTUP position. APRM channel 8 subsequently fails downscale. WHICH ONE of the following describes the consequences of this event?

a. Rod block.

b. Reactor scram.

c. One RPS trip system is inoperable.

d. One quadrant of the core has no power monitoring.

QUESTION: 011 (1.00)

WHICH ONE of the following will cause the SRM Retract Permit lights on Panel 4F to illuminate?

a. SRM HI-HI alarm is received.

b. All IRMs are on range 8.

c. SRM channels are reading greater than 500 cps.

d. All IRMs are greater than 50% of full scale on range 1.

QUESTION: 012 (1.00)

The reactor is being started up and is approaching critical. WHICH ONE of the following is the result of a loss of 24/48 VDC Power Panel A?

a. SRM Rod block.

b. SRM PERIOD SHORT alarm.

c. SRM chart recorder will not advance.

d. Inability to insert or withdraw SRM detectors.

QUESTION: 013 (1.00)

Reactor startup and turbine warmup are in progress with the following conditions:

Mode switch is in STARTUP
Reactor pressure is 750 psig
Third stage extraction steam pressure is 25 psig

WHICH ONE of the following reactor scrams are bypassed?

a. APRM Inop

b. Turbine trip

c. MSIV closure

d. Low condenser vacuum

QUESTION: 014 (1.00)

The reactor is operating at 100% power. During a surveillance the RPS I subchannels are tripped. WHICH ONE of the following is the status of the Scram Pilot Solenoid Valves (117 and 118) and the Alternate Rod Injection (ARI) Solenoid Valves?

a. 117 - deenergized
   118 - energized
   ARI Valves - energized

b. 117 - deenergized
   118 - energized
   ARI Valves - deenergized

c. 117 - energized
   118 - deenergized
   ARI Valves - energized

d. 117 - energized
   118 - deenergized
   ARI Valves - deenergized

QUESTION: 015 (1.00)

Data is received on the Standby Liquid Control System. WHICH ONE of the following requires reactor shutdown within 24 hours?

a.	Weight Percent Sodium Pentaborate	12%
	Volume of Solution	2070 gallons
	Solution Temperature	89 deg F
b.	Weight Percent Sodium Pentaborate	18%
	Volume of Solution	1720 gallons
	Solution Temperature	93 deg F
c.	Weight Percent Sodium Pentaborate	10%
	Volume of Solution	3220 gallons
	Solution Temperature	74 deg F
d.	Weight Percent Sodium Pentaborate	14%
	Volume of Solution	1160 gallons
	Solution Temperature	87 deg F

QUESTION: 016 (1.00)

The plant is operating at 100% power. The 480V USS 1A2 bus loses power. WHICH ONE of the following Core Spray pumps is affected?

a. Core Spray Pump A

b. Core Spray Pump B

c. Core Spray Booster Pump C

d. Core Spray Booster Pump D

QUESTION: 017 (1.00)

A loss of coolant accident is in progress. WHICH ONE of the following will automatically open the Core Spray Parallel Isolation Valves?

a. Drywell pressure 2.7 psig
Reactor pressure 230 psig

b. Drywell pressure 3.3 psig
Reactor pressure 315 psig

c. Reactor water level 93 inches TAF
Reactor pressure 250 psig

d. Reactor water level 79 inches TAF
Reactor pressure 265 psig

QUESTION: 018 (1.00)

Reactor water level is 10 inches above TAF. Drywell pressure is 3.2 psig. Bus 1D voltage has been 3000 VAC for 15 seconds. WHICH ONE of the following Diesel Engine faults is still functional?

a. Engine overspeed

b. Low lube oil pressure

c. Positive crankcase pressure

d. High cooling water temperature

QUESTION: 019 (1.00)

The plant is ready for startup following a refueling outage. The first three rods have just been pulled. A report comes in that EDG #1 starting battery has an overall voltage of 106 volts. WHICH ONE of the following is the correct action?

a. Stop reactor startup until the battery can be repaired.

b. Continue reactor startup and repair the battery within 7 days.

c. Verify that all of the engineered safety features fed by EDG #1 are operational.

d. Start and load EDG #2 to greater than 20% rated power for one hour every 24 hours until the battery can be repaired.

QUESTION: 020 (1.00)

The reactor is operating at 100% power and the Feedwater Control System is in 3 element control. The control signal is lost to the A Feedwater Regulation Valve. WHICH ONE of the following describes the operation of the valve?

a. The valve fails OPEN and is operated by moving the M/A Control Transfer switch to MANUAL and turning the manual control potentiometer.

b. The valve fails OPEN and must be operated locally.

c. The valve fails AS-IS and is operated by moving the M/A Control Transfer switch to MANUAL and turning the manual control potentiometer.

d. The valve fails AS-IS and must be operated locally.

QUESTION: 021 (1.00)

The reactor is operating at 100% power. The Master feedwater controller is in MAN and each individual feedwater flow controller is in BAL. WHICH ONE of the following will change reactor water level?

a. Master manual control potentiometer.

b. Individual manual control potentiometer.

c. Master controller setpoint adjust thumbwheel.

d. Individual bias controller.

QUESTION: 022 (1.00)

During main turbine warmup the Load Limit Control is slowly moved to the prewarming mark on the Load Limit Position indicator. The Bypass Valve Opening Jack signal is then slowly increased. WHICH ONE of the following is the valve response?

a. The control valves open to the load limit position and then the bypass valves begin to open.

b. The bypass valves open to the load limit position and then the control valves begin to open.

c. The intercept valves open to the load limit position and then the bypass valves begin to open.

d. The bypass valves open to the load limit position and then the intercept valves begin to open.

QUESTION: 023 (1.00)

WHICH ONE of the following is the reason for minimizing main turbine operation between 900 and 1200 rpm?

a. To prevent damage to the low pressure turbine blades.

b. To avoid excessive turbine shaft vibration.

c. To maintain adequate turbine lube oil pressure.

d. To minimize instabilities in the turbine control system.

QUESTION: 024 (1.00)

WHICH ONE of the following could cause a rapid Reactor Water Cleanup (RWCU) filter/demineralizer breakthrough?

a. Low RWCU flow.

b. High RWCU pressure.

c. A large condenser tube leak.

d. Sodium pentaborate injection.

QUESTION: 025 (1.00)

WHICH ONE of the following isolations is bypassed when the Cleanup Filter Bypass valve (V-16-83) is open.

a. Low RWCU system flow.

b. High system pressure.

c. Standby Liquid Control system initiation.

d. High non-regenerative heat exchanger outlet temperature.

QUESTION: 026 (1.00)

WHICH ONE of the following directly causes a trip of the Shutdown Cooling pumps?

a. High temperature on "C" recirculation loop.

b. Low reactor water level.

c. Shutdown Cooling System isolation valve closure.

d. Low pump suction pressure.

QUESTION: 027 (1.00)

The control room is being evacuated and a high drywell signal is present. WHICH ONE of the following actions will bypass the isolation signals to the SDC valves V-17-19 and V-17-54?

a. Pressing the Drywell Isolation Reset pushbutton prior to leaving the control room.

b. Moving the control transfer keylock switch on local shutdown panel 1AB2 to ALTERNATE.

c. Placing the V-17-19 and -54 position switches in Pull-To-Lock prior to leaving the control room.

d. Moving the V-17-19 and -54 position switches on the local shutdown panel 1AB2 to CLOSE then to OPEN.

QUESTION: 028 (1.00)

Fifteen control rods have been withdrawn from the core during a reactor startup. The Rod Worth Minimizer has just been declared inoperable for the second time this calendar year. According to technical specifications, the startup may continue if:

a. the Manager Plant Operations grants permission.

b. a second licensed operator is present at the controls.

c. the Core Engineering Group is contacted.

d. a special report to the NRC is submitted within 30 days.

QUESTION: 029 (1.00)

The Reactor is being shutdown and power is 34%. WHICH ONE of the following will cause an Error Lockout from the Rod Worth Minimizer (RWM) system?

a. Power going below the Low Power Setpoint.

b. Three insert errors.

c. Two withdrawal errors on RWM initialization.

d. Unbypassing the RWM system.

QUESTION: 030 (1.00)

The reactor is operating at 100% power. The following diagnostic information is provided on Recirculation Pump "C":

Reduced pressure in seal cavity #2
Drywell identified leak rate has decreased.

WHICH ONE of the following is the cause of the abnormal conditions?

a. Failure of seal #1

b. Failure of seal #2

c. Plugging of orifice #1

d. Plugging of orifice #2

QUESTION: 031 (1.00)

Recirculation pump "B" is ready to be started. The pump suction valve is open. WHICH ONE of the following recirculation valve positions is required for a recirculation pump start?

a. Discharge Valve OPEN
   Bypass Valve OPEN

b. Discharge Valve CLOSED
   Bypass Valve CLOSED

c. Discharge Valve OPEN
   Bypass Valve CLOSED

d. Discharge Valve CLOSED
   Bypass Valve OPEN

QUESTION: 032 (1.00)

WHICH ONE of the following Augmented Offgas components removes iodine from the process flow?

a. HEPA filters

b. Catalytic recombiner

c. Electric preheater

d. Charcoal adsorber

QUESTION: 033 (1.00)

The Augmented Offgas (AOG) system has isolated. Given the following valve positions:

Inlet isolation valve (OG-AOV-001A)	CLOSED
Purge valve (SA-SOV-009A)	OPEN
AOG bypass valve (V-7-31)	CLOSED

WHICH ONE of the following conditions caused the AOG to isolate?

a. Hydrogen concentration upstream of the recombiner is 5.7%

b. Gas temperature out of the cooler/condenser is 243 deg F.

c. Recycle flow is 1990 scfm.

d. Main steam line radiation is 850 units.

QUESTION: 034 (1.00)

WHICH ONE of the following will allow an immediate start of Feed Pump "B" when the control switch is placed to START?

a. Bearing oil pressure 3 psig
   Pump suction pressure 66 psig

b. Bearing oil pressure 4 psig
   Pump suction pressure 52 psig

c. Bearing oil pressure 7 psig
   Pump suction pressure 42 psig

d. Bearing oil pressure 9 psig
   Pump suction pressure 32 psig

QUESTION: 035 (1.00)

The power supply to the Continuous Instrument Panel 3 (CIP-3) is lined up for normal operation. WHICH ONE of the following describes the expected system response if the 125 VDC Distribution Center 'B' is lost to the rotary inverter supplying CIP-3?

a. Power to CIP-3 continues to be supplied from VMCC-1B2 via the rotary invertor.

b. Power to CIP-3 automatically transfers to VMCC-1A2 via transformer IT-3.

c. Power to CIP-3 continues to be supplied from VMCC-1A2 via the rotary invertor.

d. Power to CIP-3 automatically transfers to VMCC-1B2 via transformer IT-3.

QUESTION: 036 (1.00)

A reactor startup is in progress and reactor power is 50% of rated. WHICH ONE of the following will result in a control rod DE-SELECT BLOCK?

a. A loss of position indication for the selected rod.

b. An APRM/recirculation flow unit trip.

c. An IRM high/inoperable trip.

d. A RWM select block.

QUESTION: 037 (1.00)

The plant is operating at 95% power. It has been determined that control rod 26-27 has experienced mechanical binding. The rod cannot be moved either by drive or scram pressure. The failure is NOT due a failed control rod drive mechanism collet housing. WHICH ONE of the following is the required action?

a. Verify that the most recent scram times of the other three rods in the array have average scram insertion time of less than 5 seconds.

b. Declare the rod inoperable and valve out the associated HCU.

c. Place the reactor in a shutdown condition within 48 hours.

d. Reduce reactor power to below the Low Power Setpoint.

QUESTION: 038 (1.00)

While testing a control rod that was drifting out of the core, the rod continues to drift out even though the Insert Line (101) and Withdraw Line (102) Isolation Valves are closed. WHICH ONE (1) of the following is the probable cause of the rod drift?

a. Stuck Collet Piston

b. Scram Outlet Valve leakage

c. Directional control valve malfunction

d. Uncoupled rod

QUESTION: 039 (1.00)

WHICH ONE of the following describes the response of the Reactor Building Heating and Ventilation (RBHV) system and the Standby Gas Treatment system (SGTS) on a loss of Vital AC Power?

a. RBHV intake and exhaust valves isolate. SGTS must be manually started to operate.

b. RBHV intake and exhaust valves remain open. SGTS automatically starts.

c. RBHV intake and exhaust valves remain open. SGTS must be manually started to operate.

d. RBHV intake and exhaust valves isolate. SGTS automatically starts.

QUESTION: 040 (1.00)

The Reactor Mode Switch is in STARTUP. The IRMs are reading on Range 10. WHICH ONE of the following conditions will cause the MSIVs to close?

a. Reactor water level falls to 92 inches above TAF.

b. Reactor pressure is 830 psig.

c. "B" steam line flow is 115% of rated.

d. "A" steam line radiation is 8 times background.

QUESTION: 041 (1.00)

The reactor has scrammed and the Isolation Condensers are in service. Reactor water level increases to 185 inches above TAF. WHICH ONE of the following valves must be closed for both isolation condensers?

a. AC steam inlet and AC condensate return valves

b. DC steam inlet and DC condensate return valves

c. DC steam inlet and AC condensate return valves

d. AC steam inlet and DC condensate return valves

QUESTION: 042 (1.00)

The following sequence of events has occurred:

- A Loss of Coolant Accident (LOCA) has occurred.
- All ECCS systems are functioning normally.
- 4160 V bus 1C has lost power.
- Emergency Diesel Generator (EDG) #1 has started, the output breaker has shut and the bus is reenergized.
- All normal loads have auto sequenced onto the bus.
- Turbine Building and Reactor Building loads have been manually restarted.
- 4160 V bus 1C steady state load is 2975 KW.

WHICH ONE of the following actions is required to be taken?

a. Manually trip the diesel generator.

b. Place the diesel in the TRANSFER mode.

c. Reduce load to less than 2850 KW within 2 hours.

d. Manually trip Turbine Building and other non-vital loads within two minutes.

QUESTION: 043 (1.00)

The reactor has scrammed but only half of the control rods have inserted into the core. When would deenergizing the scram solenoids be an effective method for alternate control rod insertion?

a. When the control rods are mechanically binding.

b. When the scram valves are open.

c. Before the scram discharge volume pressurizes.

d. Before the scram has been reset.

QUESTION: 044 (1.00)

The Level/Power Control leg of EMG-3200.01B, "RPV Control with ATWS", requires the ADS Timer Switch be placed in BYPASS. WHICH ONE of the following is the basis for this action?

a. Depressurization may cause uncontrolled injection of the Core Spray system causing large positive reactivity additions.

b. Depressurization could drive the torus temperature into the Unsafe Region of the BITT curve sooner than expected.

c. Uncontrolled depressurization would unnecessarily add heat to the torus challenging the primary containment.

d. Uncontrolled depressurization could cause a large loss of RPV inventory potentially lowering water level below the top of active fuel.

QUESTION: 045 (1.00)

The control room is being evacuated because of fire. WHICH ONE of the following actions is required to be taken prior to evacuation?

a. Trip the Main Turbine.

b. Trip all three condensate pumps.

c. Initiate the "B" Isolation Condenser.

d. Close the Main Steam Isolation Valves.

QUESTION: 046 (1.00)

Following a Control Room evacuation, the "B" Isolation Condenser has been placed in service from the Remote Shutdown Panel. An Isolation Condenser high flow isolation signal is received. WHICH ONE of the following is the effect on the Isolation Condenser System?

a. AC valves close and DC valves remain open.

b. DC valves close and AC valves remain open.

c. All AC and DC valves close.

d. All AC and DC valves remain open.

QUESTION: 047 (1.00)

The reactor is being refueled. All control rods are inserted. Shutdown cooling is in operation. As a fuel assembly is being inserted into the core the control room reports that the SRM count rate has increased by a factor of eight over the initial count rate recorded at the beginning of the shift. WHICH ONE of the following is the immediate operator action?

a. Trip all operating recirculation pumps.

b. Remove the fuel assembly being inserted.

c. Initiate the Standby Liquid Control System.

d. Stop all fuel movement and notify the Core Engineer.

QUESTION: 048 (1.00)

While moving an irradiated fuel assembly in the Fuel Pool, the Fuel Pool level is found to be decreasing at nearly one foot per minute. Refuel Floor radiation levels remain normal. WHICH ONE of the following is the required operator action?

a. Stop all fuel movement and evacuate the refuel floor.

b. Monitor for airborne activity and complete the move as planned.

c. Return the assembly to its original location.

d. Place the assembly in the nearest vacant storage rack.

QUESTION: 049 (1.00)

WHICH ONE of the following sets of conditions describes a situation in which adequate core cooling is assured?

a. Core Spray System is in operation and maintaining reactor water level at -20 inches while implementing Containment Flooding.

b. Feedwater System is in operation and maintaining reactor water level at -5 inches while implementing Level Restoration.

c. Level being maintained at -25 inches while implementing Level/Power control.

d. Level is being maintained at -45 inches with one EMRV open while implementing Steam Cooling.

QUESTION: 050 (1.00)

During a severe accident it is determined that primary containment must be vented due to high hydrogen concentration. Primary containment radiation levels exceed 20000 R/hr. The reason that harden pipe vent path may NOT be used for venting the primary containment under these conditions is:

a. vent path high radiation levels precludes local valve manipulation.

b. vent path does NOT take advantage of torus "scrubbing" of radioactive particulate.

c. vent path permits unmonitored venting to the environment.

d. vent path will cause radiation levels in the Reactor Building to exceed Max Safe levels.

QUESTION: 051 (1.00)

The reactor is operating at 4% power. Reactor pressure is 1000 psig. Electromatic Relief Valve Operability Test has just been concluded. Suppression Pool temperature is 105 deg F. WHICH ONE of the following is the required action?

a. Reduce the pool temperature to less than the normal power operation limit within 24 hours.

b. Reduce the pool temperature to less than the normal Power operation limit or be in COLD SHUTDOWN within 30 hours.

c. Manually drive control rods and reduce reactor pressure to less than 180 psig.

d. Scram the reactor.

QUESTION: 052 (1.00)

WHICH ONE (1) of the following describes the results of a loss of three feedwater heaters?

a. Core inlet subcooling decreases
Critical power ratio decreases

b. Core inlet subcooling increases
Critical power ratio decreases

c. Core inlet subcooling decreases
Critical power ratio increases

d. Core inlet subcooling increases
Critical power ratio increases

QUESTION: 053 (1.00)

The offsite radiation release rate is above the Alert limit and the Turbine Building Ventilation has shutdown. WHICH ONE (1) of the following is the reason EMG 3200.12, Radioactivity Release Control, directs that the Turbine Building Ventilation be restarted?

a. To maintain positive pressure in the Turbine Building.

b. To prevent unmonitored radiation release to the environment.

c. To provide a filtered release path to the environment.

d. To assure max safe temperature limits are not reached.

QUESTION: 054 (1.00)

WHICH ONE of the following sets of parameters must be monitored to determine if the reactor must be emergency depressurized because the Heat Capacity Temperature Limit has been exceeded?

a. RPV pressure, drywell temperature, drywell pressure

b. Torus temperature, torus water level, torus pressure

c. Drywell temperature, torus temperature, drywell pressure

d. Torus water level, RPV pressure, torus temperature

QUESTION: 055 (1.00)

During a Loss of Coolant Accident the Core Spray pumps A and C are required to maintain reactor water level. The following conditions exist:

Torus water level	113 inches
Torus pressure	4.1 psig
Torus temperature	198 deg F

WHICH ONE of the following is the MAXIMUM allowable Core Spray pump flow?

a. 3000 gpm

b. 3500 gpm

c. 4000 gpm

d. 4500 gpm

QUESTION: 056 (1.00)

WHICH ONE of the following is the basis for maintaining the volume of water in the suppression pool greater than 82,000 cu. ft.?

a. To prevent reaching ECCS Net Positive Suction Head (NPSH) limits at maximum design operating temperature.

b. To prevent Core Spray and Containment Spray corner room ambient temperatures from affecting motor temperature limits.

c. To prevent exceeding Drywell or Torus design pressures during a reactor blowdown from 1060 psig.

d. To prevent reaching Core Spray Vortex limits during ECCS maximum design flow rates.

QUESTION: 057 (1.00)

WHICH ONE of the following is the reason for the Minimum Core Flooding Interval during RPV Flooding?

a. To ensure that there is enough level in the RPV to support steam cooling.

b. To ensure that the RPV has been filled to the top of active fuel.

c. To allow enough time for all the ADS valves to be opened.

d. To allow enough time to reduce the RPV to Torus differential pressure to 60 psid.

QUESTION: 058 (1.00)

An ATWS is in progress and reactor power is 35%. WHICH ONE of the following is the reason that recirculation flow is runback to minimum prior to tripping the recirculation pumps?

a. To minimize the reactor water swell from the recirculation pump trip.

b. To determine if reactor power can be maintain below 2% without tripping the recirculation pumps.

c. To minimize the power transient below the EMRV setpoints.

d. To avoid damaging the recirculation MG set drive motor breakers by opening them at full load.

QUESTION: 059 (1.00)

Fuel damage has occurred during a loss of coolant accident. WHICH ONE of the following conditions would require entry into EMG-3200.12, "Radioactivity Release Control"?

a. Offgas radiation monitors reading 11 R/hr

b. Iodine release rate of 32 uCi/sec.

c. Dose projection at the Site Boundary of 7 mrem whole body.

d. Field team reading of 56 mrem thyroid dose at the Site Boundary.

QUESTION: 060 (1.00)

WHICH ONE of the following describes the status of the given components following a ground fault on Bus 1B?

a. Breaker S1B Closed
   Breaker 1D Closed
   EDG #2 Off

b. Breaker S1B Open
   Breaker 1D Closed
   EDG #2 Running

c. Breaker S1B Closed
   Breaker 1D Open
   EDG #2 Off

d. Breaker S1B Open
   Breaker 1D Open
   EDG #2 Running

QUESTION: 061 (1.00)

The reactor has scrammed on low condenser vacuum. All control rods have inserted, the mode switch is in SHUTDOWN, and the main turbine has tripped. WHICH ONE of the following actions is required to reset the scram?

a. Wait until the condenser vacuum is above the scram setpoint.

b. Place the SDV HI-HI LVL SCRAM switch in BYPASS.

c. Isolate the CRD Charging Water header.

d. Place the mode switch in REFUEL.

QUESTION: 062 (1.00)

WHICH ONE of the following is the reason for stabilizing reactor pressure below 1045 psig following a reactor scram and subsequent entry into RPV Control EOP?

a. To allow the scram logic to be reset.

b. To ensure proper operation of the Yarway level instruments

c. To permit RPV pressure control with the bypass valves.

d. To maintain cooldown rate below 100 deg F per hour.

QUESTION: 063 (1.00)

Given the following conditions:

Reactor power is 100% and steady state
Reactor level is 144 inches and decreasing
Steam flow is steady at 7.2x10 E6 lbm/hr
Individual feed flows are 2x10 E6 lbm/hr each
EPR is controlling normally

WHICH ONE of the following is the required action?

a. Place individual feed controllers in MANUAL.

b. Place FWC in single element mode.

c. Place the level controller in MANUAL.

d. Take local control of the Feedwater Regulating Valves.

QUESTION: 064 (1.00)

During a loss of coolant accident the following conditions exist:

Drywell pressure is 3.3 psig
Drywell temperature is 157 deg F
Reactor building ventilation radiation levels are 11 mr/hr.

WHICH of the following Emergency Operating Procedures must be entered?

a. Primary Containment Control only.

b. Secondary Containment Control only.

c. RPV Control and Primary Containment Control.

d. RPV Control and Secondary Containment Control.

QUESTION: 065 (1.00)

The plant is in the Startup Mode and rods are being pulled to achieve criticality. The reactor goes critical prior to the 1% ECP margin. WHICH ONE of the following actions must be performed?

a. Stop pulling control rods and maintain reactor power constant.

b. Insert control rods in reverse order and bring the reactor subcritical.

c. Scram the reactor and commence a cooldown.

d. Continue with the reactor startup with a second CRO at the controls to verify rod pattern.

QUESTION: 066 (1.00)

The reactor is operating at 100% power when the OFF GAS HI alarm annunciates. WHICH ONE of the following actions should be taken?

a. Scram the reactor and close the MSIVs.

b. Reduce reactor power until the alarm clears.

c. Close the Steam Jet Air Ejector air extraction valves.

d. Obtain an Offgas grab sample and continue normal operation.

QUESTION: 067 (1.00)

During a loss of coolant accident reactor pressure is 200 psig, and the following information is provided:

INSTRUMENT	REFERENCE LEG TEMPERATURE	INDICATED LEVEL
NR GEMAC A	420 deg F	110 inches
NR GEMAC B	430 deg F	100 inches
WR GEMAC	360 deg F	140 inches
Yarway A	370 deg F	90 inches
Yarway B	390 deg F	105 inches

WHICH ONE of the following instruments may be used to determine reactor water level?

a. NR GEMAC A

b. WR GEMAC

c. Yarway B

d. Yarway A

QUESTION: 068 (1.00)

The plant is operating with five recirculation pumps. A single recirculation pump trips and the following conditions exist:

Reactor Power 51%
Recirculation Flow 62,000 gpm
APRM 1 is oscillating from 46% to 55%
APRM 2 is bypassed
APRM 3 is oscillating from 49% to 56%
APRM 4 is oscillating from 47% to 53%
APRM 5 is oscillating from 45% to 53%
APRM 6 is oscillating from 49% to 52%
APRM 7 is oscillating from 50% to 55%
APRM 8 is bypassed

WHICH ONE of the following is the required operator action?

a. Restart the tripped recirculation pump.

b. Insert CRAM array to lower power below 80% rod line.

c. Increase recirculation flow to 72,000 gpm.

d. Scram the reactor.

QUESTION: 069 (1.00)

The plant has experienced a trip of all recirculation pumps. WHICH ONE of the following is the reason that the reactor is depressurized to atmospheric pressure prior to restarting any recirculation pumps?

a. To maintain balanced natural circulation.

b. To prevent excessive starting loads on recirculation pump MG sets.

c. To limit thermal stresses on stub tube area of the RPV.

d. To prevent inadvertent reactivity additions.

QUESTION: 070 (1.00)

The following alarms have come in during 100% power operation:

TRIP SOL 1 DC LOST
TRIP SOL 2 DC LOST

WHICH ONE of the following will trip the main turbine?

a. High reactor water level.

b. Worn thrust bearing.

c. Manual trip on Panel 7F in the control room.

d. Manual trip on the turbine front standard.

QUESTION: 071 (1.00)

The plant is operating at 100% power when a total loss 125 VDC occurs. WHICH ONE of the following will isolate AND lose the capability for the isolation to be reset?

a. Main Steam Isolation Valves

b. Isolation Condensers

c. Shutdown Cooling system

d. Scram Discharge Volume

QUESTION: 072 (1.00)

The main turbine generator has tripped. The following conditions exist:

Reactor power is 37 %
Recirculation flow is 76,000 gpm

WHICH ONE of the following is the required operator action?

a. Scram the reactor.

b. Enter EMG-3200.1B, "RPV Control - With ATWS".

c. Insert CRAM array.

d. Maintain reactor power below 40%.

QUESTION: 073 (1.00)

Following a reactor scram, reactor pressure is 980 psig and reactor water level is 172 inches and increasing. WHICH ONE of the following is the required operator action?

a. Trip all but one feedwater pump.

b. Trip all feedwater pumps.

c. Isolate the isolation condensers.

d. Trip all condensate pumps.

QUESTION: 074 (1.00)

The plant is operating at 100% power when the following conditions are observed:

TIME=0 Alarm CCW FLOW LO A for 'A' Recirculation Pump annunciates

TIME=30 sec Alarm CCW FLOW LO E for 'E' Recirculation Pump annunciates

After 2 minutes the above conditions still exist. WHICH ONE of the following is the required action?

a. Increase RBCCW flow to the affected Recirculation Pumps

b. Reduce Recirculation flow to minimum and secure the affected Recirculation Pumps.

c. Scram the reactor and trip all Recirculation Pumps.

d. Secure the Reactor Water Cleanup System.

QUESTION: 075 (1.00)

Given the following conditions:

RBCCW pumps have tripped due to a loss of offsite power
Reactor water level is +80 inches
Drywell pressure is 3.5 psig

WHICH ONE of the following describes restarting the RBCCW pumps when the diesel generators reenergize the emergency busses?

a. The pumps that tripped will automatically restart when power is restored to the respective pump busses.

b. The pumps must be manually started upon clearing the trip signal by placing the control room control switch to OFF and then to START.

c. The trip logic must be bypassed locally on the respective pump switchgear prior to manual pump start.

d. The low-low level signal must be jumpered out prior to automatic pump start.

QUESTION: 076 (1.00)

The plant is experiencing a loss of instrument air pressure while operating at 85 % power. WHICH ONE of the following requires a manual reactor scram?

a. One control rod begins to drift in.

b. Reactor water level cannot be controlled with Feedwater Control valves.

c. Instrument air pressure has decreased to 52 psig.

d. Main Condenser vacuum has decreased to 24 inches Hg.

QUESTION: 077 (1.00)

WHICH ONE of the following will fail OPEN on a complete loss of Service and Instrument Air?

a. Scram discharge volume vent valves

b. Isolation condenser makeup water valves

c. CRD flow control valves

d. Reactor Building to Torus vacuum breakers

QUESTION: 078 (1.00)

While at power an inadvertent primary containment isolation occurs because of a false high drywell pressure signal. WHICH ONE of the following systems will still be in operation following the isolation?

a. Reactor Building Heating and Ventilation System

b. Reactor Water Cleanup System

c. Reactor Building Closed Cooling Water System

d. Containment Atmosphere Control System.

QUESTION: 079 (1.00)

The plant is in an ATWS condition and the following conditions exist:

Boron is being injected
RPV water level is 82 inches
Drywell pressure is 3.4 psig
Steam line radiation is 6 times greater than background
Reactor pressure is 800 psig
Mode switch is in SHUTDOWN
MSIVs are closed
Main Condenser is available

WHICH ONE of the following isolation signals must be bypassed to open the MSIVs?

a. Lo-lo reactor water level

b. Steam line high radiation.

c. Low reactor pressure

d. Steam line high flow

QUESTION: 080 (1.00)

The plant is preparing for shutdown cooling. Reactor pressure is 175 psig and RPV temperature is 360 deg F. Shutdown cooling containment isolation valve V-17-19 has been determined to be inoperable. In accordance with Tech Specs, it has been deactivated in the shut position. WHICH ONE of the following is the means for cooldown with this valve inoperable?

a. The cooldown will continue with turbine bypass valves until the reactor is in a COLD SHUTDOWN condition.

b. The SDC inlet valve may be opened for 4 hours allowing SDC to do the cooldown to the point where Primary Containment Integrity is no longer required.

c. The inlet valve may be opened once temperature is less than 350 degrees F to allow a normal plant cooldown.

d. Cool down the plant using alternate decay heat removal methods via Reactor Water Cleanup.

QUESTION: 081 (1.00)

The reactor is shutdown, reactor coolant temperature is 180 degrees F, reactor water level is 160 inches, and all recirculation pumps are tripped. Shutdown cooling is lost and cannot be reestablished. WHICH ONE of the following actions should the operator take to establish or enhance natural circulation?

a. Maintain reactor water level at its normal level and open at least three (3) EMRVs.

b. Initiate one Isolation Condenser in alternate decay heat removal.

c. Start one Core Spray pump taking suction from the torus and then open one EMRV.

d. Raise reactor water level to at least 185" above TAF and ensure at least one recirculation loop is unisolated.

QUESTION: 082 (1.00)

Reactor Power is 95%. The standby CRD pump is not available for operation. The operating CRD pump trips and cannot be restarted. The CHARG WTR PRESS LO alarm is received. WHICH ONE of the following conditions will require a reactor scram?

a. CRD high temperature

b. One control rod drifting in

c. SDV NOT DRAINED alarm annunciates

d. Accumulator Rod Block light illuminates

QUESTION: 083 (1.00)

Containment flooding is in progress. WHICH ONE of the following corresponds to a primary containment water level of 1040 inches?

a. Top of the torus

b. Lowest point of the primary system

c. Top of active fuel

d. Highest vent point

QUESTION: 084 (1.00)

The reactor is shutdown and the reactor coolant temperature is 200 deg F. A report is received that the intake canal water level is -3.2 feet. WHICH ONE of the following is the required action/ classification?

a. Notification of the NRC is required, no classification is necessary.

b. Unusual Event

c. Alert

d. Site Area Emergency

QUESTION: 085 (1.00)

A person requests to perform work under an existing system outage. Upon review by the System Outage Coordinator and the person responsible for the work, it is determined that the isolation boundaries need to be expanded. WHICH ONE of the following would be an acceptable option for performing the work?

a. Initiate a Specific outage request.

b. Initiate a Personal outage request.

c. Initiate a Supplemental outage request.

d. Perform the work under the existing System outage.

QUESTION: 086 (1.00)

WHICH ONE of the following is the MAXIMUM time that a Personal Outage can be used?

a. One shift

b. 24 hours

c. 72 hours

d. 7 days

QUESTION: 087 (1.00)

WHICH ONE of the following is MINIMUM time period necessary for an operator to abstain from alcoholic beverages prior to reporting for a scheduled shift?

a. 3 hours

b. 5 hours

c. 8 hours

d. 12 hours

QUESTION: 088 (1.00)

WHICH ONE of the following is NOT authorized to grant unescorted control room access?

a. Vice President and Director

b. Plant Operations Director

c. Security Manager

d. Group Shift Supervisor (on shift only)

QUESTION: 089 (1.00)

WHICH ONE of the following is the MINIMUM staffing required by Technical Specifications, when the reactor is at 35% power and ascending to 100% during startup?

a. One (1) licensed senior reactor operator in the control room
Two (2) licensed reactor operators, one (1) in the control room, one (1) on site
Two (2) equipment operators on site

b. Two (2) licensed senior reactor operators, one (1) in the control room, one (1) on site
Two (2) licensed reactor operators in the control room
Three (3) equipment operators on site

c. Two (2) licensed senior reactor operators in the control room
Two (2) licensed reactor operators in the control room
Three (3) equipment operators on site

d. One (1) licensed senior reactor operator on site
Two (2) licensed reactor operators, one (1) in the control room, one (1) on site
Two (2) equipment operators on site

QUESTION: 090 (1.00)

The mode switch is required to be locked in the SHUTDOWN position. WHICH ONE of the following is the location of the key?

a. Panel 4F

b. GOS Desk

c. General Key Locker

d. Controlled Key Locker

QUESTION: 091 (1.00)

WHICH ONE of the following signature responsibilities can NOT be delegated by the Group Shift Supervisor to the Group Operation Supervisor?

a. Shift Turnover Checklist

b. Temporary Change Requests

c. Maintenance Job Orders

d. Switch and Tagging Outage Requests

QUESTION: 092 (1.00)

An UNUSUAL EVENT has been declared. WHICH ONE of the following must be notified within 15 minutes?

a. Nuclear Regulatory Commission

b. New Jersey State Police

c. Ocean County

d. American Nuclear Insurers

QUESTION: 093 (1.00)

During a plant tour a radioactive system is found to be leaking. WHICH ONE of the following is the operator immediate action to this spill?

a. Erect barriers and post the area as potentially contaminated.

b. Perform a radiological survey of the area.

c. Obtain a spill kit to control the spill.

d. Warn others in the vicinity of the spill.

QUESTION: 094 (1.00)

An area of the plant has been surveyed and the general area reads 130 mrem/hour with 800 dpm/100 sq cm surface contamination. WHICH ONE of the following is the required posting?

a. Contaminated Area

b. Exclusion Area

c. High Radiation Area

d. Hot Spot

QUESTION: 095 (1.00)

WHICH ONE of the following is the required number of licensed personnel on the Shutdown Crew?

a. 1

b. 2

c. 3

d. 4

QUESTION: 096 (1.00)

WHICH ONE of the following is replaced by the Shutdown Log during a refueling outage?

a. Equipment Status Log

b. Reactor Auxiliary Log

c. Environmental Data Log

d. Area and Effluent Radiation Monitor Log

QUESTION: 097 (1.00)

During the performance of a special test procedure on the Containment Spray System, the operator decides that he can NOT perform the next step in the procedure as it is written due to the present plant conditions.

WHICH ONE of the following describes the required operator action?

a. The operator should stop all system manipulations and notify the Manager Plant Operations.

b. The operator should NA this step, notify the GSS or GOS, and continue the procedure.

c. The operator should stop and back out of the procedure.

d. The operator should stop and place the system in a stable and safe condition and notify the GSS or GOS.

QUESTION: 098 (1.00)

During an emergency the Emergency Control Center has been activated. WHICH ONE of the following approves and directs information releases to the media?

a. Emergency Director

b. Emergency Support Director

c. Communications Coordinator

d. Operations Coordinator

QUESTION: 099 (1.00)

A Licensed Reactor Operator has worked the following schedule during a refueling outage:

- Tuesday -- 7 am to 7 pm
- Wednesday -- 7 am to 7 pm
- Thursday -- 7 am to 3 pm
- Friday -- 7 am to 3 pm
- Saturday -- 7 am to 3 pm
- Sunday -- 7 am to 9 pm

WHICH ONE of the following work schedules for the following Monday is the MAXIMUM allowed without exceeding the guidelines in the Technical Specifications.

a. 7 am to 11 am

b. 7 am to 3 pm

c. 7 am to 5 pm

d. 7 am to 7 pm

QUESTION: 100 (1.00)

The plant has been shutdown for an outage and preparations have been made for the initial Drywell entry. WHICH ONE of the following is the point where unlimited access to the Drywell is allowed?

a. When general Drywell area radiation levels fall below 10 mr/hr.

b. After the reactor has been shutdown for greater than 24 hours.

c. After the Drywell airlock interlock has been defeated.

d. When Drywell oxygen levels are verified at 18%.

(********** END OF EXAMINATION **********)

ANSWER: 001 (1.00)

a.

REFERENCE:

1. OCNGS: Procedure 307, 2.2.10, p. 7
2. OCNGS: TCR 828.23, LO O.

KA: 207000A202 [4.3/4.7]

207000A202 ..(KA's)

ANSWER: 002 (1.00)

c.

REFERENCE:

1. OCNGS: Procedure 307, step 2.2.5, p. 6.
2. OCNGS: TCR 828.23, LO Y.

KA: 207000A101 [3.7/3.8]

207000A101 ..(KA's)

ANSWER: 003 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 5, p. Figure 5-14
2. OCNGS: TCR 828.05, LO I.

KA: 218000A401 [4.4/4.4]

218000A401 ..(KA's)

ANSWER: 004 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 5, p. 24.
2. OCNGS: TCR 828.05, LO J.

KA: 218000K606 [3.4/3.6]

218000K606 ..(KA's)

ANSWER: 005 (1.00)

d.

REFERENCE:

1. OCNGS: TCR 828.42, p. 28, LO L.

KA: 261000K401 [3.7/3.8]

261000K401 ..(KA's)

ANSWER: 006 (1.00)

c.

REFERENCE:

1. OCNGS: Technical Specifications 4.5.L.1.a
2. OCNGS: TCR 828.42, LO K.

KA: 261000G010 [3.1/3.3]

261000G010 ..(KA's)

ANSWER: 007 (1.00)

c.

REFERENCE:

1. OCNGS: Technical Specification 3.3.H
2. OCNGS: TCR 828.38, LO K.

KA: 202002G005 [3.3/4.0]

202002G005 ..(KA's)

ANSWER: 008 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 38B, p. 48.
2. OCNGS: TCR 828.40, LO M.

KA: 202002G010 [3.3/3.3]

202002G010 ..(KA's)

ANSWER: 009 (1.00)

b.

REFERENCE:

1. OCNGS: OPM 29E, p. 6.
2. OCNGS: TCR 828.29D, LO R.2.c

KA: 215005K501 [2.8/2.9]

215005K501 ..(KA's)

ANSWER: 010 (1.00)

d.

REFERENCE:

1. OCNGS: Technical Specification Table 3.1.1, note c.
2. OCNGS: Procedure 403.3, step 4.4, p. 3.
3. OCNGS: TCR 828.29D, LO I.

KA: 215005K504 [2.9/3.2]

215005K504 ..(KA's)

ANSWER: 011 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 29B, p. 24.
2. OCNGS: TCR 828.29B, LO E.

KA: 215004A106 [3.1/3.1]

215004A106 ..(KA's)

ANSWER: 012 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 29B, p. 30.
2. OCNGS: TCR 828.29B, LO C.

KA: 215004K602 [3.1/3.3]

215004K602 ..(KA's)

ANSWER: 013 (1.00)

b.

REFERENCE:

1. OCNGS: OPM 37, Table 37-2
2. OCNGS: TCR 828.37, LO I

KA: 212000K412 [3.9/4.1]

212000K412 ..(KA's)

ANSWER: 014 (1.00)

b.

REFERENCE:

1. OCNGS: OPM 37, p. 5 and 37
2. OCNGS: TCR 828.37, LO Q

KA: 212000A108 [3.4/3.4]

212000A108 ..(KA's)

ANSWER: 015 (1.00)

d.

REFERENCE:

1. OCNGS: Technical Specification 3.2, Figures 3.2.1 and 3.2.2
2. OCNGS: TCR 828.46, LO R, EE.

KA: 211000G005 [3.6/4.4]

211000G005 ..(KA's)

ANSWER: 016 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 10, p. 14.
2. OCNGS: TCR 828.10, LO F.

KA: 209001K201 [3.0/3.1]

209001K201 ..(KA's)

ANSWER: 017 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 10, p. 16
2. OCNGS: TCR 828.10, p. 15 and 16, LO C and P.

KA: 209001A301 [3.6/3.6]

209001A301 ..(KA's)

ANSWER: 018 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 13, p. 57, 58 & 60.
2. OCNGS: TCR 828.13, LO I-1 J.

KA: 264000K402 [4.0/4.2]

264000K402 ..(KA's)

ANSWER: 019 (1.00)

a.

REFERENCE:

1. OCNGS: Technical Specification 3.7.C.1 and 4.7.B.1.c
2. OCNGS: TCR 828.13, LO I-2 G.

KA: 264000G005 [3.4/4.1]

264000G005 ..(KA's)

ANSWER: 020 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 18, p. 31-32
2. OCNGS: TCR 828.18, LO B.

KA: 259002K302 [3.7/3.7]

259002K302 ..(KA's)

ANSWER: 021 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 18, p. 15, 17
2. OCNGS: Facility Question 328
3. OCNGS: TCR 828.18, LO J.

KA: 259002A101 [3.8/3.8]

259002A101 ..(KA's)

ANSWER: 022 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 51, p. 31.
2. OCNGS: 828.51, p. 18-19, LO L.

KA: 241000A107 [3.8/3.7]

241000A107 ..(KA's)

ANSWER: 023 (1.00)

b.

REFERENCE:

1. OCNGS: 315.1, step 2.2.8, p. 5.
2. OCNGS: 828.51, LO K.

KA: 241000A113 [2.7/2.7]

241000A113 ..(KA's)

ANSWER: 024 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 39, p. 62
2. OCNGS: TCR 828.39, LO S.4.

KA: 204000K106 [2.8/2.8]

204000K106 ..(KA's)

ANSWER: 025 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 39, p. 44.
2. OCNGS: TCR 828.39, LO F.

KA: 204000A205 [2.7/2.8]

204000A205 ..(KA's)

ANSWER: 026 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 45, p. 11.
2. OCNGS: Facility question 205-45
3. OCNGS: TCR 828.45, p. LO C.

KA: 205000A303 [3.5/3.3]

205000A303 ..(KA's)

ANSWER: 027 (1.00)

b.

REFERENCE:

1. OCNGS: OPM 45, p. 10
2. OCNGS: TCR 828.45, LO E.

KA: 205000G007 [3.7/3.7]

205000G007 ..(KA's)

ANSWER: 028 (1.00)

b.

REFERENCE:

1. OCNGS: Technical Specifications 3.2.B.2.a
2. OCNGS: TCR 828.41, LO D.

KA: 201006G005 [3.2/4.0]

201006G005 ..(KA's)

ANSWER: 029 (1.00)

d.

REFERENCE:

1. OCNGS: TCR 828.41, p. 19-20, LO I

KA: 201006K403 [3.3/3.4]

201006K403 ..(KA's)

ANSWER: 030 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 38A, p. 9
2. OCNGS: TCR 828.38, LO F.

KA: 202001A109 [3.3/3.3]

202001A109 ..(KA's)

ANSWER: 031 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 38A, p. 14.
2. OCNGS: TCR 828.38, p. 11, LO F.

KA: 202001K410 [3.3/3.4]

202001K410 ..(KA's)

ANSWER: 032 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 4, 23
2. OCNGS: TCR 828.02, LO C.8

KA: 271000K508 [2.5/2.6]

271000K508 ..(KA's)

ANSWER: 033 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 4, p. 28
2. OCNGS: TCR 828.02, LO J.

KA: 271000A112 [3.1/3.5]

271000A112 ..(KA's)

ANSWER: 034 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 17, p. 15.
2. OCNGS: TCR 828.17, LO F.

KA: 259001A402 [3.9/3.7]

259001A402 ..(KA's)

ANSWER: 035 (1.00)

b.

REFERENCE:

1. OCNGS: OPM Module 56 page 56-19
2. OCNGS: TCR 828.56, LO D.
3. EQB: HNUM 15430 (1990 exam)

KA: 262002K401 [3.1/3.4]

262002K401 ..(KA's)

ANSWER: 036 (1.00)

b.

REFERENCE:

1 OCNGS: TCR 828.36, p. 16, LO H.
2. EQB: HNUM 18170 (1991 Exam)

KA: 201002K402 [3.5/3.5]

201002K402 ..(KA's)

ANSWER: 037 (1.00)

b.

REFERENCE:

1. OCNGS: Technical Specification 3.2.B.4.
2. OCNGS: TCR 828.11, LO I.

KA: 201003G005 [3.3/3.9]

201003G005 ..(KA's)

ANSWER: 038 (1.00)

a.

REFERENCE:

1. OCNGS: 2000-ABN-3200.06, p. 5, step 3.2.4.
2. OCNGS: TCR 828.11, LO M.4

KA: 201003A203 [3.4/3.7]

201003A203 ..(KA's)

ANSWER: 039 (1.00)

a.

REFERENCE:

1. OCNGS: OPM 42 p 29
2. OCNGS: TCR 828.42, LO M
3. EQB: QNUM 15333 (1990 Exam)

KA: 288000K401 [2.7/2.7]

288000K601 ..(KA's)

ANSWER: 040 (1.00)

b.

REFERENCE:

1. OCNGS: OPM 26, p. 27.
2. OCNGS: TCR 828.26, LO O.

KA: 239001K401 [3.8/3.8]

239001K401 ..(KA's)

ANSWER: 041 (1.00)

b.

REFERENCE:

1. OCNGS: 2000-ABN-3200.01, rev. 22, step 3.7, p. 4
2. OCNGS: TCR 801, LO A.

KA: 295006G010 [4.1/4.2]

295006G010 ..(KA's)

ANSWER: 042 (1.00)

d.

REFERENCE:

1. OCNGS: Procedure 341, Pages 4.0, 5.0 & 15.0
2. OCNGS: TCR 828.13, LO J.
3. EQB: 15347 (1990 Exam)

KA: 295003A102 [4.2/4.3]

295003A102 ..(KA's)

ANSWER: 043 (1.00)

c.

REFERENCE:

1. OCNGS: EOP Users Guide, p. 1B-94.
2. OCNGS: TCR 845.05, LO A.

KA: 295037K307 [4.2/4.3]

295037K307 ..(KA's)

ANSWER: 044 (1.00)

a.

REFERENCE:

1. OCNGS: EOP User Guide, p. 1B-14.
2. OCNGS: TCR 845.19, p. 5, LO B.

KA: 295037K106 [4.0/4.2]

295037K106 ..(KA's)

ANSWER: 045 (1.00)

d.

REFERENCE:

1. OCNGS: 2000-ABN-3200.30, p. 3.
2. OCNGS: TCR 801.01, LO A.9.

KA: 295016G010 [3.8/3.6]

295016G010 ..(KA's)

ANSWER: 046 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 23, p. 10.
2. OCNGS: TCR 828.23, p. 16, LO R.
3. OCNGS: Procedure 346, p. 11

KA: 295016A109 [4.0/4.0]

295016A109 ..(KA's)

ANSWER: 047 (1.00)

b.

REFERENCE:

1. OCNGS: 2000-ABN-3200.07, p. 4.
2. OCNGS: TCR 801.01, LO A.

KA: 295023K103 [3.7/4.0]

295023K103 ..(KA's)

ANSWER: 048 (1.00)

d.

REFERENCE:

1. OCNGS: Procedure 205, rev. 41, p. 56
2. EQB: 18026 (1991 exam)

KA: 295023A202 [3.4/3.7]

295023A202 ..(KA's)

ANSWER: 049 (1.00)

c.

REFERENCE:

1. OCNGS: TCR 845.15, p. 3.
2. EQB: 15356 (1990 exam)

KA: 295031A204 [4.6/4.8]

295031A204 ..(KA's)

ANSWER: 050 (1.00)

a.

REFERENCE:

1. OCNGS: EOP Users Guide, p. 2-30.
2. OCNGS: TCR 828.08, LO D.

KA: 295024K307 [3.5/4.0]

295024K307 ..(KA's)

ANSWER: 051 (1.00)

a.

REFERENCE:

1. OCNGS: Technical Specification 3.5.A and 3.0.A.
2. OCNGS: TCR 828.32, LO N.

KA: 295013G003 [3.3/4.2]

295013G003 ..(KA's)

ANSWER: 052 (1.00)

b.

REFERENCE:

1. OCNGS: TCR 852.09, p. 12, LO Q

KA: 295014K202 [3.7/4.2]

295014K202 ..(KA's)

ANSWER: 053 (1.00)

b.

REFERENCE:

1. OCNGS: EOP User's Guide, p. 12-3
1. OCNGS: TCR 845.12, LO D.

KA: 295017K302 [3.3/3.5]

295017K302 ..(KA's)

ANSWER: 054 (1.00)

d.

REFERENCE:

1. OCNGS: EMG-3200.02, Figure F.

KA: 295026K301 [3.8/4.1]

295026K301 ..(KA's)

ANSWER: 055 (1.00)

b.

REFERENCE:

1. OCNGS: EMG-3200.01A, Attachment E, figures B and C.
2. OCNGS: TCR 845.03, LO F.

KA: 295030A101 [3.6/3.8]

295030A101 ..(KA's)

ANSWER: 056 (1.00)

b.

REFERENCE:

1. OCNGS: Technical Specification 3.5 Bases, Page 3.5-8
2. OCNGS: TCR 828.32, LO N.
3. EQB: 15355 (1990 exam)

KA: 295030G004 [2.7/4.2]

295030G004 ..(KA's)

ANSWER: 057 (1.00)

b.

REFERENCE:

1. OCNGS: EOP User's Guide, p. 8A-20 and 13-6.
2. OCNGS: TCR 845.18, LO B.

KA: 295031K302 [4.4/4.7]

295031K302 ..(KA's)

ANSWER: 058 (1.00)

a.

REFERENCE:

1. OCNGS: EOP User's Guide, p. 1B-78.
2. OCNGS: TCR 845.05, LO A.

KA: 295037K301 [4.1/4.2]

295037K301 ..(KA's)

ANSWER: 059 (1.00)

d.

REFERENCE:

1. OCNGS: EPIP-OC-.01, Appendix 1, p. 5 and 6.
2. OCNGS: EMG-3200.12

KA: 295038G011 [4.2/4.5]

295038G011 ..(KA's)

ANSWER: 060 (1.00)

d.

REFERENCE:

1. OCNGS: TCR 828.16, p. 9 and 10, LO C.

KA: 295003K203 [3.7/3.9]

295003K203 ..(KA's)

ANSWER: 061 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.01, p. 7.
2. OCNGS: TCR 828.37, LO P.

KA: 295006A101 [4.2/4.2]

295006A101 ..(KA's)

ANSWER: 062 (1.00)

a.

REFERENCE:

1. OCNGS: EOP User's Guide, p. 1A-57.
2. OCNGS: TCR 845.03, LO E.

KA: 295007G007 [3.3/3.5]

295007G007 ..(KA's)

ANSWER: 063 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.17, p. 4.
2. OCNGS: Facility question 0307 (Duty Area 259)

KA: 295009A102 [4.0/4.0]

295009A102 ..(KA's)

ANSWER: 064 (1.00)

c.

REFERENCE:

1. OCNGS: EOP User's Guide, p. 1B-2
2. OCNGS: TCR 845.03, LO B.

KA: 295010G011 [4.2/4.5]

295010G011 ..(KA's)

ANSWER: 065 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.07, p. 5.
2. OCNGS: TCR 801.01, LO A.5
3. EQB: 15359 (1990 exam)

KA: 295014A104 [3.2/3.3]

295014A104 ..(KA's)

ANSWER: 066 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.26, p. 4.
2. OCNGS: TCR 828.02, LO H.

KA: 295017A102 [3.5/3.7]

295017A102 ..(KA's)

ANSWER: 067 (1.00)

d.

REFERENCE:

1. OCNGS: EMG-3200.02, Attachment E (attached)
2. OCNGS: TCR 828.55, LO C.4

KA: 295028A203 [3.7/3.9]

295028A203 ..(KA's)

ANSWER: 068 (1.00)

d.

REFERENCE:

1. OCNGS: ABN-3200.02, p. 7.
2. OCNGS: TCR 828.38, LO J.

KA: 295001G011 [3.9/4.2]

295001G011 ..(KA's)

ANSWER: 069 (1.00)

c.

REFERENCE:

1. OCNGS: ABN-3200.02, p. 16.
2. OCNGS: TCR 801.01, LO C.

KA: 295001G007 [3.3/3.6]

295001G007 ..(KA's)

ANSWER: 070 (1.00)

d.

REFERENCE:

1. OCNGS: 2000-RAP-3024.03, Q-2-a and Q-3-a.
2. OCNGS: TCR 828.51, LO H.
3. EQB: 17984 (1991 exam, modified)

KA: 295004G006 [3.6/3.8]

295004G006 ..(KA's)

ANSWER: 071 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.13, p. 4.
2. OCNGS: TCR 828.12, LO O.
3. OCNGS: Facility Question 272 (Duty Area 263)

KA: 295004A102 [3.8/4.1]

295004A102 ..(KA's)

ANSWER: 072 (1.00)

d.

REFERENCE:

1. OCNGS: ABN-3200.10, p. 3.
2. OCNGS: 828.50, LO J.

KA: 295005A101 [3.1/3.3]

295005A101 ..(KA's)

ANSWER: 073 (1.00)

b.

REFERENCE:

1. OCNGS: ABN-3200.01, p. 5.
2. OCNGS: TCR 801.01, LO A.1

KA: 295008A108 [3.5/3.5]

295008A108 ..(KA's)

ANSWER: 074 (1.00)

c.

REFERENCE:

1. OCNGS: ABN-3200.19, p. 6.
2. OCNGS: TCR 828.35, LO P.

KA: 295018K202 [3.4/3.6]

295018K202 ..(KA's)

ANSWER: 075 (1.00)

c

REFERENCE:

1. OCNGS: ABN-3200.19, p. 4
2. OCNGS: TCR 828.35, LO I.
3. EQB: 18052 (1991 exam)

KA: 295018G006 [3.5/3.4]

295018G006 ..(KA's)

ANSWER: 076 (1.00)

c.

REFERENCE:

1. OCNGS: ABN-3200.35, p. 3.
2. OCNGS: TCR 828.43, LO X.

KA: 295019G010 [3.7/3.4]

295019G010 ..(KA's)

ANSWER: 077 (1.00)

d.

REFERENCE:

1. OCNGS: OPM 43, p. 25-31.
2. OCNGS: TCR 828.45, LO O.

KA: 295019K209 [3.3/3.3]

295019K209 ..(KA's)

ANSWER: 078 (1.00)

c.

REFERENCE:

1. OCNGS: OPM 32, p. 46-48.
2. OCNGS: 828.32, LO F.
3. EQB: 15346 (1990 exam)

KA: 295020K203 [3.1/3.3]

295020K203 ..(KA's)

ANSWER: 079 (1.00)

a.

REFERENCE:

1. OCNGS: EMG-3200.01B
2. OCNGS: TCR 845.19, LO B

KA: 295020A101 [3.6/3.6]

295020A101 ..(KA's)

ANSWER: 080 (1.00)

c.

REFERENCE:

1. OCNGS: Technical Specification 3.5.A.3.a.(2), Page 3.5-3
2. OCNGS: 828.45, LO K.
3. EQB: 17649 (1992 exam)

KA: 295021G003 [3.1/3.6]

295021G003 ..(KA's)

ANSWER: 081 (1.00)

d.

REFERENCE:

1. OCNGS: 2000-OPS-3024.27 section 4.0 page 9.0
2. OCNGS: TCR 828.45, LO L.
3. EQB: 15345 modified (1990 exam)
   KA: 295021K104 [3.6/3.7]

295021K104 ..(KA's)

ANSWER: 082 (1.00)

d.

REFERENCE:

1. OCNGS: 2000-RAP-3024.01, H-7-c
2. OCNGS: TCR 828.11, LO E.

KA: 295022A102 [3.6/3.6]

295022A102 ..(KA's)

ANSWER: 083 (1.00)

c.

REFERENCE:

1. OCNGS: TCR 845.20, p. 6, LO B.

KA: 295029K207 [3.1/3.2]

295029K207 ..(KA's)

ANSWER: 084 (1.00)

c.

REFERENCE:

1. OCNGS: EPIP-OC-.01, p. 10

KA: 294001A116 [2.9/4.7]

294001A116 ..(KA's)

ANSWER: 085 (1.00)

c.

REFERENCE:

1. OCNGS: Procedure 108, p. 48.
2. OCNGS: TCR 830.01, LO O.
3. OCNGS: Facility Question 1759 (Duty Area 100)

KA: 294001K102 [3.9/4.5]

294001K102 ..(KA's)

ANSWER: 086 (1.00)

a.

REFERENCE:

1. OCNGS: Procedure 108, p. 39
2. OCNGS: TCR 830.01, LO K.

KA: 294001K102 [3.9/4.5]

294001K102 ..(KA's)

ANSWER: 087 (1.00)

b.

REFERENCE:

1. OCNGS: Facility Question 0921 (Duty Area 100)
2. 10 CFR 26

KA: 294001A103 [2.7/3.7]

294001A103 ..(KA's)

ANSWER: 088 (1.00)

c.

REFERENCE:

1. OCNGS: Procedure 122, p. 5
2, OCNGS: TCR 830.05, LO CH.

KA: 294001K105 [3.2/3.7]

294001K105 ..(KA's)

ANSWER: 089 (1.00)

b.

REFERENCE:

1. OCNGS: Technical Specifications Section 6.2.2.2 a. - d.
2. OCNGS: TCR 830.05, LO BF.
3. EQB: 15283 (1990 exam)

KA: 294001A103 [2.7/3.7]

294001A103 ..(KA's)

ANSWER: 090 (1.00)

d.

REFERENCE:

1. OCNGS: TCR 839.05, LO BA.

KA: 294001A113 [4.5/4.3]

294001A113 ..(KA's)

ANSWER: 091 (1.00)

a.

REFERENCE:

1. OCNGS: Procedure 106, section 4.2.4, p. 13.
2. OCNGS: TCR 830.05, LO AG.

KA: 294001A106 [3.4/3.6]

294001A106 ..(KA's)

ANSWER: 092 (1.00)

b.

REFERENCE:

1. OCNGS: EPIP-OC-.03, p. E1-3
2. OCNGS: TCR 830.05, LO CI.2.

KA: 294001A116 [2.9/4.7]

294001A116 ..(KA's)

ANSWER: 093 (1.00)

d.

REFERENCE:

1. OCNGS: Procedure 106.2.1, p. 3.
2. OCNGS: TCR 830.05, LO BL.

KA: 294001K103 [3.3/3.8]

294001K103 ..(KA's)

ANSWER: 094 (1.00)

c.

REFERENCE:

1. OCNGS: 6630-ADM-4110.01, p. 3.
   KA: 294001K103 [3.3/3.8]

294001K103 ..(KA's)

ANSWER: 095 (1.00)

a.

REFERENCE:

1. OCNGS: Procedure 106, p. E2-1.
   KA: 294001A110 [3.6/4.2]

294001A110 ..(KA's)

ANSWER: 096 (1.00)

b.

REFERENCE:

1. OCNGS: Procedure 106, section 4.4.6, p. 27
   KA: 294001A106 [3.4/3.6]

294001A106 ..(KA's)

ANSWER: 097 (1.00)

d.

REFERENCE:

1. OCNGS: Procedure 106 pg 58
2. OCNGS: TCR 830.05 LO AD.
3. EQB: 18022 (1991 exam)

KA: 294001A102 [4.2/4.2]

294001A102 ..(KA's)

ANSWER: 098 (1.00)

a.

REFERENCE:

1. OCNGS: EPIP-OC-.02, Exhibit 2.

KA: 294001A116 [2.9/4.7]

294001A116 ..(KA's)

ANSWER: 099 (1.00)

c.

REFERENCE:

1. OCNGS: TCR 830.05, p. 38, LO BH.
2. OCNGS: Technical Specification 6.2.2.2.i.
3. EQB: 17562 modified (1992 exam)

KA: 294001A109 [3.3/4.2]

294001A109 ..(KA's)

ANSWER: 100 (1.00)

c.

REFERENCE:

1. OCNGS: Procedure 233, p. 24.
2. EQB: 17569 (1992 exam)

KA: 294001K105 [3.2/3.7]

294001K105 ..(KA's)

(********** END OF EXAMINATION **********)

ATTACHMENT 2

SIMULATION FACILITY REPORT

Facility License: DPR-16

Facility Docket No: 50-219

Operating Test Preparation and Administration: March 21-23, 1994, and April 5-7, 1994

This form is to be used only to report observations. These observations do not constitute audit or inspection findings and are not, without further verification and review, indicative of noncompliance with 10 CFR 55.45(b). These observations do not affect NRC certification or approval of the simulation facility other than to provide information that may be used in future evaluations. No licensee action is required in response to these observations.

During the conduct of the simulator portion of the operating tests, the following items were observed:

ITEM	DESCRIPTION
	None