



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 110 AND 103 TO

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated February 14, 1994, the Northern States Power Company (NSP or the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The proposed amendments would revise the TS to reflect the new configuration for the Unit 1 480V safeguards bus arrangement (two 480V safeguards buses fed by each 4160V safeguards bus). This would make the specifications the same for both units since the safeguards bus configuration for the two units will become the same during the outage.

2.0 EVALUATION

The new 480V safeguards bus arrangement was reviewed by the NRC as part of the Station Blackout/Electrical Safeguards Upgrade Project in a safety evaluation dated April 28, 1992. An earlier license amendment approved a change to the TS to reflect the modified 480V safeguard bus configuration for Unit 2 (Amendment 96, December 17, 1992). The proposed changes would revise TS Section 3.7.A.3 and 3.7.B.6 to reflect the new configuration for the Unit 1 480V bus arrangement. Because the electrical modification has been previously evaluated and approved and the TS are being changed to reflect the 480V bus configuration that will exist on Unit 1 at the completion of the June 1994 outage, this change is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no

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significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (59 FR 14892). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: May 17, 1994