



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASH' -TON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO.117 TO FACILITY OPERATING LICENSE NO. DPR-70

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NO. 1

DOCKET NO. 50-272

1.0 INTRODUCTION

By letter dated February 23, 1990, and supplemented by letters dated June 28, 1990 and August 8, 1990, Public Service Electric & Gas Company requested an amendment to Facility Operating License No. DPR-70 for the Salem Generating Station, Unit No. 1. The proposed amendment would modify the Technical Specifications (TSs) for the Subcooling Margin Monitor (SMM) and change the TSs for Reactor Vessel Level Instrumentation System (RVLIS) with interim requirements until RVLIS is upgraded. In addition, Tables 3.3-11a and 3.3-11b have been combined into a single table, 3.3-11. The June 28, 1990 supplemental letter is applicable to Unit 2 only. The August 8, 1990 supplemental letter did not increase the scope of the original amendment request and did not affect the staff's original no significant hazards determination.

2.0 EVALUATION

These proposed changes will add specifications for instrumentation dealing with inadequate core cooling to provide assurance that the RVLIS and SMM equipment installed at the facility are operated and maintained within acceptable limits. This proposed change is in response to NUREG-0737, Technical Specifications guidance provided in NRC Generic Letter 83-37 and an additional request (Varga to Uderitz, dated November 17, 1983) for Technical Specifications for Inadequate Core Cooling (ICC) instrumentation. Until the RVLIS can be upgraded, an interim Action Statement is being proposed to eliminate the need for two separate License Change Requests.

In response to the staff evaluation issued on October 31, 1989, the licensee submitted changes to the Technical Specifications for RVLIS to provide interim requirements with a footnote to terminate applicability of this interim action at the end of the appropriate refueling outage when the RVLIS is upgraded. The RVLIS upgrade will be completed during the Salem Unit 1 10th refueling outage (Spring 1992).

Regarding the SMM interim Technical Specification requirements, the licensee requests that the implementation date for this amendment for Unit 1 be prior to startup from the 9th refueling outage (Fall 1990) because the SMM will be upgraded during the outage.

The licensee has incorporated SMM and RVLIS into Table 3.3-11 Accident Monitoring Instrumentation and Table 4.3-11 Surveillance Requirements for Accident Monitoring Instrumentation for Salem Unit 1 Technical Specifications. Regarding the Action Statements for Item 11, SMM, and Item 19, RVLIS, in TS

Table 3.3-11, the licensee has proposed to reference Action Statements 1 and 2 for SMM and RVLIS. However, an additional Action Statement 8 is being added to Table 3.3-11 Notations and is referenced in Item 19, RVLIS, of Table 3.3-11. The reference to Action Statement 8 in Table 3.3-11, Item 19, RVLIS also has a "\*\*\*\*" footnote associated with it. These Action Statements are given as follows:

- ACTION 1 With the number of OPERABLE accident monitoring channels less than the Required Number of Channels shown in Table 3.3-11, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in HOT SHUTDOWN within the next 12 hours.
- ACTION 2 With the number of OPERABLE accident monitoring channels less than the Minimum Number of Channels shown in Table 3.3-11, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in HOT SHUTDOWN within the next 12 hours.
- ACTION 8 With the number of OPERABLE Channels one less than the Required or Minimum number of channels shown in Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 48 hours or:
1. Operation may proceed provided the Required Channels shown in Table 3.3-11 for the Reactor Coolant System Subcooling Margin Monitor and the Core Exit Thermocouples are OPERABLE. With the number of OPERABLE channels for the Reactor Coolant System Subcooling Margin Monitor and the Core Exit Thermocouples shown in Table 3.3-11 less than the Required Number of Channels, follow the associated Action Statement, and
  2. Restore the system to OPERABLE status at the next scheduled CHANNEL CALIBRATION (which shall be performed upon the next entry into MODE 5, COLD SHUTDOWN).

The "\*\*\*\*" footnote associated with Table 3.3-11, Item 19, RVLIS, is as follows:

(\*\*\*) Action 8 remains in effect until startup from the Unit 1 10th refueling outage at which time, PSE&G will have installed the upgraded RVLIS. Upon expiration, Actions 1 and 2 will apply.

We have reviewed these proposed Technical Specifications for SMM and RVLIS and our findings follow:

- (1) The Action Statements 1 and 2 are acceptable because they are consistent with the GL 83-37 guidance.
- (2) The proposed implementation date (Fall 1990) for Salem Unit 1 SMM Technical Specification is acceptable. The proposed Action Statements for Salem Unit 1 SMM are in full compliance with the GL 83-37 guidance, and are therefore also acceptable.
- (3) The proposed Technical Specification for Salem Unit 1 RVLIS will Action Statements 1, 2, and 8 with "\*\*\*\*" is acceptable because the GL 83-37 guidance cannot be met until the RVLIS is upgraded (Spring 1992).

The staff has reviewed the Salem licensee's proposal for SMM and RVLIS Technical Specification revisions in Tables 3.3-11 and 4.3-11 and has found it acceptable. The staff would require the licensee to inform us of the completion of the RVLIS upgrade.

Tables 3.3-11a and 3.3-11b have been combined into a single table, 3.3-11. In combining the tables the column titled "Total Number of Channels" has been omitted. All requirements remain concerning the availability and operability of the instrument channels. This change brings the Unit 1 Technical Specifications into conformance with the Westinghouse Standard Technical Specifications. The staff finds this acceptable.

The June 28, 1990 supplement clarified the original February 23, 1990 amendment request. The "Description of Change" section of the original submittal proposed that Salem Unit No. 1 TSS Tables 3.3-11a and 3.3-11b be combined into a single table. As indicated by the original license amendment request's revised TSS pages, PSE&G intended that Salem Unit No. 2 Tables 3.3-11a and 3.3-11b should also be combined. However, the combining of Tables 3.3-11a and 3.3-11b for Unit 2 was not mentioned in the original submittal. The June 28, 1990 supplemental letter requests that Tables 3.3-11a and 3.3-11b be combined for Unit 2. Therefore, this letter does not apply to Unit 1.

The August 8, 1990 supplement corrected minor differences between the revised technical specification pages contained in the original request and the current Technical Specifications. These differences were outside the area of the proposed revisions. These differences are: In the \*\* footnote of Table 3.3-11, the phrase "means for determining" should have been "means of determining" and the word "Operable" was changed to all caps. In Table 3.3-11, Item 16, Containment Pressure-Wide Range, the reference to Action 1,2 should have been Action 7,2. The staff finds these corrections acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted

area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (55 FR 21979) on May 30, 1990 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

#### Principal Contributor:

T. Huang

Dated: February 12, 1991