



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY
DOCKET NO. 50-280
SURRY POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. DPR-32

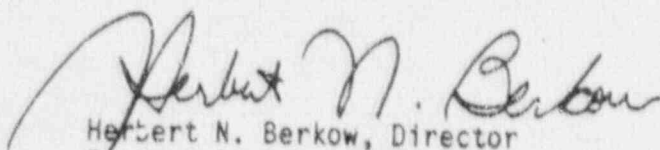
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated June 26, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 152, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20545

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 149
License No. DPR-37

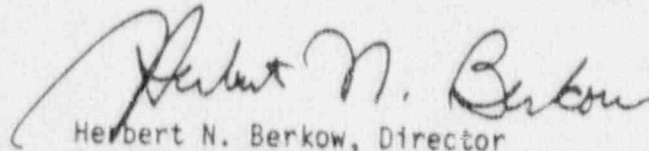
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated June 26, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. According to the attachment, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 149, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1991

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 152 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 149 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS ii
TS 3.8-2
TS 3.8-4

Insert Pages

TS ii
TS 3.8-2
TS 3.8-4

TECHNICAL SPECIFICATION
TABLE OF CONTENTS

<u>SECTION</u>	<u>TITLE</u>	<u>PAGE</u>
3.15	DELETED	
3.16	EMERGENCY POWER SYSTEM	TS 3.16-1
3.17	LOOP STOP VALVE OPERATION	TS 3.17-1
3.18	MOVABLE INCORE INSTRUMENTATION	TS 3.18-1
3.19	MAIN CONTROL ROOM BOTTLED AIR SYSTEM	TS 3.19-1
3.20	SHOCK SUPPRESSORS (SNUBBERS)	TS 3.20-1
3.21	FIRE PROTECTION FEATURES	TS 3.21-1
3.22	AUXILIARY VENTILATION EXHAUST FILTER TRAINS	TS 3.22-1
3.23	CONTROL AND RELAY ROOM VENTILATION SUPPLY FILTER TRAINS	TS 3.23-1
4.0	<u>SURVEILLANCE REQUIREMENTS</u>	TS 4.0-1
4.1	OPERATIONAL SAFETY REVIEW	TS 4.1-1
4.2	AUGMENTED INSPECTIONS	TS 4.2-1
4.3	ASME CODE CLASS 1, 2, AND 3 SYSTEM PRESSURE TESTS	TS 4.3-1
4.4	CONTAINMENT TESTS	TS 4.4-1
4.5	SPRAY SYSTEMS TESTS	TS 4.5-1
4.6	EMERGENCY POWER SYSTEM PERIODIC TESTING	TS 4.6-1
4.7	MAIN STEAM LINE TRIP VALVE	TS 4.7-1
4.8	AUXILIARY FEEDWATER SYSTEM	TS 4.8-1
4.9	EFFLUENT SAMPLING AND RADIATION MONITORING SYSTEM	TS 4.9-1
4.10	REACTIVITY ANOMALIES	TS 4.10-1
4.11	SAFETY INJECTION SYSTEM TESTS	TS 4.11-1
4.12	VENTILATION FILTER TESTS	TS 4.12-1
4.13	DELETED	
4.14	DELETED	

6. Positive reactivity changes shall not be made by rod drive motion or boron dilution unless the containment integrity is intact
7. The containment isolation valves shall be listed in Tables 3.8-1 and 3.8-2.

B. Internal Pressure

1. If the internal air partial pressure rises to a point 0.25 psi above the allowable value of the air partial pressure (TS Fig. 3.8-1), the reactor shall be brought to hot shutdown within 6 hours.
2. If the leakage condition cannot be corrected without violating the containment integrity or if the internal partial pressure continues to rise, the reactor shall be brought to cold shutdown within the next 30 hours.
3. If the internal pressure falls below 8.25 psia, the reactor shall be placed in hot shutdown within 6 hours and in cold shutdown within the next 30 hours.
4. If the air partial pressure cannot be maintained greater than or equal to 9.0 psia, the reactor shall be brought to hot shutdown within 6 hours.

Basis

The Reactor Coolant System temperature and pressure being below 350°F and 450 psig, respectively, ensures that no significant amount of flashing steam will be formed and hence that there would be no significant pressure buildup in the containment if there is a loss-of-coolant accident.

If the containment air partial pressure rises to a point 0.25 psi above the allowable value, the reactor shall be brought to the hot shutdown condition. If a LOCA occurs at the time the containment air partial pressure is 0.25 psi above the allowable value, the maximum containment pressure will be less than 45 psig, the containment will depressurize in less than 1 hour, and the maximum subatmospheric peak pressure will be less than 0.0 psig.

If the containment air partial pressure cannot be maintained greater than or equal to 9.0 psia, the reactor shall be brought to the hot shutdown condition. The shell and dome plate liner of the containment are capable of withstanding an internal pressure as low as 3 psia, and the bottom mat liner is capable of withstanding an internal pressure as low as 8 psia.

References

FSAR Section 4.3.2	Reactor Coolant Pump
FSAR Section 5.2	Containment Isolation
FSAR Section 5.2.1	Design Bases
FSAR Section 5.5.2	Isolation Design
FSAR Section 6.3.2	Containment Vacuum System