



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 10, 1991

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for January, 1991.

Very truly yours,

WRo
G. J. Diederich
for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1991

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT
- D. LICENSEE EVENT REPORTS
- E. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- F. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, generator on-line at 1130 MWE
	0215	Reduced power to 1050 MWE for CRD exercising.
	1600	Increased power to 1130 MWE.
8	0200	Reduced power to 850 MWE for CRD exercising, quarterly surveillances and to perform rod set.
	1800	Increased power to 1135 MWE.
11	2345	Reduced power to 890 MWE for system load.
12	0800	Increased power to 1075 MWE.
	2330	Reduced power to 745 MWE for heater drain work.
14	0300	Increased power to 1135 MW.
15	0100	Reduced power to 950 MWE for CRD exercising and for rod withdrawal.
	1300	Increased power to 1138 MWE.
22	0100	Reduced power to 750 MWE for rod set.
	1900	Increased power to 1136 MWE.
28	0030	Reduced power to 1000 MWE for feedwater pump testing.
	0600	Increased power to 1128 MWE.
29	0030	Reduced power to 970 MWE for feedwater check valve testing and rod set.
	1400	Increased power to 1136 MWE.
31	2400	Reactor critical, generator on line at 1135 MWE.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION
(None)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including
SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
90-001-00	01/23/91	Failure of relay in the Reactor Building ventilation system causes Engineered Safety Feature isolation.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L02989	1E51-C003 RCIC Water Leg Pump	Pump casing leaking oil	None	Replaced pump
L03899	1E31-N614A RHR EQ Area Ventilation	Differential temperature switch	None	Replaced switch
L04386	Hydraulic Control Unit 30-59	Bent valve stem	Degraded control rod operation	Replaced valve
L04836	OXY-VC165B Ammonia Detector	Worn cassette carrier guide posts	None	Rebuilt unit
L04838	OXY-VC125B Ammonia Detector	Worn cassette carrier guide posts	None	Rebuilt unit
L05043	1VR05YA Reactor Building Isolation Damper	Faulty relay	None	Replaced relay

(See attached SOR Failure Report)

SOR dr Switch Failure Data Sheet

Equipment Piece Number: 1E31-N007AA Model Number: 103AS-B203-NX-JJTTX6

Serial Number: 86-6-2517

Application: RCIC High Steam Line Flow Isolation Switch

Date and Time of Discovery: 01/15/91 1345 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 110.4 - 112.4 "WC

Nominal Setpoint: 123.0 "WC

Action Limits: <106.0 or >116.8 "WC

Reject Limits: <102.8 or >120.2 "WC

Technical Specification

Limit: 128.0 "WC

As Found Setpoint: --- "WC

Date and Time of Return to Service: 01/19/91 0335 hours

Model Number of Replacement Switch: 103AS-B203-NX-JJTTX6

Serial Number of Replacement Switch: 90-8-6256

DVR Number: 1-1-91-002

Cause: Switch would not hold pressure during functional testing.
During inspection the switch was found to have a hole in the diaphragm.
The cause of the hole could not be determined.

Corrective Action: Replaced switch. Inspected failed switch.

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE FEBRUARY 10, 1991
COMPLETED BY K.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

- | | | | |
|--|--------------|-------------------------------------|-------|
| 1. REPORTING PERIOD: | JANUARY 1991 | GROSS HOURS IN REPORTING PERIOD | 744 |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (MW): | 2,323 | MAX DEMAND CAPACITY (MWe-Net): | 1,036 |
| | | DESIGN ELECTRICAL RATING (MWe-Net): | 1,078 |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): | | N/A | |
| 4. REASONS FOR RESTRICTION (IF ANY): | | | |

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	41,353.4
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	40,492.8
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MMBTU)	2,404,486	2,404,486	116,608,069
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	828,925	828,925	30,832,213
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	802,635	802,635	37,172,717
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	66.6
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	69.2
14. UNIT SERVICE FACTOR (%)	100.0	100.0	65.2
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	65.2
16. UNIT CAPACITY FACTOR (USING MDC) (%)	104.1	104.1	57.7
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	100.1	100.1	55.5
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	0.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

Refueling (L1R04) 02/16/91 10 WEEKS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

N/A

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MW-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE FEBRUARY 10, 1991
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-327-6761

REPORT PERIOD: JANUARY 1991

<u>DAY</u>	<u>POWER</u>	<u>DAY</u>	<u>POWER</u>
1	1,089	17	1,107
2	1,107	18	1,107
3	1,105	19	1,103
4	1,100	20	1,101
5	1,100	21	1,087
6	1,098	22	981
7	1,089	23	1,111
8	1,021	24	1,107
9	1,103	25	1,108
10	1,107	26	1,103
11	1,093	27	1,096
12	977	28	1,084
13	820	29	1,059
14	1,098	30	1,103
15	1,065	31	1,112
16	1,105		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None)

SUMMARY OF OPERATION:

The Unit remained on line at high power throughout the month. Several minor power reductions were required due to low grid demand, routine surveillances, and feedwater system testing.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None)

2. ECCS System Outages
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual
(None)

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
5	1DG011	Administrative control.
12	1E12-F052A	Perform surveillance LES-EQ-012.
48	1E51-C003	Rebuild water leg pump.
61	1E51-C003	Change water leg pump minimum flow orifice.
77	1D002P	Lubricated coupling.
79	1E22-S001	Replaced air start solenoids.
84	1E22-S001	Change soak back oil pump filter.
97	1E12-F052	Remove spring pack for test.
105	1DG01K	Replaced solenoids
106	1DG01P	Breaker inspection.
109	1E12-C300D	Repair cable jacket at switchgear breaker.
113	1E12-F336B	Valve maintenance.
118	1E22-C001	Perform LES-GM-129.

LASALLE NUCLEAR POWER STATION
UNIT 2
MONTHLY PERFORMANCE REPORT
JANUARY 1991
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(Unit 2)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT
- D. LICENSEE EVENT REPORTS
- E. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- F. UNIQUE REPORTING REQUIREMENTS
 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, generator on line at 1130 MWE.
	2300	Reduced power to 1070 MWE for CRD exercising.
2	1000	Increased power to 1132 MWE.
9	0130	Reduced power to 1045 MWE for CRD exercising.
	0730	Increased power to 1130 MWE.
15	2340	Reduced power to 1060 MWE for CRD exercising.
16	1100	Increased power to 1133 MWE.
23	0200	Reduced power to 1070 MWE for CRD exercising.
	0930	Increased power to 1130 MWE.
26	0400	Reduced power to 750 MWE to perform monthly/quarterly surveillances and for rod set.
	1200	Increased power to 1100 MWE.
27	1200	Increased power to 1132 MWE.
30	0030	Reduced power to 1064 MWE for CRD exercising.
	1400	Increased power to 1130 MWE.
31	2400	Reactor critical, generator on line at 1130 MWE.

- B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION
(None)
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including
SOR differential pressure switch failure reports).
(See Table 1)
- D. LICENSEE EVENT REPORTS (Unit 2)
- | <u>LER Number</u> | <u>Date</u> | <u>Description</u> |
|-------------------|-------------|---|
| 91-001-00 | 01/10/91 | Fire rated assemblies found inoperable during inspection. |
- E. DATA TABULATIONS (Unit 2)
1. Operating Data Report.
(See Table 2)
 2. Average Daily Unit Power Level.
(See Table 3)
 3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L97984	2B DG Lube Oil Circ. Pump Breaker	Pushbutton	None	Replaced pushbutton

(No SOR failures this month)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE FEBRUARY 10, 1991
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS:

1. REPORTING PERIOD:	JANUARY 1991	GROSS HOURS IN REPORTING PERIOD:	744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWH):	3,323	MAX DEMAND CAPACITY (MWe-Net):	1,036
		DESIGN ELECTRICAL RATING (MWe-Net):	1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):		N/A	
4. REASONS FOR RESTRICTION (IF ANY):			

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	37,234.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	36,591.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	6.0
9. THERMAL ENERGY GENERATED (MMH)	2,450,520	2,450,520	107,889,695
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	832,572	832,572	35,716,404
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	806,342	806,342	34,225,432
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	67.5
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	70.7
14. UNIT SERVICE FACTOR (%)	100.0	100.0	66.4
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	66.4
16. UNIT CAPACITY FACTOR (USING MDC) (%)	164.6	104.6	59.9
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	100.5	100.5	57.6
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	14.6

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
(NONE)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
N/A

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LAFALLE TWO
DATE FEBRUARY 10, 1991
COMPLETED BY H.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: JANUARY 1991

<u>DAY</u>	<u>POWER</u>	<u>DAY</u>	<u>POWER</u>
1	1,090	17	1,092
2	1,077	18	1,094
3	1,093	19	1,094
4	1,092	20	1,093
5	1,092	21	1,092
6	1,093	22	1,092
7	1,093	23	1,085
8	1,091	24	1,091
9	1,082	25	1,089
10	1,092	26	923
11	1,093	27	1,085
12	1,090	28	1,086
13	1,090	29	1,087
14	1,094	30	1,074
15	1,092	31	1,092
16	1,073		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	(YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None)

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required for routine surveillances.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT
(None)				

2. ECCS System Outages
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual.
(None)

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1	2DG08CA	Lubrication maintenance.
2	2E51-F360	Replaced motor.
9	2E12-C300A 2E12-C300B	Coupling lubrication.
12	2E12-F087A	Administrative control.
15	2E21-C001	Pump lubrication.
26	2E12-C003	Pump lubrication.
30	2E12-C300C 2E12-C300D	Coupling lubrication.
35	2E22-S001	Soak back oil pump maintenance.
38	2E22-C003	Coupling lubrication.
39	2E22-F349 2E22-F350	Reroute piping.
40	2E22-S001	Replace circuit breaker.
41	2E22-S001	Lubrication.
46	2E22-F349 2E22-F350	Weld repair.
50	2E51-C003	Replace orifice plate.
52	2E51-F054	Replace cable.