



BOSTON EDISON
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

R. A. Anderson
Vice President &
Station Director
Nuclear Operations

February 13, 1991
BECO Ltr. #91-014

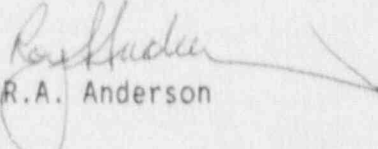
U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

Subject: January 1991 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


R.A. Anderson

WJM/bal

Attachment

cc: Regional Administrator, Region 1
U.S. Nuclear Regulatory Commission
475 Allendale Rd.
King of Prussia, PA 19406

Senior Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
UNIT Pilgrim 1
DATE February 13, 1991
COMPLETED BY W. Munro
TELEPHONE (508) 747-8474

MONTH January 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>355</u>	17	<u>664</u>
2	<u>475</u>	18	<u>664</u>
3	<u>505</u>	19	<u>663</u>
4	<u>630</u>	20	<u>665</u>
5	<u>664</u>	21	<u>665</u>
6	<u>662</u>	22	<u>664</u>
7	<u>663</u>	23	<u>665</u>
8	<u>663</u>	24	<u>665</u>
9	<u>664</u>	25	<u>663</u>
10	<u>664</u>	26	<u>663</u>
11	<u>628</u>	27	<u>665</u>
12	<u>620</u>	28	<u>664</u>
13	<u>664</u>	29	<u>664</u>
14	<u>665</u>	30	<u>664</u>
15	<u>664</u>	31	<u>663</u>
16	<u>664</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE February 13, 1991
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period January 1991
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe) None

10. Reasons For Restrictions, If Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>159072.0</u>
12. Number Of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>93344.7</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>89660.1</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1433016.0</u>	<u>1433016.0</u>	<u>16767165.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>493840.0</u>	<u>493840.0</u>	<u>52141354.0</u>
18. Net Electrical Energy Generated (MWH)	<u>475427.0</u>	<u>475427.0</u>	<u>50101835.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>56.4</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>56.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>95.4</u>	<u>95.4</u>	<u>47.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>97.6</u>	<u>97.6</u>	<u>48.1</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>12.7</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Refueling Outage No. 8, May 1991, approximately 70 days</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup N/A

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Operational Summary for January 1991

The unit started the reporting period at approximately 76 percent power with the "C" Reactor Feedwater Pump (RFP) secured for repairs. On January 1, 1991 at 1000 hours a power reduction to approximately 50 percent was made to conduct a backwash of the main condenser, and to replace the brushes on the "A" Recirculation Motor Generator Set. At 2043 hours the "A" Recirculation Motor Generator Set was restarted, and by 0303 hours on January 2, 1991 reactor power was increased to 76 percent. On January 3, 1991 power was reduced to approximately 52 percent to perform a control rod pattern change. Power was again increased to 100 percent on January 4, 1991 and remained there until January 11, 1991 when at 2009 hours, maintenance personnel, welding on the turbine building exhaust fan above the "A" RFP, reported smoldering on the roof. Power was reduced to 70 percent and the "A" RFP was secured. An Unusual Event was declared at 2031 hours and terminated at 2201 hours. On January 12, 1991 the "A" RFP was returned to service, power was increased to 100 percent and remained there for the remainder of the reporting period. Power reductions to 90 percent were performed on January 6, 11, 19 and 26 for control rod exercising.

Safety Relief Valve Challenges
Month of January 1991

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: Second Quarter 1991
3. Scheduled date for restart following refueling: Third Quarter 1991
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1986/87 refueling outage was of the same design as loaded in the previous outage, and consisted of 192 assemblies.
7. (a) There are 580 fuel assemblies in the core.
(b) There are 1320 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1000 fuel assemblies.

PILGRIM NUCLEAR POWER STATION
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Feed-water System	"C" Reactor Feed-water Pump (RFP) suction relief valve standpipe.	Pipe cracked and leaking. (F&MR 90-426)	Physical contact of the two (2) inch outlet pipe with the wall caused a twisting moment on the inlet standpipe.	Installed Temporary Modification TM 91-01 consisting of a collar welded together axially, and then welded around the "C" RFP suction relief valve (PSV-3444) inlet pipe. The two (2) inch outlet pipe was shortened to remove interference with the wall.	Replacement of inlet pipe during RFO-8.	N/A
Reactor Core Isolation Cooling System	Control cable between the Electronic Speed Controller (EGM) and the Electro-Hydraulic Valve Actuator (EGR).	Turbine tripped on high steam flow. (F&MR 91-33).	Faulty control cable due to environmental conditions resulting from seat leakage on MO 1301-61 valve.	Replaced control cable and signal converter card. Performed Proc. 8.E.13.1 successfully to prove operability.	Refurbish MO1301-61 valve. (MR 19013040)	LER 91-001-00 (To be issued)
Reactor Core Isolation Cooling System	Turbine Exhaust line.	Failed rupture disc PSD-9 (F&MR 90-128).	Extreme environmental conditions resulting from seat leakage on MO1301-61 valve.	Installed new rupture disc.	Refurbish MO1301-61 valve. (MR 19013040)	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293

NAME Pilgrim 1

DATE February 13, 1991

COMPLETED BY W. Munro

TELEPHONE (508) 747-8474

REPORT MONTH January 1991

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
18	01/01/91	F	0.0	N/A	5	N/A	N/A	N/A	Power reduction to backwash main condenser and replace brushes on "A" Recirc. MG set.

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)