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Re: 10CFR50.73(a)(2)(i) February 5, 1991 MP-91-112

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65

Docket No. 50-336

Licensee Event Report 90-004-01

Gentlemen:

This letter forwards update Licensee Event Report 90-004-01 required to be submitted pursuant to 50.73(a)(2)(i) and in response to the Noti e of Violation, Inspection Report 50-336/90-1.

Very truly yours.

NORTHEAST NUCLEAR ENERGY COMPANY

Director, Millstone Station

SES/JMB:mo

Attachment: LER 90-004-01

T. T. Martin, Region I Administrator

W. J. Raymond, Senior Resident Inspector, Millstone Unit Nos. 1, 2 and 3

G. S. Vissing, NRC Project Manager, Millstone Unit No. 2

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NRC Form 366		FAR REGIS ATORY COMMERCION	* DDD AVE A			
U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER)			EXPRES 4/30/92 Estimated burden per rasponse to comply with this information bollection request 50.0 hrs. Forward comments regarding burden estimate to the Records and Reports Management Branch (p-530). U.S. Nuclear			
			the Paperwork Reduction Management and Budget	Washington, DC 20555, and to Project (3150-0104). Office of Washington, DC 20503		
Millstone Nuclear Power Station Unit 2			DOCKET NUMBER (2) PAGE (A)			
TITLE (4)	WHISTORE NUCLEAR FOW	ver station Chit 2	0 6 0	1 1 0 3 3 6 1 0 F 0 2		
Control	Roon, Ventilation Techn.	ical Specification Violation				
EVENT DATE (6)	ER NUMBER (6)	REPORT DATE (7)	OTHER FACILITIE	S INVOLVED (6)		
MONTH DAY YEA		MEER MONTH DAY YEAR	FACILITY NAMES	015101010111		
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CFNERATING MODE (9)	THIS REPORT IS BEING SUB	MITTED PURBUANT TO THE REQUIR	EMENTS OF 10 OFR \$ (Check	one or more of the following:(11)		
	20.402(b)	\$0.402(c)	50.73(a)(2)(iv)	79.71(b)		
POWER 0101	20:406(a)(1)(f) 0 20:406(a)(1)(f)	50.36(c)(1)	50,73 (a) (2) (v)	73.71(0)		
	20.405(a)(1)(lil)	50.38(c)(2) x 50.73(a)(2)(ii)	50.73 (a)(2)(vii) 60.79(a)(2)(viii)(A)	OTHER (Specify in Abstract below and in Text NRC Form 366A)		
	20 405(a)(1)(iv)	50.73(a)(2)(II)	50.73(a)(2)(viii)(B)			
	20.406(a)(1)(iv)	50.73(4)(2)(10)	50.73(a)(2)(x)			
***************************************		LICENSEE CONTACT FOR THIS	LER (12)			
NAME			AREA	TELEPHONE NUMBER		
Joseph N	1. Bergin, Ext. 5352		21.0	3 4 4 4 7 - 1 7 9 1		
	COMPLETE ONE LINE F	OR EACH COMPONENT FAILURE DE	SCAIBED IN THIS REPORT (13			
DAUSE SYSTEM CO	MANUERT MANUERO-	CAUSE SYS	TEM COMPONENT MANUF	AC - REPORTABLE TO NAMES		
	SUPPLEMENTAL REP	PORT EXPECTED (14)	EXPE	CTED MONTH DAY YEAR		
YES (If yes, co	185/ON [15]					
ABSTRACT (Limit to	1400 spaces. Le approximately	fifteen single-space typewritten lines	(16)			
satisfying Te requires that one emerger or initiate ar mode. The seven days of CRAC was pon February placed in no of Technical emergency of operation not apply for change has a mode when	two independent emergency ventilation system into individual maintain the operable definition of OPERABIL of performing Maintenance at 00:25 hours, the system and until 16:31 hours of Specification 3.05 which is normal power supply become supply is operable a with the CRAC in the new Mode 5 or 6 and the Coeen processed to eliminate other facility is inoperable of the other facility is inoperable.	in Mode 6, Refueling, it was 6.1, for Control Room Air ency ventilation systems be operable, restore the inoperation control room emergency and ITY requires an Emergency te on the Emergency Diesels mode of operation, on February 28. The CRAC is states that when a system eing inoperable, it may be control in the recurrence of the requirement for place that the requirement for place that the requirement for place trable due only to its associations as a result of this in	Conditioning, (CRAC) OPERABLE, while in relationship to operable release up system in the Diesel Generator to be Generator for the "A" bruary 27 at 22:26 hour the recirculation mode was taken out of recirculationsidered operable for operable. After appeted that Technical Specific and the operable CRAC and the operable CRAC attention of the control o	This Specification modes 5 or 6. With status within 7 days e recirculation e available. After facility the "B" ars. Two hours later e of operation and reulation on the basis solely because of its the normal or proximately 16 hours cification 3.05 did echnical Specification in recirculation		

NAC Form 366A

U. S. NUCLEAR REGULATORY COMMISSION

APPROVED DIVE NO. 3150-0104 EXPIRES: 4/30/92

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Estimated burden per response to comply with this information collection request 50.0 hrs. Forward comments regarding burden estimate to the Records and Reports Management Branch (p-630). U. 5. Nuclear Regulatory Commission, Washington, DC 20555, and to the Paperwork Reduction Project (3150-0104). Office of Management and Budget, Washington, DC 20503.

FACILITY NAME (1)	DOOKET NUMBER (2)		LER NUMBER (6)	PAGE	
		YEAR	RECLIENTIAL PREVISION NUMBER NUMBER		
Millstone Nuclear Power Station					
Unit 2	0 5 0 0 0 3 3 6	910	0 0 1 4 0 1 1	012 OF 012	

TEXT (If more space is required, use additional NRC Form 366A ti) (17)

1. Description of Event

On February 28, 1989 at 16:00 hours, with the Unit in mode 6. Refueling at 79 degrees, it was determined that the Unit had operated outside of the Technical Specifications for Control Room Air Conditioning System for 16 hours. During the course of normal scheduled maintenance the "A" Emergency Diesel Generator was removed from service. This activity is allowed per the Unit's Technical Specification 3.8.1.2.

After seven days of performing maintenance on the "A Facility" Emergency Diesel Generator, the "B" CRAC was placed in the recirculation mode of operation, on February 27 at 22:26 hours, per the requirements of Technical Specification 3.7.6.1. Two hours later on February 28 at 00:25 hours, the "B" CRAC was taken out of the recirculation mode of operation and left in normal until 16:31 hours on February 28. The CRAC was taken out of recirculation on the basis of Technical Specification 3.05 which states that when a system is declared inoperable solely because its emergency or normal power supply is inoperable, it may be considered operable in the normal or emergency power supply is operable and all redundant systems are operable. After approximately 16 hours of operation with the CRAC in the normal mode it was determined that Technical Specification 3.05 did not apply for Mode 5 or 6 and the CRAC was placed in the recirculation mode.

II. Cause of Event

The root cause of this event is an inconsistency in the Technical Specifications. During Unit operation in Modes 5 and 6. Technical Specification 3.7.6.1 requires both trains of the Control Room Air Conditioning system be OPERABLE. The definition of OPERABLITY requires all support equipment required to take a component or system OPERABLE, shall also be OPERABLE. Per Technical specification 3.8.1.2 one Diesel Generator can be removed from service in modes 5 and 6. This action will place the Unit in Technical Specification Action Statement 3.7.6.1 a, per the definition of operability.

III. Analysis of Event

This event is being reported pursuant to the requirements of 10 CFR 50.73(a)(2)(i). There are no safety consequences as a result of this event. There were no safety systems out of service as a result of this event. This report is being submitted at this time in response to the requirements of the Notice of Violation Inspection Report 50-336/90-1.

IV. Corrective Action

A Technical Specification Change Request has been approved, which eliminates the requirement for the operability of the Emergency Diesel Generator to support the operability of the affected train of Control Room Air Conditioning while in modes 5 & 6.

V. Additional Information

There were no failed components during this event

Similar LERs: None