

December 13, 1982

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555

Subject: LaSalle County Station Units 1 and 2

Amendment No. 61 to the Final Safety

Analysis Report (FSAR)

NRC Docket Nos. 50-373 and 50-374

Dear Mr. Denton:

Attached is Amendment No. 61 to the LaSalle County Station Units 1 and 2 Final Safety Analysis Report. FSAR Amendment #61 is issued to bring the official licensing record (FSAR) into conformance with the as-built and as-tested configuration of the plant. The update covers seismic qualification efforts completed during early 1982, reanalysis of containment response with added inlet feedwater enthalpy, correction of typographical errors and clarification of references, replacement of the ammonia detector for the control room HVAC; update of the recirculation system description to include the flux estimator, and clarification of leak testing writeup. Additionally an amplified definition of MSIV closure time was made in the description of a startup test. More explicit elaboration on these changes is given below. These updates are informational to bring the FSAR into conformance with the as-built plant. No setpoint changes are involved, no equipment deletions or invalidations are involved.

Specifically, Section 3.9 was re-edited to incorporate the results of the SQRT program and the plant equipment evaluation with the condensation oscillation loads and model refinements made during 1981. The extension of vibratory conditions to 100 hertz from the earlier 33 hertz is reported therein. Dynamic operability test results are included for the major NSSS components, MSIV, and the mainsteam safety-relief valves. Revalidation of the major reactor internals thru extensive normal operations and anticipated transients are now documented. Extensive reanalysis of piping systems to include the NRC agreed-upon load combinations, with condensation oscillation loads, are also included in Section 3.9.3. The adequacy of equipment support structures for floor-mounted equipment has been included also now that the lists are completed.

8212210463 821213 PDR ADOCK 05000373 K PDR 1/60

The containment response to LOCA with less downcomer submergence and with added feedwater inlet enthalpy is included in the update of Chapter 6 to the more conservative assumptions used in the bounding case. This refinement brings the FSAR into agreement with Chapter 6 of the Design Assessment Report. Additionally, several word changes were made in Section 6.4 as a result of completing the leak rate tests. None of these changed the initial intent of the original statements.

Likewise, as a result of system demonstration tests on HVAC equipment which confirmed the mean estimated valves of temperature, humidity, and differential pressure, Sections 7.12 and 9.4.1.2 which deal with power-generation design bases were edited to remove the design tolerances for these variables. Their removal has no safety significance; in fact, such design tolerance numbers should never have appeared in the original FSAR text.

Startup test Table 14.2.-122, the MSIV closure test at various power levels, was amended by inclusion of two definitions of MSIV closure time: one with the electrical circuit-delay time-interval and one without this interval. This enables a more useful measurement during startup while still measuring overall response time for comparison to the include additional test conditions for startup coverage on MSIV actuation in response to prior NRC requests.

Appendix G was updated on several pages to include reference to the flux estimator used in the recirculation system control loop (a non-safety system) and on several other pages to reference the more recent measurement of vessel subcooling resulting from better quality instrumentation which was installed in late 1981.

Typographical corrections and correction of internal references were made as follows: Table 6.2.-21 at penetration M29, pages 7.2-3,4; 7.2-9, 12, 14, 20 and 92; page 8.3-38, page 9.5-14a. Updated reporting of the as-built conditions which were improved as a result of pre-operational testing includes the following: Use of Fyrequell EHC in the hydraulic controls for the flow control valve in the recirculation system as described in Appendix G; update of instrument ranges for the area radiation monitors as recorded in Table 12.3-13; clarification of the list of local alarms as listed on page 7.3.4.5; update of the instrument ranges of the ammonia detectors and the chlorine detectors used in the HVAC ducts to the control room as recorded on pages 7.3.52a, b and 9.4-6 and 9.4-12.

The changes made in fire detection and fire protection in response to Region III field audits are recorded in Sections 8.3, 9.5.1.2 and Appendix H. All updates and corrections to text on fire protection as-built condition of the plant. The applicability of certain changes such as the fire quality of the door frames is fairly broad hence a considerable number of Appendix H pages were updated to reflect the as-built conditions. Sixteen pages were placed on hold pending completion of the door frame adequacy appraisal by a certifying body.

furnished by other Commonwealth Edison employees and contractors. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

Three (3) signed originals and sixty (60) copies of this transmittal are submitted for your use.

If there are any questions in this matter, please contact this office.

Very truly yours,

Childred 12/10/82 C. W. Schroeder

Nuclear Licensing Administrator

1 m

Enclosure

cc: NRC Resident Inspector - LSCS

SUBSCRIBED and SWORN to before me this 13 bloday of december, 1982

Cosale 1 Signit Notary Public