

U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION V

Report No. 50-275/82-35

License No. 50-275 Priority DPR-76 Category _____

Licensee: Pacific Gas and Electric Company

P. O. Box 7442

San Francisco, California 94120

Facility Name: Diablo Canyon Unit 1

Inspection at: Diablo Canyon Site, San Luis Obispo County, California

Inspection conducted: October 25-29, 1982

Inspectors: J. F. Burdoin 11-26-82
J. F. Burdoin, Reactor Inspector Date Signed

Date Signed

Date Signed

Approved by: I. W. Zwetzig for 11/26/82
G. B. Zwetzig, Chief, Engineering Programs Section Date Signed

Summary:

Inspection during the period of October 25-29, 1982
(Report No. 50-275/82-35)

Areas Inspected: Announced inspection by regional-based inspector of construction and modification activities including: a site tour, modifications to piping and electrical raceway supports resulting from the verification program, preservice inspection of the reactor vessel, and eddy current examination of the steam generators.

The inspection involved 37 inspection hours by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

IE:V Form 219(1)

DETAILS

1. Individuals Contacted

a. Pacific Gas and Electric Company (PG&E)

*D. A. Rockwell, Assistant Project Superintendent
W. E. Coley, Resident Electrical Engineer
*F. M. Russell, Resident Civil Engineer
*J. Arnold, Resident Mechanical Engineer
M. E. Leppke, Supervisor, Onsite Engineering Group
K. A. Nilson, Mechanical Field Engineer
B. V. Gragg, Civil/Structural Steel Group Supervisor
*J. R. Bratton, Lead QC Inspector
D. R. Bell, QC Inspector
P. Palomo, Civil Inspector
*R. Dizon, Electrical Inspector
C. W. Young, Mechanical Inspector
M. J. Mello, Mechanical Inspector
*C. M. Seward, QA Engineer
J. S. Diamonon, QC Supervisor
D. A. Gonzales, QC Inspector

Various other engineering and QC personnel.

b. H. P. Foley Company (Foley)

J. L. Thompson, QA Engineering Group Supervisor

c. Pullman Power Products Corporation

H. W. Karner, QA/QC Manager
C. D. Aqueda, QA Controller

* Denotes attendees at exit meeting on October 29, 1982.

2. Site Tour

A tour was made of Unit 1 auxiliary building. At elevation 85' the auxiliary building operator's station and HPSI Pumps 11 and 12 areas were inspected. It was observed that the shaft coupling for Pump 11 was removed. This condition was identified to the licensee's representative.

Other equipment areas inspected in the auxiliary building were: charging Pumps 11, 12 and 13 at elevation 73'; RHR Pumps 11 and 12 at elevation 60'; and penetration areas GE and GW at elevation 115'. In the GE area, insulation observed to be missing from a three inch steam line to the auxiliary feedwater pump turbine was pointed out to the licensee's representative.

No items of noncompliance or deviations were identified.

3. Modifications Resulting from the Verification Program

a. Safety-Related Pipe Support and Restraint Systems, Unit 1

Modifications to the six listed large bore pipe supports located in the containment were examined to determine by visual examination, whether the pipe supports met the requirements specified by licensee's drawings, procedures, and specifications. The following pipe supports, which had been inspected and accepted by the contractor's Quality Control, were examined by the inspector, and appeared to satisfy the applicable requirements of ESD-223, "Installation and Inspection of Class 1 Pipe Supports."

11-52SL	57N-58R
12-159SL	57N-138R
57N-57R	58S-32R

No items of noncompliance or deviations were identified.

b. (Closed) Valve Yoke Stiffening, FCV-95

Modifications to upgrade the stiffening for valve FCV-95 were performed under order DCO-E-M-0876. The modifications consisted of replacing 3/8" thick steel yoke stiffening plates with 1/2" thick steel plates. The valve was inspected in the field and the quality control records of the modifications were examined and found acceptable. This item is closed.

c. Annulus Structural Steel Strengthening

Modifications to strengthen the structural steel in the annulus area of Unit 1 containment, in the early stages of constructions were inspected. The following modifications in process were inspected: eight at elevation 106', five at elevation 117', two at elevation 140' and one at elevation 101'. The inspection of this item is incomplete and will be reported in a subsequent report (50-275/82-35/01).

d. Other Modification Items

The status of the following modifications has been released by engineering, and the Construction Department is scheduled to initiate work on these items in November/December 1982.

1. Fuel handling building, structural steel.
2. Annunciator cabinets, additional seismic bracing.

3. 125V d.c., replace eight circuit breakers.
4. Additional electrical raceway supports.

These items will be reported in a subsequent report.
(50-275/82-35/02).

4. TMI Task Action Plan Item (50-275/82-16/03) II.F.2

The installation of In-core thermocouples cables which had been pulled (but not terminated) to the cold junction boxes in the containment, and between these boxes and the penetrations was reported in inspection report 275/82-16. Termination of these cables in the cold junction boxes and at the inboard side of the containment penetrations was examined in the field. No items of noncompliance or deviations were identified. This item remains open, and the upgraded status will be reported in a subsequent inspection report.

5. (Closed) Preservice Inspection (Baseline), Unit 1

The licensee has contracted with Westinghouse to perform a second preservice examination of the reactor vessels for Units 1 and 2. The Westinghouse team had set up the equipment and were in the process of calibrating, adjusting and working the "Bugs" out of the ultrasonic equipment during the early part of the inspection week. Actual examination of Unit 1 vessel welds did commence toward the latter part of the week. The following welds are to be examined:

- a. Eight Longitudinal welds,
- b. Five horizontal welds,
- c. Six lower dome welds,
- d. Eight nozzle welds, and
- e. Sixteen nozzle/pipe safe-end welds.

The preservice process and equipment setup were inspected in the containment, and the following procedures which describe the preservice inspection program were examined:

OPS-NSD-101, Preservice and Inservice Inspection Documentation

NSD-IST-10, Qualification for Ultrasonic Manual Equipment

ISI-154, Preservice and Inservice Inspection of Reactor Vessels

RV-ISI-01, Reactor Vessel Inspection Program Preparation and Documentation

ISI-47, Manual Ultrasonic Examination of Welds in Vessels.

The qualification records for three of the Westinghouse NDE personnel performing the NDE activities observed during this inspection were reviewed. This included one Level I and two Level II NDE personnel. The persons concerned were qualified to perform ultrasonic testing within their certification levels.

The licensee's preservice inspection program is considered satisfactory.

No items of noncompliance or deviations were identified.

This item is closed for Unit 1.

6. (Closed) Eddy Current Examination of Steam Generators, Unit 1

The licensee has contracted with Conam Inspection of Richmond, California to perform Eddy current examination of the "U" tubes of the steam generators for Units 1 and 2.

The examination of Unit 1 steam generator tubes was in process during this inspection period. The equipment setup and examination process in the containment was inspected. The following procedures which describe the Eddy current program were examined.

Procedure No. 42-MP-007, Rev. 0, Management Plan

Procedure No. 42-PP-008, Rev. 0, Pre-Service Program Plan

Procedure No. 42-EC-059, Rev. 0, Multifrequency Eddy Current Procedure

Procedure No. 42-QC-003, Rev. 0, Equipment Maintenance and Calibration

Procedure No. 42-EC-012, Rev. 0, Verifying Accuracy of Eddy Current Reports

Procedure No. 42-EC-060, Rev. 0, Report Form Procedure

The Eddy current examination of Unit 1, steam generators 3 and 4 tubes was in process. The examination of the tubes (excluding those in three rows closest to the divider plates) in steam generators 1 and 2 had already been completed. There are approximately 3300 tubes in each steam generator. The plans call for the completing the four steam generators and going back and examining the tubes in the three rows adjacent to the divider plates (since the short radius at "U" tube bend, near the divider plates, requires more time to work the probe through the bend).

The licensee's Eddy Current examination of steam generators is considered satisfactory.

No items of noncompliance or deviations were identified.

This item is closed for Unit 1.

7. Management Interview

A meeting was held with the licensee representatives (identified in Paragraph 1) on October 29, 1982 to discuss the scope and findings of the site inspection.