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February 25, 1994

Docket Nos. 50-277 50-278

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Subject: Semi-Annual Effluent Release Report No. 36 July 1, 1993 through December 31, 1993 Peach Bottom Atomic Power Station Unit Nos. 2 and 3

Gentlemen:

Enclosed are two copies of the Semi-Annual Effluent Release Report No. 36, July 1, 1993 through December 31, 1993 for Peach Bottom Atomic Power Station Unit Nos. 2 and 3.

This report is being submitted in compliance with 10 CFR 50.36a (a) (2) and the Technical Specifications of Operating Licenses DPR-44 and DPR-56, and to fulfill the requirements of Regulatory Guide 10.1.

Sincerely,

Mr. Crainey

G. R. Rainev Vice President

GRR:aa

enclosures

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cc:

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- W. L. Schmidt, USNRC Senior Resident Inspector
- R. I. McLean, State of Maryland
- T. T. Martin, Administrator, Region I, USNRC
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PEACH BOTTOM ATOMIC POWER STATION Unit Numbers 2 and 3 Docket Numbers 50-277 and 50-278 10

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SEMI-ANNUAL EFFLUENT RELEASE REPORT

NO. 36

JULY 1, 1993 THROUGH DECEMBER 31, 1993

Submitted to The United States Nuclear Regulatory Commission Pursuant to Facility Operating Licenses DPR-44 and DPR-56

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Technical Concurrences: (for accuracy of information)

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Chemistry Manager

1. INTRODUCTION

In accordance with the Unique Reporting Requirements of Technical Specification 6.9.2h(2) applicable during the reporting period, this report summarizes the Effluent Release Data for Peach Bottom Atomic Power Station Units 2 and 3 for the period July 1, 1993 through December 31, 1993. The notations E and E- are used to denote positive and negative exponents to the base 10, respectively.

The release of radioactive materials during the reporting period was within the Technical Specification limits. There were no changes made to the Off-Site Dose Calculation Manual (ODCM) during the reporting period.

Estimated particulate, iodine, and fission gas releases were determined for a 7-day period when an improperly secured main stack particulate and cartridge filter assembly resulted in diluted sample flow (PBAPS LER 2-93-15).

There were two unplanned releases of liquid radioactive material.

Table 1A Page 1 of 1 Gaseous Effluents - Summation of All Releases

| | UNITS | QUARTER 3 | QUARTER 4 | EST. ERROR TOTAL % |
|---|----------|---|---|--|
| A. Fission & activation gases | | destantiation and an interaction of the second second | Contractory of the second s | Anna and an and the second |
| 1. Total release | Ci | 5.25E3 | 2.08E3 | 18.8 |
| 2. Average release rate for period | µCi/sec | 6.67E2 | 2.65E2 | 18.8 |
| 3. Gamma Air Dose | Millirad | 1.40E-1 | 6.63E-2 | 18.8 |
| Percent of Technical Specification | % | 1.40E0 | 6.63E-1 | 18.8 |
| 4. Beta Air Dose | Millirad | 1.39E-1 | 6.10E-2 | 18.8 |
| Percent of Technical Specification | % | 6.95E-1 | 3.05E-1 | 18.8 |
| B. Iodines | | | | |
| 1. Total iodine-131 | Ci | 3.06E-2 | 4.34E-3 | 22.9 |
| 2. Average release rate for period | μCi/sec | 3.89E-3 | 5.52E-4 | 22.9 |
| 3. Critical Organ dose | Millirem | 1.19E-1 | 1.87E-2 | 22.9 |
| Percent of Technical Specification | % | 7.93E-1 | 1.25E-1 | 22.9 |
| C. Particulates | | | | |
| Particulates with half-lives greater than 8 days (includes Alpha and Strontium 89-90) | Ci | 3.48E-3 | 2.24E-3 | 22.9 |
| 2. Average release rate for period | μCi/sec | 4.43E-4 | 2.85E-4 | 22.9 |
| 3. Average Gross Alpha Radioactivity | Ci | 5.68E-6 | 2.26E-5 | 22.9 |
| D. Tritium | | | | |
| 1. Total release | Ci | 8.88E0 | 4.68E0 | 23.5 |
| 2. Average release rate for period | µCi/sec | 1.13E0 | 5.95E-1 | 23.5 |

Table 1B Page 1 of 2 Gaseous Effluents for Release Point - Main Stack

| | | Continu | ous Mode | Batch Mode | | |
|-------------------|-------|--------------|--------------|---|--------------|--|
| Nuclides Released | Units | Quarter 3 | Quarter 4 | Quarter 3 | Quarter 4 | |
| 1. Fission gases | | | | | | |
| Krypton - 85M | Ci | 1.51E2 | 7.40E1 | 0 | 0 | |
| Krypton - 87 | Ci | 1.85E2 | 1.25E2 | 0 | 0 | |
| Kryptca - 88 | Ci | 5.55E1 | 7.49E1 | 0 | 0 | |
| Xenon - 133 | Ci | 1.16E3 | 4.93E2 | 0 | 0 | |
| Xenon - 135 | Ci | 7.89E2 | 3.97E2 | 0 | 0 | |
| Xenon - 135M | Ci | 7.26E2 | 1.51E2 | 0 | 0 | |
| Xenon - 138 | Ci | 7.45E2 | 2.87E2 | 0 | 0 | |
| Xenon - 133M | Ci | 9.61E0 | ō | 0 | 0 | |
| Unidentified | Ci | 0 | 2.51E2 | 0 | 0 | |
| Total for Period | Ci | 3.82E3 | 1.85E3 | 0 | 0 | |
| 2. Iodines | | | | | | |
| Iodine - 131 | Ci | 1.93E-2 | 2.24E-3 | 0 | 0 | |
| Iodine - 133 | Ci | 5.58E-2 | 9.48E-4 | 0 | 0 | |
| Iodine - 135 | Ci | 1.74E-2 | 0 | 0 | 0 | |
| Total for Period | Ci | 9.25E-2 | 3.19E-2 | 0 | 0 | |
| 3. Particulates | | | | | | |
| Strontium - 89 | Ci | 1.35E-3 | 1.14E-3 | 0 | 0 | |
| Stroutium - 90 | Ci | 1.63E-6 | 2.32E-6 | 0 | 0 | |
| Strontium - 91 | Ci | 5.71E-4 | 1.04E-3 | 0 | 0 | |
| Cesium - 134 | Ci | 0 | 1.47E-5 | 0 | 0 | |
| Cesium - 137 | Ci | 3.79E-5 | 3.26E-5 | 0 | 0 | |
| Cesium - 138 | Ci | 6.89E-2 | 2.53E-2 | 0 | 0 | |
| Barium - 139 | Ci | 4.22E-3 | 1.32E-2 | and an end of the second se | 0 | |
| Barium - 140 | Ci | 3.95E-4 | 8.44E-4 | 0 | 0 | |
| Lanthanum - 140 | Ci | 2.38E-4 | 6.31E-4 | 0 | 0 | |
| Cobalt - 57 | Ci | 0 | 0 | 0 | 0 | |
| Cobali - 58 | Ci | 0 | 0 | 0 | 0 | |

Table 1B Page 2 of 2 Gaseous Effluents For Release Point - Main Stack

| | | Continuous Mode | | Batch Mode | |
|-------------------|-------|-----------------|--------------|--------------|--------------|
| Nuclides Released | Units | Quarter 3 | Quarter 4 | Quarter 3 | Quarter 4 |
| Cobalt - 60 | Ci | 0 | 5.57E-6 | 0 | 0 |
| Zinc - 65 | Ci | 0 | 0 | 0 | 0 |
| Yttrium - 91M | Ci | 5.73E-3 | 6.05E-4 | 0 | 0 |
| lodine - 133 | Ci | 9.88E-5 | 7.18E-6 | 0 | 0 |
| Copper - 64 | Ci | 0 | 0 | 0 | 0 |
| Rubidium - 88 | Ci | 0 | 0 | 0 | 0 |
| Manganese - 54 | Ci | 0 | 0 | 0 | 0 |
| Strontium - 92 | Ci | 0 | 0 | 0 | 0 |
| Iodine - 135 | Ci | 1.69E-5 | 0 | 0 | 0 |
| Tellurium - 132 | Ci | 4.27E-6 | 0 | 0 | 0 |
| Molybdenum - 99 | Ci | 9.94E-7 | 0 | 0 | 0 |
| Technetium - 99m | Ci | 1.51E-6 | 0 | 0 | 0 |
| | | | | | |
| | | | | | |
| TOTAL FOR PERIOD | Ći | 8.15E-2 | 4.28E-2 | 0 | 0 |

Table 1CPage 1 of 2Gaseous Effluents for Release Point - Unit 2 & Unit 3 Roof Vents

| | Continuous M | | ous Mode | Batch Mode | |
|-------------------|--------------|--------------|--------------|--------------|--------------|
| Nuclides Released | Units | Quarter 3 | Quarter 4 | Quarter 3 | Quarter 4 |
| 1. Fission gases | | | | | |
| Krypton - 85M | Ci | 0 | 0 | 0 | 0 |
| Krypton - 87 | Ci | 0 | 0 | 0 | 0 |
| Krypton - 88 | Ci | 0 | 0 | 0 | 0 |
| Xenon - 133 | Ci | 2.64E1 | 0 | 0 | 0 |
| Xenon - 135 | Ci | 4.74E1 | 0 | 0 | 0 |
| Xenon - 135M | Ci | 0 | 0 | 0 | 0 |
| Xenon - 138 | Ci | 0 | 0 | 0 | 0 |
| Unidentified | Ci | 4.39E2 | 2.25E2 | 0 | 0 |
| Total for Period | Ci | 5.13E2 | 2.25E2 | 0 | 0 |
| . Iodines | | | | | |
| Iodine - 131 | Ci | 1.13E-2 | 2.10E-3 | 0 | 0 |
| Iodine - 133 | Ci | 7.14E-2 | 1.18E-2 | 0 | 0 |
| Iodine - 135 | Ci | 1.15E-1 | 2.50E-2 | 0 | 0 |
| Total for Period | Ci | 1.98E-1 | 3.89E-2 | 0 | 0 |
| . Particulates | | | | | |
| Strontium - 89 | Ci | 5.80E-4 | 1.39E-4 | 0 | 0 |
| Strontium - 90 | Ci | 4.13E-6 | 3.07E-6 | 0 | 0 |
| Strontium - 91 | Ci | 1.21E-3 | 9.41E-5 | 0 | 0 |
| Cesium - 134 | Ci | 4.61E-5 | 0 | 0 | 0 |
| Cesium - 137 | Ci | 1.86E-4 | 3.12E-5 | 0 | 0 |
| Cesium - 138 | Ci | 4.09E-2 | 3.68E-3 | 0 | 0 |
| Barium - 139 | Ci | 8.65E-3 | 1.98E-3 | 0 | 0 |
| Barium - 140 | Ci | 4.96E-4 | 0 | 0 | 0 |
| Lanthanum - 140 | Ci | 6.09E-4 | 9.39E-5 | 0 | 0 |
| Cobalt - 57 | Ci | 0 | 0 | 0 | 0 |
| Cobalt - 58 | Ci | 3.41E-6 | 0 | 0 | 0 |
| Cobalt - 60 | Ci | 5.98E-5 | 0 | 0 | 0 |

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Table 1C Page 2 of 2 Gaseous Effluents For Release Point - Unit 2 & Unit 3 Roof Vents

| | | Continuous Mode | | Batch Mode | |
|-------------------|-------|-----------------|--------------|--------------|--------------|
| Nuclides Released | Units | Quarter 3 | Quarter 4 | Quarter 3 | Quarter 4 |
| Zinc - 65 | Ci | 0 | 0 | 0 | 0 |
| Yttrium - 21M | Ci | 3.73E-3 | 6.53E-5 | 0 | 0 |
| Iodine - 133 | Ci | 9.17E-4 | 3.64E-4 | 0 | 0 |
| Copper - 64 | Ci | 0 | 0 | 0 | 0 |
| Rubidium - 88 | Ci | 0 | 0 | Ŭ | 0 |
| Manganese - 54 | Ci | 0 | 0 | 0 | 0 |
| Strontium - 92 | Ci | 0 | 0 | 0 | 0 |
| Cadmium - 109 | Ci | 1.02E-4 | 0 | 0 | 0 |
| Technetium - 99m | Ci | 1.74E-5 | 10 | 0 | 0 |
| | | | | | |
| OTAL FOR PERIOD | Ci | 5.75E-2 | 6.45E-3 | 0 | 0 |

Table 2A Page 1 of 1 Liquid Effluents - Summation of All Releases

| | Units | Quarter 3 | Quarter 4 | Est. Error Total % |
|--|----------|--------------|--------------|-----------------------|
| A. Fission & activation products | | | | |
| 1. Total release (not including tritium, gases, alpha) | Ci | 2.61E-2 | 2.75E-2 | 22.9 |
| 2. Average diluted concentration during period | µCi/ml | 5.99E-11 | 7.39E-11 | 22.9 |
| 3. Total Body Dose ADULT BODY | Millirem | 3.10E-3 | 5.43E-3 | 22.9 |
| Percent of Technical Specification | H. | 1.03E-1 | 1.81E-1 | 22.9 |
| 4. Maximally Exposed Organ Dose TEEN LIVER | Millirem | 5.47E-3 | 9.72E-3 | 22.9 |
| Percent of Technical Specification | % | 5.47E-2 | 9.72E-2 | 22.9 |
| B. Tritium | | | | |
| 1. Total release | Ci | 1.88E0 | 3.37E-1 | 15.0 |
| 2. Average diluted concentration during period | µCi/ml | 4.31E-9 | 9.06E-10 | 15.0 |
| C. Dissolved and entrained gases | | | | |
| 1. Total release | Ci | 9.67E-3 | 1.49E-4 | 22.9 |
| 2. Average diluted concentration during period | µCi/ml | 2.22E-11 | 4.01E-13 | 22.9 |
| D. Gross alpha radioactivity | | | | |
| 1. Total release | Ci | 3.71E-5 | 6.24E-6 | 22.9 |
| 2. Average diluted concentration during period | µCi/ml | 8.51E-14 | 1.68E-14 | 22.9 |
| E. Volume of waste released (prior to dilution) | liters | 9.61E5 | 1.73E5 | 12.7 |
| F. Volume of dilution water used during period | liters | 4.36E11 | 3.72E11 | 10.9 |

Table 2B Page 1 of 1 Liquid Effluents

| | | Continue | ous Mode | Batch | Mode |
|-----------------------------|-------|--------------|--------------|--------------|--------------|
| Nuclides Released | Units | Quarter 3 | Quarter 4 | Quarter 3 | Quarter 4 |
| Strontium - 89 | Ci | 9.18E-6 | 9.94E-6 | 1.08E-4 | 3.21E-6 |
| Strontium - 90 | Ci | 5.33E-7 | 6.51E-7 | 3.08E-5 | 6.25E-7 |
| Alpha | Ci | 8.68E-7 | 9.93E-7 | 3.62E-5 | 5.25E-6 |
| Tritium | Ci | 4.51E-2 | 4.99E-2 | 1.83E0 | 2.87E-1 |
| Phosphorus - 32 | Ci | 6.19E-6 | 6.25E-6 | 1.64E-4 | 1.95E-5 |
| Iron - 55 | Ci | 3.51E-4 | 3.48E-4 | 5.53E-4 | 6.66E-5 |
| Xenon - 131M | Ci | 0 | 0 | 0 | 0 |
| Xenon - 133 | Ci | 0 | 0 | 3.41E-3 | 4.01E-5 |
| Xenon - 133M | Ci | 0 | 0 | 0 | 0 |
| Xenon - 135 | Ci | 0 | 0 | 6.22E-3 | 1.09E-4 |
| Xenon - 135M | Ci | 0 | 0 | 3.67E-5 | 0 |
| Krypton - 85M | Ci | 0 | 0 | 0 | 0 |
| Krypton - 87 | Ci | 0 | 0 | 0 | 0 |
| Krypton - 88 | Ci | 0 | 0 | 0 | 0 |
| Manganese - 54 | Ci | 3.33E-3 | 3.65E-3 | 2.72E-7 | 0 |
| Cesium - 134 | Ci | 2.27E-4 | 2.57E-4 | 4.64E-6 | 3.38E-6 |
| Cesium - 137 | Ci | 5.99E-4 | 6.53E-4 | 1.71E-5 | 1.20E-5 |
| Cesium - 138 | Ci | O | 0 | 0 | 0 |
| Zinc - 65 | Ci | 8.88E-3 | 9.75E-3 | 0 | 0 |
| Sodium - 24 | Ci | 0 | 0 | 0 | 0 |
| Cobalt - 58 | Ci | 9.18E-4 | 9.80E-4 | 0 | 0 |
| Cobalt - 60 | Ci | 1.04E-2 | 1.12E-2 | 1.82E-5 | 2.82E-6 |
| Chromium - 51 | Ci | 5.23E-4 | 5.07E-4 | 0 | 0 |
| TOTAL FOR PERIOD (ABOVE) | Ci | 7.03E-2 | 7.73E-2 | 1.84E0 | 2.87E-1 |

j.

PEACH BOTTOM UNITS 2 & 3 JULY 1, 1993 TO DECEMBER 31, 1993 CLASSES OF SOLID RADIOACTIVE WASTE SHIPMENTS

| Total # of Shipments | Waste Description (source of waste) | Container/Type | Individual Volume (cubic ft.) | Total Volume (cubic ft.) | Total Curie | Principal Radionuclides | | |
|-------------------------|---|----------------------------------|---|--------------------------------|----------------|---|--|--|
| Class A | Class A | | | | | | | |
| 19 | Dewatered Resin | HIC/Type A Cask | 202.1 | 3839.9 | 4.30E+02 | Zn-65, Co-60, Cs-137, Cs-134, Cr-51 | | |
| 1 | Dewatered Resin | HIC/Type A Cask | 132.4 | 132.4 | 1.78E+01 | Zn-65, Co-60, Cs-137, Mn-54, Cs-134 | | |
| 13 | DAW | Metal Drum/STC, Metal Box/STC | variable | (*) 207.6 | 1.13E-01 | Co-60,Zn-65, Fe-55, Cs-137, Mn-54 | | |
| 71 | DAW | Metal Drum/STC, Metal Box/STC | variable | (**) 856.3 | 1.00E+00 | Co-60, Zn-65, Fe-55, Cs-137, Mn-54 | | |
| 1 | DAW | HIC/Type A Cask | 38.3 | 114.9 | 3.37E+00 | Co-60, Fe-55, Zn 65, Ni-63 | | |
| 1 | DAW | HIC/Type A Cask | 205.8 | 205.8 | 1,41E+00 | Co-60, Cs-137, Zn-65, Fe-55, Mn-54 | | |
| CLASS B | | | | | | | | |
| 8 | Dewatered Resin | HIC/Type A Cask | 202.1 | 1616.8 | 3.06E+02 | Zn-65, Cs-137, Co-60, Cs-134, Fe-55 | | |
| 2 | Dewatered Filters | HIC/Type A Cask | 38.3 | 229.8 | 3.20E+01 | Co-60, Fe-55, Zn-65, Ni-63 | | |
| CLASS C | desentation of the second s | | 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - | | | | | |
| 3 | Irradiated H'Ware | Steel Liner/ Type B Cask | 57.8 | 173.4 | 6.03E+04 | Fe-55, Co-60, Mn-54, Ni-63, Cr-51 | | |
| TOTALS | | | | 7376.9 | 6.11E+04 | | | |

NOTES:

* - Indicates actual total PECO radwaste shipped from Quadrex, after volume reduction, to the burial site.

** - Indicates actual total PECO radwaste shipped from SEG, after volume reduction, to the burial site.

ATTACHMENT A SUPPLEMENT INFORMATION

Facility: Peach Bottom Units 2 & 3

Licenses: DPR-44 DPR-56

1.

Regulatory Limits (Technical Specification Limits)

A. Noble Gases:

| 1. | ≤500 ≤3000 | mRem/Yr mRem/Yr | - total body - skin | ÷ ,^ | "instantaneous" limits Tech. Spec. 3.8.C.1.a |
|----|------------------------|--------------------|---------------------------|------|--|
| 2. | ≤ 10 ≤ 20 | mRad mRad | - air gamma - air beta | | quarterly air dose limits Tech. Spec. 3.8.C.2.a |
| 3. | ≤20 ≤40 | mRad mRad | - air gamma - air beta | * | yearly air dose limits Tech. Spec. 3.8.C.2.b |

B. Iodines, Tritium, Particulates with Half Life >8 days:

| 1. | ≤1500 | mRem/Yr (inhalation path) | - any organ | 121 * | "instantaneous" limits Tech. Spec. 3.8.C.1.b |
|----|-------|------------------------------|-------------|-------|---|
| 2. | ≤15 | mRem | - any organ | 16 | quarterly dose limits Tech. Spec. 3.8.C.3.a |

- yearly dose limits

Tech. Spec. 3.8.C.3.b

≤30 mRem - any organ

- C. Liquid Effluents
 - 1.Concentration ≤ 10 CFR 20,
Appendix B, Table II, Col. 2- "instantaneous" limits
Tech. Spec. 3.8.B.1

| 2 | ≤ 3.0 ≤ 10 | mRem mRem | - total body - any organ | - quarterly dose limits Tech. Spec. 3.8.B.2.a |
|----|-------------------------|--------------|-------------------------------|--|
| 3. | ≤6.0 | mRem | - total body | - yearly dose limits |
| | ≤ 20 | mRem | any organ | - Tech. Spec. 3.8.B.2.b |

2. Maximum Permissible Concentrations:

MPCs are not used to calculate permissible release rates and concentrations for gaseous releases.

The MPCs specified in 10 CFR 20, Appendix B, Table II, Column 2, for identified nuclides are used to calculate permissible release rates and concentrations for liquid release per Peach Bottom Technical Specification 3.8.B.1.

3. Average Energy:

Not Applicable

- Measurements and Approximations of Total Radioactivity:
 - A. Fission and Activation Gases:

The method used is the Nuclear Data 6700 Counting System or the Canberra S95/Dual Host 3400 Counting System - Gas Marinelli -

B. Iodine:

The method used is the Nuclear Data 6700 Counting System or the Canberra S95/Dual Host 3400 Counting System - Charcoal Cartridge -

C. Particulates:

The method used is the Nuclear Data 6700 Counting System or the Canberra S95/Dual Host 3400 Counting System - Air Particulate Sample, (47mm and 57mm filters) -

D. Liquid Effluents:

The method used is the Nuclear Data 6700 Counting System and the Radwaste Liquid Discharge Pre-Release Method with a liter marinelli

5. Batch Releases:

A. Liquid:

| | QIKS | QIK 4 |
|--|---------|--------|
| Number of batch releases: | 15 | 7 |
| Total time for batch releases (minutes): | 3506 | 893 |
| Maximum time period for batch release (minutes): | 368 | 260 |
| Average time period for batch release (minutes): | 234 | 128 |
| Minimum time period for batch release (minutes): | 43 | 65 |
| Dilution volume (liters): | 1.37E10 | 2.28E9 |

B. Gaseous:

Not applicable

6. Abnormal Releases:

A. Liquid:

1. Event description - On 7/23/93 routine sampling of the high pressure service water (HPSW) effluent to the discharge canal detected low level radioactive contamination. Subsequent investigation determined that a trace amount of condensate stay full or primary coolani water was leaking past the Unit 2 'B' residual heat removal (RHR) heat exchanger floating head gasket into the 'B' loop of the HPSW system. The gasket was replaced and leak tested satisfactorily.

It was estimated that the contaminated water released 7.65E-3 millirem total body dose to the discharge canal from 7/23/93 to 12/7/93. This dose contribution was well below the limits specified in Technical Specifications.

Analysis of Release - The representative sample obtained was analyzed for all the parameters of a radioactive effluent release. The results were then calculated based on the actual release conditions. The Dose contributions and Isotope quantities from this continuous release were added to this Semi-Annual Effluent Report for the applicable reporting period.

2. Event description - On 11/10/93 routine sampling of the high pressure service water (HPSW) effluent to the discharge canal detected low level radioactive contamination. Subsequent investigation determined that a trace amount of condensate stay full or primary coolant water was leaking past the Unit 2 'A' residual heat removal (RHR) heat exchanger floating head gasket into the 'A' loop of the HPSW system. The gasket was replaced and leak tested satisfactorily.

It was estimated that the contaminated water released 6.41E-4 millirem total body dose to the discharge canal from 11/10/93 to 1/9/94. This dose contribution was well below the limits specified in Technical Specifications.

Analysis of Release - The representative sample obtained was analyzed for all the parameters of a radioactive effluent release. The results were then calculated based on the actual release conditions. The Dose contributions and Isotope quantities from this continuous release were added to this Semi-Annual Effluent Report for the applicable reporting period.

Abnormal Releases: (Continued)

B. Gaseous:

6.

PBAPS LER 2-93-15

Event Description - While performing the weekly filter change on December 1, 1993 the Main Stack Particulate Filter and Charcoal Cartridge Assembly was found to be improperly secured, resulting in diluted sample flow. Accordingly, representative sampling of Main Stack particulate, iodine, and fission gas releases was not obtained for the 7-day period from November 24 through December 1, 1993.

Analysis of Release - Based on existing radiological controls and current and previous plant experience, it is believed that no abnormal particulate, iodine, or fission gas releases occurred. Release values during the unmonitored period were calculated by simply extending the before and after release rates into the unmonitored period.